July 18, 1985

Docket Nos. 50-266
and 50-301

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Mr. C. W. Fay, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room 308
Milwaukee, Wisconsin 53201

Dear Mr. Fay:

The Commission has issued the enclosed Amendment Nos. 92 and 96 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2. The amendments consist of changes to the Technical Specifications in response to your application dated February 29, 1984 as modified June 7, 1984.

These amendments incorporate Limiting Conditions for Operation and surveillance requirements for accident monitoring instrumentation installed in response to NUREG-0737 "Clarification of TMI Action Plan Requirements." Your proposed Technical Specifications regarding reactor coolant gas vents have not been included in our review of your submittals. This item will be addressed in separate correspondence.

Also, as discussed with members of your staff, your administrative program for maintenance of your post-accident sampling capability should reference that provisions for maintenance of sampling and analysis equipment are included in the program as per the guidance of Generic Letter 83-37. While it is acceptable to the staff that your maintenance of the post-accident sampling and analysis equipment be part of your normal plant-wide maintenance program, as stated in your February 29, 1984 submittal, the staff feels that this should be identified in your Technical Specifications as per our guidance. Therefore, you are requested to modify your Technical Specification 15.6.8.3A accordingly with your next license amendment application for Point Beach Nuclear Plant Units 1 and 2.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next monthly Federal Register notice.

Sincerely,

/S/

Edward J. Butcher, Acting Chief
Operating Reactors Branch #3
Division of Licensing

Enclosures:
1. Amendment No. 92 to DPR-24
2. Amendment No. 96 to DPR-27
3. Safety Evaluation

cc w/enclosures:
See next page

ORB#3:DL	ORB#3:DL	ORB#3:DL	OELD	AD:OR:DL
PMKreutzer	TColburn	EJButcher		GClainas
7/9/85	7/9/85	7/11/85	7/15/85	7/18/85

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 92
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated February 29, 1984 as modified June 7, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 92 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Edward J. Butcher, Acting Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 18, 1985

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 96
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated February 29, 1984 as modified June 7, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 96 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Edward J. Butcher, Acting Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 18, 1985

ATTACHMENT TO LICENSE AMENDMENTS NO. 92 AND 96
TO FACILITY OPERATING LICENSE NO. DPR-24 AND DPR-27
DOCKET NOS. 50-266 AND 50-301

Revise Appendix A as follows:

Remove Pages	Insert Pages
15.3.5-1	15.3.5-1
	Table 15.3.5-5 (continued)
Table 15.4.1-1 (continued)	Table 15.4.1-1 (2 of 4)
Table 15.4.1-1 (4 of 4)	Table 15.4.1-1 (4 of 4)
-	15.6.8-3
-	15.6.9-11

15.3.5 INSTRUMENTATION SYSTEM

Operational Safety Instrumentation

Applicability

Applies to plant instrumentation systems.

Objectives

To provide for automatic initiation of the Engineered Safety Features in the event that principal process variable limits are exceeded, and to delineate the conditions of the plant instrumentation and safety circuits necessary to ensure reactor safety.

Specification:

A. The Engineered Safety Features initiation instrumentation setting limits shall be as stated in Table 15.3.5-1.

B. For on-line testing or in the event of a sub-system instrumentation channel failure, plant operation at rated power shall be permitted to continue in accordance with Tables 15.3.5-2 through 15.3.5-4.

C. In the event the number of channels of a particular sub-system in service falls below the limits given in the column entitled Minimum Operable Channels, or Minimum Degree of Redundancy cannot be achieved, operation shall be limited according to the requirement shown in Tables 15.3.5-2 through 15.3.5-4, Operator Action when minimum operable channels unavailable.

D. The accident monitoring instrumentation channels in Table 15.3.5-5 shall be operable. In the event the number of channels in a particular sub-system falls below the minimum number of operable channels given in Column 2, operation and subsequent operator action shall be in accordance with Column 3. This specification is not applicable in the cold or refueling shutdown conditions.

Basis:

Instrumentation has been provided to sense accident conditions and to initiate operation of the Engineered Safety Features (1).

Unit 1 - Amendment No. 55, 92
Unit 2 - Amendment No. 60, 96

TABLE 15.3.5-5 (Continued)

NO.	FUNCTIONAL UNIT	NO. OF CHANNELS	MINIMUM OPERABLE CHANNELS	OPERATOR ACTION IF CONDITIONS OF COLUMN 2 CANNOT BE MET
7.	Containment High Range Radiation Monitor	3	2	If operability cannot be restored within seven days after failure, prepare a special report to be submitted within thirty days in accordance with 15.6.9.3.G.
8.	Containment High Range Pressure Monitor	2	1	If operability cannot be restored within 48 hours, be in hot shutdown within twelve hours.
9.	a. Containment Water Level Keyway	2	1	Operation may continue up to thirty days. If operability cannot be restored, be in hot shutdown within the next twelve hours.
	b. Containment Water Level Sump B Continuous Indication	2	1	If the operability cannot be restored within 48 hours, be in hot shutdown within twelve hours.
10.	Containment Hydrogen Monitors	4	1	If operability cannot be restored within 72 hours, be in hot shutdown within the next six hours.
11.	Reactor Vessel Fluid Level System	4	1	If operability cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
12.	In-Core Thermocouples	4/core quadrant	2/core quadrant	If operability of at least two thermocouples per core quadrant cannot be restored within 48 hours, be in hot shutdown within the next twelve hours.
13.	Main Steam Line Radiation Monitors (SA-11)	1/steam line	1/steam line	If operability cannot be restored within seven days, prepare a special report to be submitted within thirty days in accordance with 15.6.9.3.H.

Unit 1 - Amendment No. 92
Unit 2 - Amendment No. 96

TABLE 15.4.1-1 (CONTINUED)
(Page 2 of 4)

	Channel Description	Check	Calibrate	Test	Remarks
10.	Rod Position Bank Counters	S (1)**	N.A.	N.A.	1) With analog rod position
11.	Steam Generator Level	S **	R	M (1)**	1) Includes test of logic for reactor trip on low-low level and automatic actuation logic for auxiliary feedwater pumps
12.	Steam Generator Flow Mismatch	S **	R	M **	
13.	Charging Flow	N.A.	R	N.A.	
14.	Residual Heat Removal Pump Flow	N.A.	R	N.A.	
15.	Boric Acid Tank Level	D	R	N.A.	
16.	Refueling Water Storage Tank Level	N.A.	R	N.A.	
17.	Volume Control Tank Level	N.A.	R	N.A.	
18.	Reactor Containment Pressure	D	R	B/W (1)**	1) Isolation valve signal
19.	Radiation Monitoring System	D	R	M	
20.	Boric Acid Control	N.A.	R	N.A.	
21.	Containment Water Level	M	R	N.A.	
22.	Turbine Overspeed Trip*	N.A.	R	M (1)**	1) Block trip
23.	Accumulator Level and Pressure	S	R	N.A.	

* Overspeed Trip Mechanism, and Independent Turbine Speed Detection and Valve Trip System.

** Not required during periods of refueling shutdown, but must be performed prior to starting up if it has not been performed during the previous surveillance period.

Unit 1 - Amendment No. 13, 60, 92
Unit 2 - Amendment No. 11, 86, 96

TABLE 15.4.1-1 (Page 4 of 4)

No.	Channel Description	Check	Calibrate	Test	Remarks
40.	Containment High Range Radiation	S **	R	M **	Calibration to be verification of response to a source.
41.	Containment Hydrogen Monitor	D	R/Q	N.A.	Gas Calibration - Q, Electronic Calibration - R Sample gas for calibration at 2% and 6% hydrogen.
42.	Reactor Vessel Fluid Level System	M	R	N.A.	
43.	In-Core Thermocouple	M	R	N.A.	Calibration to be verification of response to a source.

S - Each Shift
D - Daily
W - Weekly
B/W - Biweekly
Q - Quarterly
M - Monthly
P - Prior to each startup if not done previous week.
R - Each Refueling interval (But not to exceed 18 months).
N.A. - Not applicable.

**Not required during periods of refueling shutdown, but must be performed prior to starting up if it has not been performed during the previous surveillance period.

***Not required during periods of refueling shutdown if steam generator vessel temperature is greater than 70°F.

****When used for the overpressure mitigating system each PORV shall be demonstrated operable by:

a. Performance of a channel functional test on the PORV actuation channel, but excluding valve operation, within 31 days prior to entering a condition in which the PORV is required operable and at least once per 31 days thereafter when the PORV is required operable.

b. Testing valve operation in accordance with the inservice test requirements of the ASME Boiler and Pressure Vessel Code, Section XI.

Unit 1 - Amendment No. 38,47,55,76,92
Unit 2 - Amendment No. 30,33,60,80,96

15.6.8 PLANT OPERATING PROCEDURES (Continued)

15.6.8.4 The following programs shall be established, implemented, and maintained.

A. Post-Accident Sampling*

A program**which will ensure the capability to obtain and analyze reactor coolant, containment atmosphere, and in-plant gaseous effluent samples under accident conditions. The program shall include the following:

(i) Training of personnel; and

(ii) Procedures of sampling and analysis.

*Post-Accident Coolant Sampling and Post-Accident Containment Atmospheric Sampling Systems.

**It is acceptable if the licensee maintains details of the program in plant operation manuals.

G. Failure of Containment High-Range Radiation Monitor

A minimum of two in-containment radiation-level monitors with a maximum range of 10^8 rad/hr (10^7/hr for photos only) should be operable at all times except for cold shutdown and refueling outages. This is specified in Table 15.3.5-5, item 7. If the minimum number of operable channels are not restored to operable condition within seven days after failure, a special report shall be submitted to the NRC within thirty days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to operable status.

H. Failure of Main Steam Line Radiation Monitors

If a main steam line radiation monitor (SA-11) fails and cannot be restored to operability in seven days, prepare a special report outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the channel to operable status within thirty days of the event.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 92 AND 96 TO

FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-266 AND 50-301

Introduction and Background

In November 1980, the staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements," which included all TMI Action Plan items approved by the Commission for implementation at nuclear power reactors. NUREG-0737 identifies those items for which Technical Specifications were scheduled for implementation after December 31, 1981. The staff provided guidance on the scope of Technical Specifications for all of these items in Generic Letter 83-37 which was issued to all Pressurized Water Reactor (PWR) licensees on November 1, 1983. In this Generic Letter, the staff requested licensees to:

1. review their facilities' Technical Specifications to determine if they were consistent with the guidance provided in the Generic Letter, and

2. submit an application for a license amendment where deviations or absence of Technical Specifications were found.

By letter dated February 29, 1984 as modified June 7, 1984, Wisconsin Electric Power Company (the licensee) responded to Generic Letter 83-37 by submitting Technical Specification change requests for Point Beach Units 1 and 2. This evaluation covers the following TMI Action Plan items:

1. Post-Accident Sampling (II.B.3)
2. Containment High-Range Radiation Monitor (II.F.1.3)
3. Sampling and Analysis of Plant Effluents (II.F.1.2)
4. Containment Pressure Monitor (II.F.1.4)
5. Containment Water Level Monitor (II.F.1.5)
6. Containment Hydrogen Monitor (II.F.1.6)
7. Instrumentation for Detection of Inadequate Core Cooling (II.F.2)

EVALUATION

1. Post-Accident Sampling (II.B.3)

The guidance provided by Generic Letter 83-37 requested that an administrative program should be established, implemented and maintained to ensure that the licensee has the capability to obtain and analyze reactor coolant and containment atmosphere samples under accident conditions. The Post-Accident Sampling System is not required to be operable at all times. Administrative procedures are to be established for returning inoperable instruments to operable status as soon as practicable.

The licensee has provided a proposed revision to the TS which is consistent with the guidelines provided in our Generic Letter 83-37 with the exception that the licensee's proposed Technical Specification changes do not indicate that their Post-Accident Sampling program includes provisions for maintenance of sampling and analysis equipment as requested in our guidance. The staff has verified with the licensee via telephone conference call that such provisions do exist in their program. The staff also has requested that the licensee propose language to their program description contained in their Technical Specifications to reflect that these program provisions exist when their next request for Technical Specification changes is forwarded to the NRC. We conclude that the licensee otherwise has an acceptale TS for the Post-Accident Sampling System.

2. Containment High-Range Radiation Monitor (II.F.1.3)

The licensee has installed two in-containment monitors in each Point Beach Unit that is consistent with the guidance of TMI Action Plan Item II.F.1.3. Generic Letter 83-37 provided guidance for limiting conditions of operation and surveillance requirements for these monitors. The licensee proposed TSs that are consistent with the guidance provided in our Generic Letter 83-37. We conclude that the proposed TSs for Item II.F.1.3 are acceptable.

3. Sampling and Analysis of Plant Effluents (II.F.1.2)

The guidance provided by Generic Letter 83-37 requested that an administrative program should be established, implemented, and maintained to ensure the capability to collect and analyze or measure representative samples of radioactive iodines and particulates in plant gaseous effluents during and following an accident. The licensee has proposed TSs that are consistent with our guidance. We conclude that the TSs for sampling and analysis of plant effluents are acceptable.

4. Containment Pressure Monitor (II.F.1.4)

Each Point Beach Unit has been provided with two supplementary channels for monitoring containment pressure following an accident. The licensee has proposed TSs that are consistent with the guidelines contained in Generic Letter 83-37. We conclude that the proposed TSs for containment pressure monitor are acceptable.

5. Containment Water Level Monitor (II.F.1.5)

Narrow range and wide range containment water level monitors provide the capability required by TMI Action Plan Item II.F.1.5. The TSs for both units contain limiting conditions of operation and surveillance requirements that are consistent with the guidance contained in Generic Letter 83-37. We conclude that the proposed TSs for containment water level monitors are acceptable.

6. Containment Hydrogen Monitor (II.F.1.6)

The licensee installed containment hydrogen monitors that provide the capability required by TMI Action Plan Item II.F.1.6. The proposed Point Beach Units 1 and 2 TSs contain appropriate limiting conditions of operation and surveillance for these monitors. We conclude that the proposed TSs are acceptable as they are consistent with the guidance contained in Generic Letter 83-37.

7. Instrumentation for Detection of Inadequate Core Cooling (II.F.2)

Generic Letter 83-37 provided the guidance on TSs for the subcooling margin monitors, a reactor coolant inventory tracking system and core exit thermocouples. The licensee indicated that all hardware modifications are completed. We have reviewed the proposed TSs for the reactor coolant inventory tracking system and core exit thermocouples and conclude that the proposed TSs are acceptable as they meet the intent of our guidance contained in Generic Letter 83-37.

ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: July 18, 1985

Principal Contributor:
C. Patel
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