

March 7, 1985

Docket Nos. 50-266
and 50-301

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Mr. C. W. Fay, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room 308
Milwaukee, Wisconsin 53201

Dear Mr. Fay:

The Commission has issued the enclosed Amendment Nos. **88** and **93** to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2. The amendments consist of changes to the Technical Specifications in response to your application dated June 8, 1984.

These amendments revise Technical Specification 15.3.10 to define the "fully withdrawn" condition of a control rod as equal to or greater than 225 steps withdrawn. Your request also proposed a revised figure 15.3.10-1, "Control Rod Insertion Limits," which deleted reference to the number of steps withdrawn and instead graphed "percent of control bank withdrawn" to "percent of full power." This change has already been incorporated into Unit 2 Technical Specifications by issuance of amendment No. 90 to Facility Operating License DPR-27. Therefore this portion of your change request shall be applicable to Point Beach Unit 1 only.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next monthly Federal Register notice.

Sincerely,

Original signed by:

Timothy G. Colburn, Project Manager
Operating Reactors Branch #3
Division of Licensing

Enclosures:

1. Amendment No. **88** to DPR-24
2. Amendment No. **93** to DPR-27
3. Safety Evaluation

cc w/enclosures:
See next page

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*signed
CMT 3/7/85*

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ORB#3:DL
PMKreutzer
2/29/84

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ORB#3:DL
TColburn:dd
2/29/84

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JRMiller
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3/1/84

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AD OR:DL
GCLatras
3/6/84



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY
DOCKET NO. 50-266
POINT BEACH NUCLEAR PLANT, UNIT NO. 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 88
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated June 8, 1984 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

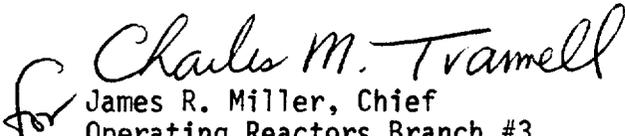
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 88 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


for James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 7, 1985



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 93
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated June 8, 1984 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 93 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 7, 1985



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 88 TO FACILITY OPERATING LICENSE NO. DPR-24

AMENDMENT NO. 93 TO FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NOS. 50-266 AND 50-301

Revise Appendix A as follows:

Remove Page

(Units 1 and 2) 15.3.10-1
(Unit 1 only) Figure 15.3.10-1

Insert Page

15.3.10-1
Figure 15.3.10-1

15.3.10 CONTROL ROD AND POWER DISTRIBUTION LIMITS

Applicability

Applies to the operation of the control rods and to core power distribution limits.

Objective

To insure (1) core subcriticality after a reactor trip, (2) a limit on potential reactivity insertions from a hypothetical rod cluster control assembly (RCCA) ejection, and (3) an acceptable core power distribution during power operation.

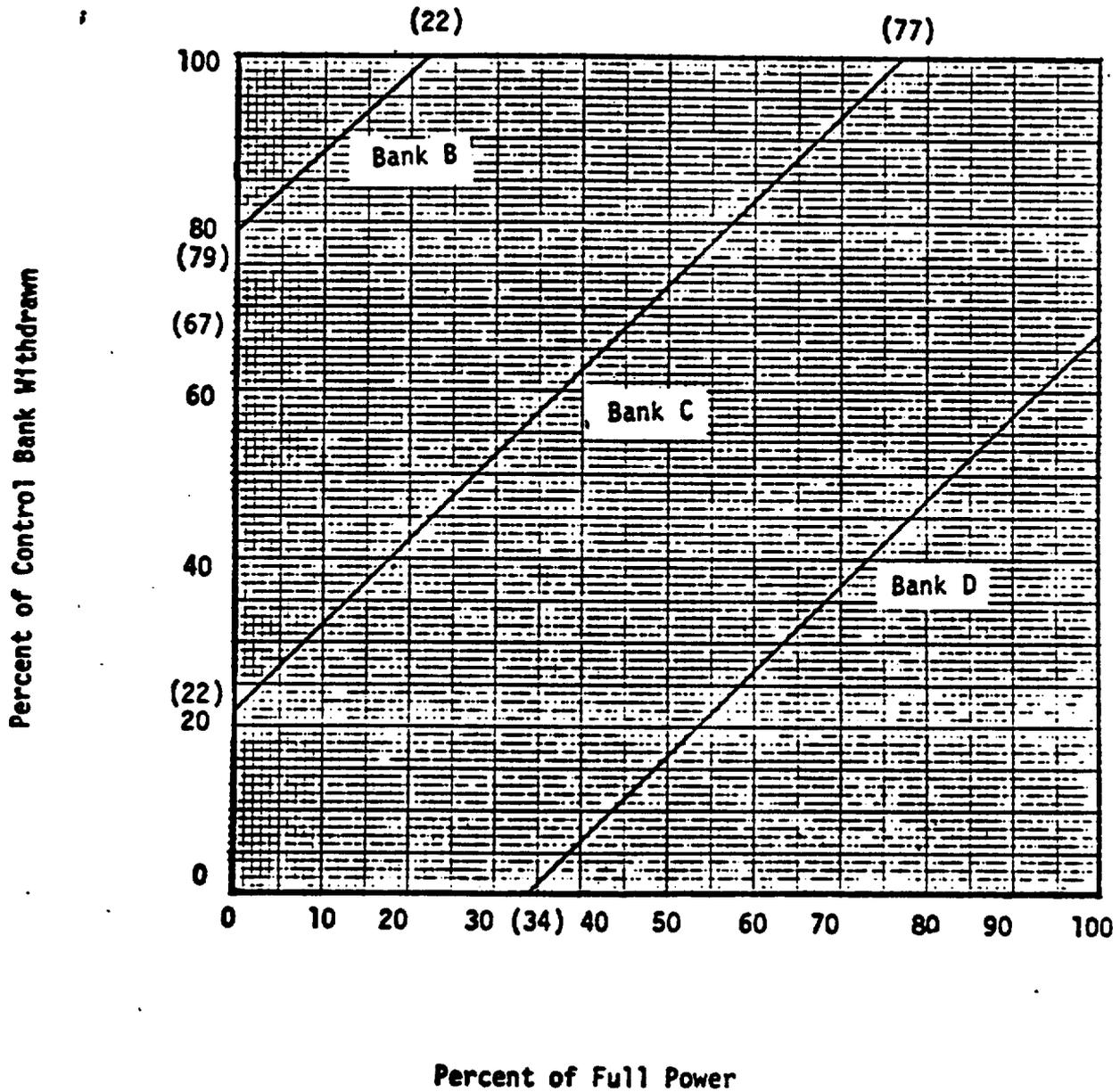
Specification

A. Bank Insertion Limits

1. When the reactor is critical, except for physics tests and control rod exercises, the shutdown banks shall be fully withdrawn. ⁽¹⁾
2. When the reactor is critical, the control banks shall be inserted no further than the limits shown by the lines on Figure 15.3.10-1. Exceptions to the insertion limit are permitted for physics tests and control rod exercises.
3. The shutdown margin shall exceed the applicable value as shown in Figure 15.3.10-2 under all steady-state operating conditions from 350°F to full power. An exception to the stuck RCCA component of the shutdown margin requirement is permitted for physics tests.
4. Except for physics tests a shutdown margin of at least 1% $\Delta k/k$ shall be maintained when the reactor coolant temperature is less than 350°F.
5. When the reactor is in the hot shutdown condition or during any approach to criticality, except for physics tests, the critical rod position shall not be lower than the insertion limit for zero power. That is, if the control rods were withdrawn in normal sequence with no other reactivity change, the reactor would not be critical until the control banks were above the insertion limit.

(1) Fully withdrawn is defined as a bank demand position equal to or greater than 225 steps. This definition is applicable to shutdown and control banks.

FIGURE 15.3.10-1
CONTROL BANK INSERTION LIMITS
POINT BEACH UNITS 1 AND 2



Unit 1 - Amendment No. 25, 49, 86, 88



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 88 AND 93 TO

FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-266 AND 50-301

Introduction

In a letter dated June 8, 1984, Wisconsin Electric Power Company proposed changes to Technical Specification 15.3.10, Control Rod and Power Distribution Limits, for Point Beach Units 1 and 2. The proposed changes revise the definition of "fully withdrawn" for the control rods as equal to or greater than 225 steps and the corresponding Figure 15.3.10-1. In a letter dated August 22, 1984, the licensee provided more detailed results of an analysis supporting the proposed changes.

EVALUATION

To minimize localized control rod cluster assembly (RCCA) wear at the top of the controls, the proposed Technical Specification changes will allow operation with the RCCAs inserted three steps into the reactor from their normal withdrawn position of 228 steps. At 225 steps withdrawn, the RCCSSs are only 0.3 inches into the active fuel region. Because of the low rod worth in the top region of core, the resultant power distribution perturbations are calculated to be less than 1%, and can be accommodated with available margin. Similarly, the effect on shutdown margin is minimal ($0.02\%_{\Delta\rho}$), and can be accommodated by available excess shutdown margin ($>0.75\%_{\Delta\rho}$).

The impact on other key safety parameters was found to be negligible in the licensee's analysis. Because the proposed change will insert the RCCAs so little into the active fuel region, we would expect essentially negligible effects of the proposed change as reported in the licensee's evaluation. We therefore find the proposed change acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to common defense and security or to the health and safety of the public.

Date: March 7, 1985

Principal Contributor:

M. Dunenfeld
T. Colburn