ACSMS-016

OCT 31 1983

Docket Nos. 50-266
and 50-301

Mr. C. W. Fay
Vice President - Nuclear Power
Wisconsin Electric Power Company
231 West Michigan Street
Milwaukee, Wisconsin 53201

Dear Mr. Fay:

The Commission has issued the enclosed Amendment No. 78 to Facility Operating License No. DPR-24 and Amendment No. 82 to Facility Operating License No. DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated April 27, 1982 modified by letter dated March 23, 1983.

These amendments revise the loss of voltage relay setpoints and associated time delays in Table 15.3.5-1 of the Technical Specifications.

As discussed with and agreed to by members of your staff, these relays will be installed during the next cold shutdown for Units 1 and 2. The next scheduled cold shutdowns for Units 1 and 2 end approximately the end of March 1984 and mid-November 1984 respectively. You have indicated that, if a unit is placed in cold shutdown prior to these outages, these relays would be installed. Please keep us informed in this regard.

A copy of the Safety Evaluation is enclosed. The notice of issuance will be included in the Commission's next monthly Federal Register notice.

Also enclosed is a corrected copy of TS page Table 15.4.1-1 which was issued with Amendment Nos. 76 and 80 on October 6, 1983. The correction consists of changing the word "shutdown" to "interval" in the definition of surveillance

frequency code "R". The change was discussed and approved in the Safety Evaluation supporting Amendments 76 and 80; however, it was overlooked at the time of issuance of the TS changes.

Sincerely,

Original signed by

Timothy G. Colburn, Project Manager
Operating Reactors Branch #3
Division of Licensing

Enclosures:
1. Amendment No. 78 to DPR-24
2. Amendment No. 82 to DPR-27
3. Safety Evaluation

cc w/enclosures:
See next page

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Amendment + FR Notice

Wisconsin Electric Power Company

cc:
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Chairman
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 78
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated April 27, 1982 as modified March 23, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 78 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective following installation of the new relays but no later than March 1984.

FOR THE NUCLEAR REGULATORY COMMISSION

James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 31, 1983

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 82
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated April 27, 1982 as modified March 23, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 82 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective following installation of the new relays but no later than November 1984.

FOR THE NUCLEAR REGULATORY COMMISSION

James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 31, 1983

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 78 TO FACILITY OPERATING LICENSE NO. DPR-24

AMENDMENT NO. 82 TO FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NO. 50-266 AND 50-301

Revise Appendix A as follows:

Remove Pages	Insert Pages
Table 15.3.5-1 page 2 of 2	Table 15.3.5-1 page 2 of 2
Table 15.4.1-1 (continued)	Table 15.4.1-1 (continued)

TABLE 15.3.5-1 (Continued)
(Page 2 of 2)

NO.	FUNCTIONAL UNIT	CHANNEL	SETTING LIMIT
9.	Degraded Voltage (4.16 KV)	Disconnection of affected bus from offsite power	3875 volts ± 2% Time delay: less than 60 sec at 0-100% of voltage setting
10.	Loss of Voltage		
	a. 4.16 KV	Disconnection of affected bus from offsite power	a. 3220 volts ± 2% Time delay: ≤1 sec ± 10% at 0-100% of voltage setting**
	b. 480 V	Load shedding	b. 256 volts ± 3% Time delay: ≤ .75 sec ± 10% at 0 volts ≤3.5 sec ± 20% at 90% of voltage setting**

** This limit is effective upon installation of the associated relays but in no case later than upon return to power from the Fall 1983 refueling outage for Unit 1 and the Fall 1984 refueling outage for Unit 2.

S - Each shift
D - Daily
W - Weekly
B/W - Biweekly
Q - Quarterly
M - Monthly
P - Prior to each startup if not done previous week.
R - Each Refueling Interval (But not to exceed 18 months).
N.A. - Not applicable.

** Not required during periods of refueling shutdown, but must be performed prior to starting up if it has not been performed during the previous surveillance period.

*** Not required during periods of refueling shutdown if steam generator vessel temperature is greater than 70°F.

**** When used for the overpressure mitigating system each PORV shall be demonstrated operable by:

a. Performance of a channel functional test on the PORV actuation channel, but excluding valve operation, within 31 days prior to entering a condition in which the PORV is required operable and at least once per 31 days thereafter when the PORV is required operable.

b. Testing valve operation in accordance with the inservice test requirements of the ASME Boiler and Pressure Vessel Code, Section XI.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 78 TO FACILITY OPERATING LICENSE NO. DPR-24

AND AMENDMENT NO. 82 TO FACILITY OPERATING LICENSE NO. DPR-27

WISCONSIN ELECTRIC POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-266 AND 50-301

INTRODUCTION

By letter dated April 27, 1982 (Ref. 3), as modified March 23, 1983 (Ref. 1), Wisconsin Electric Power Company (WEPC) requested a revision to the Point Beach Unit Nos. 1 and 2 Technical Specifications (item 10 of Table 15.3.5-1) to change the undervoltage setpoint and the trip time delay of the loss of voltage relays. The proposed changes are in response to the staff's Safety Evaluation dated January 6, 1982 (Ref. 2) on the interim change request for the degraded voltage relay setpoints.

In Reference 2, the staff identified that the existing setpoints of the loss of voltage relays would not adequately protect the Class 1E equipment from grid undervoltage during the time delay of degraded voltage relays. Subsequently, in a letter dated April 27, 1982 (Ref. 3), the licensee identified that they had misinterpreted the characteristic curves of the loss of voltage and load shedding relays and submitted an application to amend the Technical Specifications. The time delays for 0 volts and also 90% of the voltage setpoints were changed for both sets of loss of voltage relays. Before the completion of the staff review of the proposed changes to the Technical Specifications, the licensee proposed to change the type and the setpoints of the loss of voltage relays per their letter dated June 1, 1982 (Ref. 4). This necessitated a revision to the proposed changes to the Technical Specifications which had been submitted by the licensee in Reference 1.

EVALUATION

The present Technical Specifications list 2450 volts ± 3% (min. 57% of 4160) as the setpoint for the 4.16 kV loss of voltage relays. The associated time delay is 0.3 seconds for 0 volts and 1.2 second for 90% of the voltage setting with a ± 5% drift. Since the one minute rating of all Class 1E motors is 75% terminal voltage (approximately 72% of respective bus voltage), and the time delay of degraded voltage trip setpoint is a maximum of 60 seconds, the equipment is not adequately protected against undervoltage in the bus voltage range between 57% and 72% of 4160 volts for about one minute. This has been discussed in detail in Reference 2. Also, since the existing loss

of voltage relays are induction disc type with inverse time undervoltage characteristics, they cannot be set to a voltage/time curve that would provide trip above 72% of bus voltage and still permit starting large motors with the minimum calculated 4160 volt bus voltage (Ref. 4). The licensee has, therefore, selected a definite trip time, solid state relay (ITE-27D) which is capable of achieving both the above requirements. The Technical Specification change (Reference 1) proposes the loss of voltage relay setpoint to be 3220 volts ± 2% (min. 3220 - 64 = 76% of 4160 volts) with a maximum time delay of 1 sec. ± 10% from 0-to 100% of voltage setting. The proposed voltage setpoint will protect Class 1E equipment from any undervoltage below its one minute rating with a 4% safety margin and prevent spurious trips when starting large motors.

The revised time delay tolerance of both the 4.16 kV and 480 V loss of voltage relays is based on the correct interpretation of the voltage/time characteristic curves as identified by the licensee in Reference 3. Therefore, the relays will be accurately calibrated for the correct tolerance and spurious trips will not result.

Based on the above evaluation, the staff finds that the proposed changes to the Technical Specifications (item 10 of Table 15.3.5-1) will ensure adequate protection of Class 1E equipment from grid undervoltage conditions and are therefore acceptable.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: October 31, 1983

Principal Contributor:
T. Colburn
I. Ahmed

References

1. WEPC Letter (C. W. Fay) to NRC (H. R. Denton) dated March 23, 1983.
2. NRC Letter (T. G. Colburn) to WEPC (Sol Burstein) dated January 6, 1982.
3. WEPC Letter (C. W. Fay) to NRC (H. R. Denton) dated April 27 1982.
4. WEPC Letter (C. W. Fay) to NRC (H. R. Denton) dated June 1 1982.