

June 17, 1986

Docket Nos. 50-266
and 50-301

Mr. C. W. Fay, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room 308
Milwaukee, Wisconsin 53201

Dear Mr. Fay:

The Commission has issued the enclosed Amendment Nos. 100 and 103 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2. The amendments consist of changes to the Technical Specifications in response to your application dated August 8, 1985.

These amendments add additional surveillance requirements for reactor trip breakers. The amendments also add reference to maintenance of sampling and analysis equipment as part of the Post-Accident Sampling Program and correct two typographical errors.

A copy of the Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

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Timothy G. Colburn, Project Manager
PWR Project Directorate #1
Division of PWR Licensing-A

Enclosures:

1. Amendment No. 100 to DPR-24
2. Amendment No. 103 to DPR-27
3. Safety Evaluation

cc w/enclosures:
See next page

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6/6/86

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Mr. C. W. Fay
Wisconsin Electric Power Company

Point Beach Nuclear Plant
Units 1 and 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 100
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated August 8, 1985 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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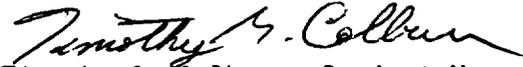
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 100, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Timothy G. Colburn, Project Manager
PWR Project Directorate #1
Division of PWR Licensing-A

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 17, 1986.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 103
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated August 8, 1985 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 103, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Timothy G. Colburn, Project Manager
PWR Project Directorate #1
Division of PWR Licensing-A

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 17, 1986.

ATTACHMENT TO LICENSE AMENDMENT NOS. 100 AND 103
TO FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27
DOCKET NOS. 50-266 AND 50-301

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

Table 15.4.1-2 page 2 of 3
Table 15.4.1-2 page 3 of 3
15.6.8-3
15.6.9-5

INSERT

Table 15.4.1-2 page 2 of 3
Table 15.4.1-2 page 3 of 3
15.6.8-3
15.6.9-5

TABLE 15.4.1-2 (Continued)

	<u>Test</u>	<u>Frequency</u>
9. Control Rods	Rod drop times of all full length rods (3)	Each refueling or after maintenance that could affect proper functioning ⁽⁴⁾
10. Control Rod	Partial movement of all rods	Every 2 weeks ⁽⁶⁾
11. Pressurizer Safety Valves	Set point	Each refueling shutdown
12. Main Steam Safety Valves	Set point	Each refueling shutdown
13. Containment Isolation Trip	Functioning	Each refueling shutdown
14. Refueling System Interlocks	Functioning	Each refueling shutdown
15. Service Water System	Functioning	Each refueling shutdown
16. Primary System Leakage	Evaluate	Monthly (6)
17. Diesel Fuel Supply	Fuel inventory	Daily
18. Turbine Stop and Governor Valves	Functioning	Monthly (6)(10)
19. Low Pressure Turbine Rotor Inspection (5)	Visual and magnetic particle or liquid penetrant	Every five years
20. Boric Acid System	Storage Tank temperature	Daily
21. Boric Acid System	Visual observation of piping temperatures (all $\geq 145^{\circ}\text{F}$)	Daily
22. Boric Acid Piping Heat Tracing	Electrical circuit operability	Monthly
23. PORV Block Valves	Complete Valve Cycle	Quarterly (6)

TABLE 15.4.1-2 (Continued)

	<u>Test</u>	<u>Frequency</u>
24.	Integrity of Post Accident Recovery Systems Outside Containment	Evaluate Each refueling cycle
25.	Containment Purge Supply and Exhaust Isolation Valves	Verify valves are locked closed Monthly (9)
26.	Reactor Trip Breakers	a. Verify independent operability of automatic shunt and undervoltage trip functions. Monthly (9)
		b. Verify independent operability of manual trip to shunt and undervoltage trip functions. Each refueling shutdown
27.	Reactor Trip Bypass Breakers	a. Verify operability of the undervoltage trip function. Prior to breaker use
		b. Verify operability of the shunt trip functions. Each refueling shutdown
		c. Verify operability of the manual trip to undervoltage trip functions. Each refueling shutdown

- (1) Required only during periods of power operation.
- (2) E determination will be started when the gross activity analysis of a filtered sample indicates $\geq 10\mu\text{Ci/cc}$ and will be redetermined if the primary coolant gross radioactivity of a filtered sample increases by more than $\geq 10\mu\text{Ci/cc}$.
- (3) Drop test shall be conducted at rated reactor coolant flow. Rods shall be dropped under both cold and hot condition, but cold drop tests need not be timed.
- (4) Drop tests will be conducted in the hot condition for rods on which maintenance was performed.
- (5) As accessible without disassembly of rotor.
- (6) Not required during periods of refueling shutdown.
- (7) At least once per week during periods of refueling shutdown.
- (8) At least three times per week (with maximum time of 72 hours between samples) during periods of refueling shutdown.
- (9) Not required during periods of cold or refueling shutdown.
- (10) During end-of-cycle period of operation when boron concentration is less than 100 ppm, this test may be waived due to operational limitations.
- (11) Sample to be taken after a minimum of 2 EFPD and 20 days power operation since the reactor was last subcritical for 48 hours or longer.

Unit 1 - Amendment No. 6A, 6B, 71, 83, 100

Unit 2 - Amendment No. 69, 70, 76, 87, 103

15.6.8 PLANT OPERATING PROCEDURES (Continued)

15.6.8.4 The following programs shall be established, implemented, and maintained.

A. Post-Accident Sampling*

A program**which will ensure the capability to obtain and analyze reactor coolant, containment atmosphere, and in-plant gaseous effluent samples under accident conditions. The program shall include the following:

- (i) Training of personnel;
- (ii) Procedures of sampling and analysis; and
- (iii) Provisions for maintenance of sampling and analysis equipment.

*Post-Accident Coolant Sampling and Post-Accident Containment Atmospheric Sampling Systems.

**It is acceptable if the licensee maintains details of the program in plant operation manuals.

E. Failure of Containment High-Range Radiation Monitor

A minimum of two in-containment radiation-level monitors with a maximum range of 10^8 rad/hr (10^7 /hr for photons only) should be operable at all times except for cold shutdown and refueling outages. This is specified in Table 15.3.5-5, item 7. If the minimum number of operable channels are not restored to operable condition within seven days after failure, a special report shall be submitted to the NRC within thirty days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to operable status.

F. Failure of Main Steam Line Radiation Monitors

If a main steam line radiation monitor (SA-11) fails and cannot be restored to operability in seven days, prepare a special report outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the channel to operable status within thirty days of the event.

Unit 1 - Amendment No. 77, 82, 87, 100

Unit 2 - Amendment No. 76, 86, 101, 103



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 100 AND 103 TO
FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27
WISCONSIN ELECTRIC POWER COMPANY
POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-266 AND 50-301

INTRODUCTION

Generic Letter 83-28 was issued by NRC on July 8, 1983, indicating actions to be taken by licensees based on the generic implication of the Salem ATWS events. Item 4.3 of the generic letter requires that modifications be made to improve the reliability of the reactor trip system by implementation of an automatic actuation of the shunt attachment on the reactor trip breakers. By letter dated June 1, 1984, Wisconsin Electric Power Company (licensee) responded to Generic Letter 83-28. The staff issued a safety evaluation (SE) on September 26, 1984 indicating the acceptable and unacceptable aspects of the design and requested the licensee to resubmit it for staff's approval. By letters dated February 28, and August 8, 1985, the licensee resubmitted the proposed design for the incorporation of the automatic actuation of the shunt trip attachments for the reactor trip breakers, and also requested Technical Specification (TS) changes which would add new surveillance requirements for the reactor trip breakers.

The shunt trip modifications including the main control board bypass breaker position indication for Unit 1 were completed during the Spring 1985 outage. The shunt trip modifications for Unit 2 were completed during the Fall 1984 outage.

EVALUATION

The licensee proposed a revision to the TS surveillance requirements of Table 15.4.1-2, page 3 of 3 which was responsive to the surveillance parameters noted in the staff's safety evaluation of September 26, 1984. The staff has found that the proposed TS address independent testing of the undervoltage and shunt trip attachments during power operation for reactor trip breakers and independent testing of the control room manual switch contacts during each refueling outage. Further, the proposed technical specifications address testing to verify the operability of the undervoltage trip function of the bypass breakers prior to use for surveillance tests for reactor trip breakers and testing to verify operability of the shunt trip function of

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the bypass breakers during each refueling outage. The proposed change to the TS addresses items 10 and 13 of the September 26, 1984 SE and are in accordance with the guidelines provided in NRC Generic Letter 85-09 of May 23, 1985, and are, therefore, acceptable.

The staff has also reviewed the licensee's proposed Technical Specification changes of:

- (1) adding a reference to maintenance of sampling and analysis equipment as part of their Post-Accident Sampling Program; and
- (2) correction of two typographical errors.

The staff has determined that these changes are purely administrative in nature and acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). These amendments also involve changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, with respect to these items, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors:

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Date: June 17, 1986.

Distribution Copies:

Docket Files 50-266/301

NRC PDR

Local PDR

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