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Docket No. 50-266 DEC 30 1983

Mr. C. W. Fay
Vice President - Nuclear Power
Wisconsin Electric Power Company
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Dear Mr. Fay:

The Commission has issued the enclosed Amendment No. 81 to Facility Operating License No. DPR-24 for the Point Beach Nuclear Plant, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated July 5, 1983.

This amendment authorizes reactor operation of Point Beach Unit 1 at either 2250 or 2000 psia after return to power following steam generator replacement.

Specific changes to the Technical Specifications have been incorporated to allow for operation of Unit 1 at either 2000 or 2250 psia. These are (1) defining the overtemperature delta T reactor trip equation for each operating condition; (2) defining a low pressure reactor trip for each operating condition to allow adequate operating margin; (3) defining an operational reactor pressure limit for each operating condition; and (4) defining system leak testing pressure for each operating condition.

Unit 1 was restricted to operation at 2000 psia by the Commission's January 3, 1980 Order Modifying Confirmatory Order of November 30, 1979. The technical basis for imposing this operating restriction from the 2250 psia design operating pressure was contained in the Safety Evaluation accompanying that Order which is incorporated herein by reference. Also included in that Safety Evaluation was the technical basis for allowing operation of Unit 2 at either 2000 or 2250 psia. In that evaluation, we concluded that operation of Unit No. 1 at 2000 psia should be required in order to reduce the differential pressure stress on the steam generator tubes. Whereas similar operation of Unit No. 2 would have had the same effect, there was no safety reason for requiring 2000 psia operation for Unit No. 2 at that time because the condition of the steam generator tubes was and is acceptable for operation at either 2000 psia or 2250 psia. Therefore, the amendment authorized operation of Unit No. 2 at either reactor coolant pressure.

The conditions which prompted restricting Unit 1 operation to 2000 psia, severe degradation of the steam generator tubes, will no longer exist following the replacement of the Unit 1 steam generator lower assemblies. Therefore, there will no longer be any reason to restrict operation of Unit 1 at 2000 psia. The

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Mr. C. W. Fay

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replacement lower assemblies have been designed for operation at either 2000 or 2250 psia reactor coolant system pressure. The technical basis for allowing operation of Unit 1 at either pressure is contained in our January 3, 1980 Safety Evaluation which at that time pertained only to Unit 2 but now is equally applicable to Unit 1.

As discussed with members of your staff, a typographical error was contained in your evaluation. The correct value for the low pressurizer pressure trip at 2250 psia operation is 1865 psig not 1855 psig as contained in your application.

We have evaluated the potential for environmental impact of plant operation in accordance with the enclosed amendment. We have determined that the amendment does not authorize a change in effluent types or total amounts beyond that previously reviewed nor an increase in power level, and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

We have also concluded based on the considerations contained in our January 3, 1980 Safety Evaluation and those discussed above that there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

The Notice of Issuance will be contained in the Commission's next monthly Federal Register notice.

Sincerely,

Original Signed by J. R. Miller

James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Enclosure:
Amendment No. 81 to DPR-24

cc w/enclosure:
See next page

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PMK/peutzer
12/16/83

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ORB#3:DL
TColburn/pn
12/16/83

JRM
ORB#3:DL
JRMiller
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 81
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated July 5, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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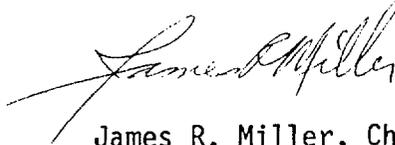
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 81, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 20 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 30, 1983

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 81 TO FACILITY OPERATING LICENSE NO. DPR-24

DOCKET NO. 50-266

Revise Appendix A as follows:

Remove Pages

15.2.3-2
15.3.1-19
15.4.3-1

Insert Pages

15.2.3-2
15.3.1-19
15.4.3-1

- * (3) Low pressurizer pressure - ≥ 1865 psig for operation at 2250 psia primary system pressure
 ≥ 1790 psig for operation at 2000 psia primary system pressure

(4) Overtemperature ΔT

$$\leq \Delta T_o \left(K_1 - \frac{K_2(T-T')}{1+\tau_1 S} + K_3(P-P') - f(\Delta I) \right)$$

where

ΔT_o = indicated ΔT at rated power, °F

T = average temperature, °F

T' = 574.2°F

P = pressurizer pressure, psig

P' = 2235 psig

*K₁ ≤ 1.117 for operation at 2250 psia primary system pressure
 ≤ 1.30 for operation at 2000 psia primary system pressure

K₂ = 0.0150

K₃ = 0.000791

τ_1 = 25 sec

τ_2 = 3 sec

and $f(\Delta I)$ is an even function of the indicated difference between top and bottom detectors of the power-range nuclear ion chambers; with gains to be selected based on measured instrument response during plant startup tests, where q_t and q_b are the percent power in the top and bottom halves of the core respectively, and $q_t + q_b$ is total core power in percent of rated power, such that:

- (a) for $q_t - q_b$ with -17, +9 percent, $f(\Delta I) = 0$.
- (b) for each percent that the magnitude of $q_t - q_b$ exceeds +9 percent the ΔT trip set point shall be automatically reduced by an equivalent of two percent of rated power.

*Appropriate safety analyses shall be performed prior to shifting operation from one primary system pressure to the other.

G. OPERATIONAL LIMITATIONS

The following DNB related parameters shall be maintained within the limits shown during Rated Power operation:

1. T_{AVG} shall be maintained at or below 578°F.
- *2. Reactor coolant system pressure shall be maintained:
 \geq 2205 psig during operation at 2250 psia, or
 \geq 1955 psig during operation at 2000 psia.
3. Reactor Coolant System Total Flow Rate \geq 178,000 gpm.

Basis:

Although the operational limitations above require reactor coolant system total flow be maintained above a minimum rate, no direct means of measuring absolute flow during operation exists. However, during initial startup reactor coolant flow was measured and correlated to core ΔT . Therefore monitoring of ΔT may be used to verify the above minimum flow requirement is met. If a change in steady state full power ΔT greater than 3°F is observed, the actual flow measurements will be taken.

*See footnote, page 15.2.3-2.

15.4.3 PRIMARY SYSTEM TESTING FOLLOWING OPENING

Applicability

Applies to test requirements for primary system integrity.

Objective

To specify tests for primary system integrity after the system is closed following normal opening, modification or repair.

Specification

- a) When the primary system is closed after it has been opened, the system will be leak tested at:
 - 1) Not less than 2335 psig for operation at 2250 psia primary system pressure, or
 - 2) Not less than 2085 psig for operation at 2000 psia primary system pressure.
- b) When primary system modifications or repairs have been made which involved new strength welds on components greater than 2 in. diameter, the new welds will receive both a surface and 100% volumetric non-destructive examination.
- c) When primary system modifications or repairs have been made which involve new strength welds on components 2 in. diameter or smaller, the new welds will receive a surface examination.

Basis

For normal opening the integrity of the system, in terms of strength, is unchanged. If the system does not leak at 2335 psig (operating pressure + 100 psi: ± 100 psi is normal system pressure fluctuation), it should be leak tight during normal operation at 2250 psia. If the system does not leak at 2085 it should be leak tight during normal operation at 2000 psia.