

DCS 715-016

APR 6 1984

Docket Nos. 50-266
50-301

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Mr. C. W. Fay
Vice President - Nuclear Power
Wisconsin Electric Power Company
231 West Michigan Street
Milwaukee, Wisconsin 53201

Dear Mr. Fay:

The Commission has issued the enclosed Amendment No. 83 to Facility Operating License No. DPR-24 and Amendment No. 87 to Facility Operating License No. DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated December 23, 1983.

These amendments revise the surveillance interval for evaluating the integrity of post-accident recovery systems outside containment from "yearly" to "each refueling cycle".

A copy of the Safety Evaluation is enclosed. The notice of issuance will be included in the Commission's next monthly Federal Register notice.

Sincerely,

Original signed by

Timothy G. Colburn, Project Manager
Operating Reactors Branch #3
Division of Licensing

Enclosures:

1. Amendment No. 83 to DPR-24
2. Amendment No. 87 to DPR-27
3. Safety Evaluation

cc: w/enclosures
See next page

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PKreutzer
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JRM/ter
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RBachmann
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GCL/ainas
4/6/84

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Wisconsin Electric Power Company

cc:

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Mr. Gordon Blaha
Town Chairman
Town of Two Creeks
Route 3
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Ms. Kathleen M. Falk
General Counsel
Wisconsin's Environmental Decade
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U. S. Environmental Protection Agency
Federal Activities Branch
Region V Office
ATTN: Regional Radiation
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Chairman
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Regional Administrator
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Office of Executive Director for Operations
799 Roosevelt Road
Glen Ellyn, Illinois 60137



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 83
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated December 23, 1983 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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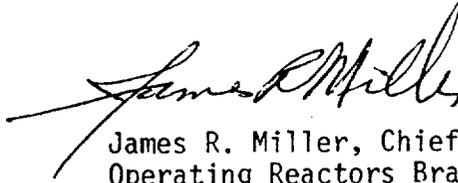
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 83, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 6, 1984



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 87
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated December 23, 1983 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 87, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 6, 1984

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 83 TO FACILITY OPERATING LICENSE NO. DPR-24

AMENDMENT NO. 87 TO FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NOS. 50-266 AND 50-301

Revise Appendix A as follows:

Remove Pages

Table 15.4.1-2
page 2 of 3

Insert Pages

Table 15.4.1-2
page 2 of 3

TABLE 15.4.1-2 (Continued)

	<u>Test</u>	<u>Frequency</u>
9. Control Rods	Rod drop times of all full length rods (3)	Each refueling or after maintenance that could affect proper functioning (4)
10. Control Rod	Partial movement of all rods	Every 2 weeks (6)
11. Pressurizer Safety Valves	Set point	Each refueling shutdown
12. Main Steam Safety Valves	Set point	Each refueling shutdown
13. Containment Isolation Trip	Functioning	Each refueling shutdown
14. Refueling System Interlocks	Functioning	Each refueling shutdown
15. Service Water System	Functioning	Each refueling shutdown
16. Primary System Leakage	Evaluate	Monthly (6)
17. Diesel Fuel Supply	Fuel inventory	Daily
18. Turbine Stop and Governor Valves	Functioning	Monthly (6)(10)
19. Low Pressure Turbine Rotor Inspection (5)	Visual and magnetic particle or liquid penetrant	Every five years
20. Boric Acid System	Storage Tank temperature	Daily
21. Boric Acid System	Visual observation of piping temperatures (all $\geq 145^{\circ}\text{F}$)	Daily
22. Boric Acid Piping Heat Tracing	Electrical circuit operability	Monthly
23. PORV Block Valves	Complete Valve Cycle	Quarterly (6)
24. Integrity of Post Accident Recovery Systems Outside Containment	Evaluate	Each refueling cycle
25. Containment Purge Supply and Exhaust Isolation Valves	Verify valves are locked closed	Monthly (9)

(1) Required only during periods of power operation.

(2) \bar{E} determination will be started when the gross activity analysis of a filtered sample indicates ≥ 10 $\mu\text{c}/\text{cc}$ and will be redetermined if the primary coolant gross radioactivity of a filtered sample increases by more than 10 $\mu\text{c}/\text{cc}$.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 83 TO FACILITY OPERATING LICENSE NO. DPR-24
AND AMENDMENT NO. 87 TO FACILITY OPERATING LICENSE NO. DPR-27
WISCONSIN ELECTRIC POWER COMPANY
POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-266 AND 50-301

Introduction:

By letter from D. G. Eisenhut dated July 2, 1980, the NRC staff transmitted to all licensees of pressurized water reactors a set of model Technical Specifications (TS). These TS were designed to provide guidance in the scope and type of required specifications in the areas of equipment and administrative requirements which the staff felt were necessary to insure that facility operation of these reactors was maintained within acceptable limits following implementation of the TMI-1 Category "A" Lessons Learned Items. Licensees were requested to amend the TS of their facilities' operating licenses using the guidance provided by the staff. One of these TS related to establishing a program to insure systems integrity of selected systems outside containment by leakage testing "at a frequency not to exceed refueling cycle intervals".

Background:

By letter dated February 4, 1981, Wisconsin Electric Power Company (licensee) requested changes to the Point Beach Unit 1 and 2 Technical Specifications in response to Mr. Eisenhut's letter. Included with the requested changes was a proposed specification for evaluating the integrity of post-accident recovery systems outside containment on a yearly basis. While this wording did not exactly match that of the model TS provided as guidance in Mr. Eisenhut's letter, it was found acceptable to the NRC staff because Point Beach Units 1 and 2 were operating on an annual refueling cycle (small deviations in surveillance intervals, $\pm 25\%$, were allowed by the TS to allow for minor schedule changes). These proposed TS were approved by issuance of amendments on September 30, 1981. This particular requirement is contained in Table 15.4.1-2 of the TS.

Subsequent to the issuance of these amendments, Point Beach Unit 1 shut down for an extended refueling outage (approximately 6 months) to conduct major steam generator repairs. Because of the length of this outage and corresponding length of the operating cycle for Unit 2, neither Unit 1 nor 2 would be able to meet the yearly requirement for conducting an evaluation of the integrity of post-accident recovery systems outside containment even after applying the $\pm 25\%$ schedule deviation allowance currently contained in their TS. Therefore, the licensee has requested that the surveillance frequency

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for this requirement be changed from "yearly" to "each refueling cycle". This would allow conformance to the requirement even during rare occasions when the operating cycle or outage duration of a unit must be extended beyond the scheduled interval by more than 25%.

Discussion and Evaluation

The frequency provided as guidance in Mr. Eisenhut's July 2, 1980 letter for this surveillance requirement was "a frequency not to exceed refueling cycle intervals". The basis for selecting this surveillance frequency was that the requirement should be performed as often as practical in order to insure that leakage from systems outside containment which could contain radioactive fluid following an accident would be reduced to as low a level as practical. However, it was anticipated that a unit shutdown would probably be required in order to complete the surveillance requirement. Therefore, a refueling interval was accepted as the surveillance frequency. It was known at the time that this surveillance frequency was accepted that many plants' refueling cycles exceeded 12 month intervals (i.e. 18 or 24 month intervals).

Because the requested changes to the surveillance frequency more closely match the wording contained in the staff's July 2, 1980 guidance letter, and because the real time interval between consecutive evaluations (approximately 18 months) is well within the range experienced for plants required to perform this requirement, the staff finds the licensee's proposed changes to be acceptable for Point Beach Units 1 and 2.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact, and pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: April 6, 1984

Principal Contributor:
T. G. Colburn