

ACS MS-016

Docket Nos. 50-266
and 50-301

DEC 28 1983

Mr. C. W. Fay
Vice President - Nuclear Power
Wisconsin Electric Power Company
231 West Michigan Street
Milwaukee, Wisconsin 53201

Dear Mr. Fay:

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The Commission has issued the enclosed Amendment No. 79 to Facility Operating License No. DPR-24 and Amendment No. 84 to Facility Operating License No. DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated December 15, 1982.

These amendments add an additional reporting requirement in the Technical Specifications to report all challenges to the pressurizer power operated relief valves and pressurizer safety valves in the annual report.

These changes and others were requested by NRC Generic Letter 82-16 dated September 20, 1982. This letter addressed several NUREG-0737 items for which the NRC staff had determined that Technical Specification changes may be necessary to achieve full implementation. These items were scheduled for implementation prior to December 31, 1981. In your response to Generic Letter 82-16 you stated that all items except one were satisfied by current Technical Specifications or were under NRC review as a license amendment. However, the Table provided on page 2 of your response describes the status of item I.A.1.3 (Limit Overtime) as being addressed by administrative procedures. We disagree that administrative procedures are adequate to address this issue and request that you submit an application for amendment as requested in Generic Letter 82-16 to address this issue. All other items appear to be addressed as stated in your response.

A copy of the Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next monthly Federal Register notice.

Sincerely,

Original signed by

Timothy G. Colburn, Project Manager
Operating Reactors Branch #3
Division of Licensing

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Enclosures:

1. Amendment No. 79 to DPR-24
2. Amendment No. 84 to DPR-27
3. Safety Evaluation

cc w/enclosures:
See next page

ORB#3:DL
PMKreutzer
12/1/83

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TCoburn/pn
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ORB#5:DL
PERickson
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ORB#3:DL
JRMilner
12/1/83

OELD
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AD:OR:DL
GCLainas
12/1/83

Wisconsin Electric Power Company

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 79
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated December 15, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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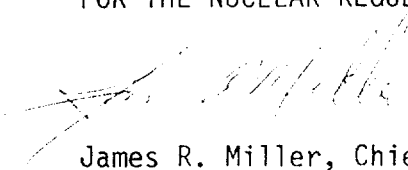
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 79, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 28, 1983



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 84
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated December 15, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 84, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 28, 1983

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 79 TO FACILITY OPERATING LICENSE NO. DPR-24

AMENDMENT NO. 84 TO FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NOS. 50-266 AND 50-301

Revise Appendix A as follows:

Remove Page

15.6.9-2

Insert Page

15.6.9-2

- a. 90 days following completion of the startup tests.
- b. 90 days following resumption or commencement of commercial power operation.
- c. 9 months following initial criticality.

B. Annual Results and Data Report

- 1. A results and data report covering the period of the previous calendar year shall be submitted prior to March 1 of each year.
- 2. This report shall include:
 - a. Complete results of steam generator tube inservice inspection completed during the calendar year as required by specification 15.4.2.c.2.
 - b. A tabulation on an annual basis of the number of station, utility, and other personnel receiving exposures greater than 100 mrem/year and their associated man-rem exposure according to work and job functions. The dose assignments to various duty functions may be estimates based on pocket dosimeter, TLD or film badge measurements. Small exposures totalling less than 20 percent of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.
 - c. A description of facility changes, tests or experiments as required pursuant to 10 CFR 50.59(b).
 - d. A tabulation of all challenges to the pressurizer power operated relief valves or pressurizer safety valves.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 79 TO FACILITY OPERATING LICENSE NO. DPR-24
AND AMENDMENT NO. 84 TO FACILITY OPERATING LICENSE NO. DPR-27
WISCONSIN ELECTRIC POWER COMPANY
POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-266 AND 50-301

I. Introduction

A. Description of the Proposed Action

This Safety Evaluation (SE) evaluates Technical Specification (TS) change proposed by Wisconsin Electric Power Company (licensee) for the Point Beach facility in response to the NRC's Generic Letter (GL) No. 82-16, "NUREG-0737 Technical Specifications".

B. Background Information

Following the accident at Three Mile Island, Unit 2, the staff developed an NRC Action Plan, NUREG-0660, to provide a comprehensive and integrated plan to improve safety at power reactors. Specific NUREG-0660 items, approved by the Commission for implementation at power reactors, were issued as NUREG-0737. NUREG-0737 specified that new TSs would be required for several of the items. Accordingly, on September 20, 1982, the NRC issued GL 82-16, requesting all pressurized water reactor licensees to (1) review their existing TSs against the guidance provided in GL 82-16, and (2) to submit proposed TSs for those items deviating from the NRC's guidance. GL 82-16 requested information on the following NUREG-0737 items:

1. STA Training (I.A.1.1.3).
2. *Shift Manning - Overtime Limits (I.A.1.3.1).
3. Short Term Auxiliary Feedwater System (AFWS) Evaluation (II.E.1.1).
4. Safety Grade AFW Initiation and Flow Indication (II.E.1.2).
5. Dedicated Hydrogen Penetrations (II.E.4.1).
6. Containment Pressure Setpoint (II.E.4.2.5).
7. Containment Purge Valves (II.E.4.2.6).
8. *Radiation Signal on Purge Valves (II.E.4.2.7).
9. Upgrade B&W AFWS (II.K.2.8).
10. *B&W Safety-Grade Anticipatory Reactor Trip (II.K.2.10).
11. B&W Thermal-Mechanical Report (II.K.2.13).
12. *Reporting Safety and Relief Valve Failures and Challenges (II.K.3.3).
13. *Anticipatory Trip on Turbine Trip (II.K.3.12).

*Model TS provided.

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C. Scope of Review

The staff's review consisted of a verification of the status of each of the above items and an evaluation of the licensee's proposed TSs against the model TSs provided in Generic Letter 82-16 and other guidance related to NUREG-0737. For the Point Beach facility, only the above Item 12 is evaluated in this SE. The remaining items are not being evaluated in this SE because either (1) the item has not been completed at the facility, (2) the item has been previously closed out by the staff for the facility, (3) the staff position has not been finalized for the item, (4) the item does not apply to the Point Beach facility, or (5) the licensee did not respond to the request in Generic Letter 82-16. An explanation of each of the items which will not be evaluated in this SE is given below.

1. STA Training (I.A.1.1.3)

In a previous submittal of December 23, 1980 (Ref. 1), the licensee committed to a training program for STAs. Implementation of the training was verified by onsite inspection (Inspection Reports Nos. 50-266/81-13, 50-301/81-15 and 50-266/81-15, 50-301/81-17). Our July 2, 1980 letter provided model TSs for TMI lessons learned category "A" items. Included were TSs that specified the qualifications, training and on-duty requirements for the Shift Technical Advisors (STAs). However, the STA training requirements are now under further consideration by the Commission, and no action will be taken to amend the TSs until further guidance is provided by the Commission.

2. Shift Manning - Overtime Limits (I.A.1.3.1)

The licensee did not respond to the request in Generic Letter 82-16 and instead asserted that administrative procedures were in place to adequately address this item. The licensee should submit an amendment application to incorporate the overtime limits in the Technical Specifications for Point Beach Units 1 and 2.

3. Short Term AFWS Evaluation (II.E.1.1)

The licensee made a submittal on April 27, 1982 (Ref. 2), later clarified by letter of May 27, 1982 (Ref. 3). System modifications are in the process of being made, after which TSs will be issued. Modifications are to be installed after the refueling for Unit 2, scheduled for June, 1983 and the Fall, 1983 refueling for Unit 1.

4. Safety Grade AFWS Initiation and Flow Indication (II.E.1.2)

Auxiliary feedwater system automatic initiation was a feature of the original design for Point Beach. Safety grade flow indication was installed in the lines to each steam generator in response to this item. The licensee made a submittal on April 27, 1982 (Ref. 2), later clarified by letter of May 27, 1982 (Ref. 3). System modifications are in the process of being made, after which TSs will be issued. Modifications are to be installed after the refueling for Unit 2 scheduled for June, 1983 and the Fall, 1983 refueling for Unit 1.

5. Dedicated Hydrogen Penetrations (II.E.4.1)

Our letter of April 9, 1980 (Ref. 4) acknowledged that the licensing basis for Point Beach (original design) consists of a hydrogen purge system that is single failure proof and sized to meet the flow requirements of the system during an accident. It also noted that the plant does not have a recombiner system. By letter of September 14, 1981 the staff documented the acceptance of this system. The existence of the dedicated penetrations was verified by onsite inspection (Inspection Reports Nos. 50-266/81-13, 50-301/81-15, and 50-266/82-01, 50-301/82-01). The valves in the penetration lines are tested under the requirements of 10 CFR 50 Appendix J.

6. Containment Pressure Setpoint (II.E.4.2.5)

By letters dated December 23, 1980 (Ref. 1), and May 7, 1981 (Ref. 5), the licensee responded to this item. The NRC staff, by letter of August 4, 1981, indicated that the existing pressure setpoint specified in the plant TS is satisfactory. This was verified by onsite inspection (Inspection Report Nos. 50-266/81-15, 50-301/81-17).

7. Containment Purge Valves (II.E.4.2.6)

The licensee's previous submittals of August 28, 1981 (Ref. 6) and January 28, 1982 (Ref. 7) addressed the operation and surveillance testing requirements for the containment purge supply and exhaust system isolation valves. The staff reviewed the proposed TSs, and indicated that they were acceptable by letter of October 4, 1982 (License Amendment Nos. 64/69).

8. Radiation Signal on Purge Valves (II.E.4.2.7)

The licensee's previous submittal of December 23, 1980 (Ref. 1), indicated that the automatic closure of the containment purge and vent isolation valves upon receipt of a high radiation signal was a part of the original plant design for the large containment purge supply and exhaust valves, and no modifications were necessary. The licensee indicated that small 1-inch

ventilation isolation gate valves do not close on high radiation signals, but do close on receipt of a containment isolation signal. This position was found acceptable by NRC letter of October 4, 1982.

9. Upgrade B&W AFWS (II.K.2.8)

This item pertains to B&W facilities only, and is not applicable to this facility.

10. B&W Safety-Grade Anticipatory Reactor Trip (II.K.2.10)

This item pertains to B&W facilities only, and is not applicable to this facility.

11. B&W Thermal-Mechanical Report (II.K.2.13)

This item pertains to B&W facilities only, and is not applicable to this facility.

13. Anticipatory Trip on Turbine Trip (II.K.3.12)

An anticipatory reactor trip on turbine trip is a part of the licensing basis (original plant design) for Point Beach, and is described in TS Section 15.3.5.

II. Evaluation

By letter dated December 15, 1982 the licensee submitted proposed Change Request No. 86 (Ref. 8) to the Point Beach Unit Nos. 1 and 2 TS. The staff's evaluation of the proposed TS for GL 82-16 Item 12 (II.K.3.3) Reporting of Safety Valve and Relief Valve Failures and Challenges, is as follows:

A. GL 82-16, Item 12

The licensee, in response to this item, has proposed a TS change, which will add reporting requirements to TS Section 15.6.9.1.B.2.d, requiring the licensee to include challenges to the pressurizer power operated relief valves or pressurizer safety valves to a previously required annual report. Reports of failures of pressurizer power operated relief valves were already required by TS Section 15.6.9.A.1. While not following the format of the suggested model TS, the proposed change meets the intent. We find that the licensee has fulfilled our request in formalizing this reporting requirement, and therefore this change is acceptable.

III. Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this

determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact, and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

IV. Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 28, 1983

Principal Contributors:

T. N. Tambling
J. E. Foster
T. G. Colburn

References

1. Licensee submittal of December 23, 1980, C. Fay (WEPC) to H. Denton (NRR).
2. Licensee submittal of April 27, 1982, C. Fay (WEPC) to H. Denton (NRR).
3. Licensee submittal of May 27, 1982, C. Fay (WEPC) to H. Denton (NRR).
4. NRC letter of April 9, 1980, Schwencer (NRR) to Burstein (WEPC).
5. Licensee submittal of May 7, 1981, C. Fay (WEPC) to H. Denton (NRR).
6. Licensee submittal of August 28, 1980, Burstein (WEPC) to H. Denton (NRR).
7. Licensee submittal of January 28, 1982, C. Fay (WEPC) to H. Denton (NRR).
8. Licensee submittal of December 15, 1982, C. Fay (WEPC) to H. Denton (NRR).