

DCS MS-014

APR 1 1983

Docket Nos. 50-266  
and 50-301

Mr. C. W. Fay  
Vice President - Nuclear Power  
Wisconsin Electric Power Company  
231 West Michigan Street  
Milwaukee, Wisconsin 53201

Dear Mr. Fay:

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*See Correction  
letter of 4/15/83*

The Commission has issued the enclosed Amendment No. 70 and 75 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consists of changes to the Technical Specifications in response to your application dated October 21, 1982.

These amendments modify the requirements for periodic leakage rate testing of the containment airlock doors consistent with those specified in 10 CFR Part 50 Appendix J.

Minor modifications to the proposed Technical Specifications (TS) contained in your October 21, 1982 application have been made as discussed with and agreed to by members of your staff to more accurately reflect the requirements of Appendix J.

Reference 1 to the enclosed Safety Evaluation has been provided to you previously as Appendix A to the Technical Evaluation Report (TER) contained in our June 25, 1982 evaluation of your requested exemptions from the requirements of Appendix J.

We have also noted your January 4, 1983 letter notifying us of your intent to change the TS basis regarding leakage testing of the containment purge supply and exhaust valves. Although not part of this TS change request, we offer the following comments. We do not agree with your determination that these valves are more appropriately tested using type "B" tests; however, as stated in our June 25, 1982 letter, your present method of leakage testing these valves acceptably meets the requirements for Type "C" testing and therefore, no changes to the TS basis are necessary. Type "B" testing requirements would be appropriate for true barriers such as blank flanges but not for locked closed valves. Since it is likely that the Point Beach containment purge supply and exhaust valves will be operated during cold or refueling shutdown, we feel it is more appropriate to consider their testing as falling under Type "C" testing requirements. These comments have been discussed with members of you staff.

8304 120705 830401  
PDR ADDCK 05000266  
PDR

OFFICE	.....	.....	.....	.....	.....	.....	.....
SURNAME	.....	.....	.....	.....	.....	.....	.....
DATE	.....	.....	.....	.....	.....	.....	.....

Mr. C. W. Fay

- 2 -

We understand from your earlier letter dated August 13, 1982 that modifications are planned for the containment spray system which will allow leakage testing of the containment spray isolation valves in accordance with Appendix J requirements. Further, we understand that these modifications were scheduled to be completed during the Fall 1982 refueling outage for Unit 1 and will be completed during the Spring 1983 refueling outage for Unit 2. Please notify us if our understanding of this issue is incorrect.

Your August 13 letter also requested a delay in submitting TS changes for duration of Type "A" containment leakage testing until August 1983 or approximately six months prior to the next scheduled containment integrated leak rate test. Your basis for this delay was to allow completion of an EPRI funded Quadrex Corporation study which you assert will validate the technical acceptance criteria for short duration (less than 24 hours) leakage testing and which you believe will be superior to that contained in NRC approved Bechtel Topical Report BN-TOP-1. This delay is acceptable; however, irrespective of any delays occurring in the completion of the Quadrex Corporation study, changes to your TS should be submitted sufficiently in advance of your next scheduled Type "A" test to allow for NRC review.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Timothy G. Colburn, Project Manager  
Operating Reactors Branch #3  
Division of Licensing

Enclosures:

- 1. Amendment No. 70 to DPR-24
- 2. Amendment No. 75 to DPR-27
- 3. Safety Evaluation
- 4. Notice of Issuance

cc: w/enclosures  
See next page

F.R. NOTICE  
+ AMEND.

OFFICE	ORB#3:DL	ORB#3:DL	ORB#3:DL	AD:DL	DL		
SURNAME	PM reutzer	T Colburn/pn	RAClark	GCLainas	R. Bachman		
DATE	3/21/83	3/23/83	3/23/83	3/29/83	3/30/83		



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

DISTRIBUTION:  
Docket File  
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PMKreutzer

Docket No. 50-266/50-301

Docketing and Service Section  
Office of the Secretary of the Commission

SUBJECT: WISCONSIN ELECTRIC POWER COMPANY, Point Beach Nuclear Plant,  
Unit Nos. 1 and 2.

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (12 ) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- Notice of Availability of Applicant's Environmental Report.
- Notice of Proposed Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).
- Other: Amendment Nos. 70 and 75

Referenced documents have been provided PDR.

Division of Licensing  
Office of Nuclear Reactor Regulation

Enclosure:  
As Stated

OFFICE	→ ORB#3:DL					
SURNAME	→ PMKreutzer/pn					
DATE	→ 4/4/83					

Wisconsin Electric Power Company

cc:

Mr. Bruce Churchill, Esquire  
Shaw, Pittman, Potts and Trowbridge  
1800 M Street, N. W.  
Washington, D. C. 20036

USNRC Resident Inspectors Office  
6612 Nuclear Road  
Two Rivers, Wisconsin 54241

Mr. James J. Zach, Manager  
Nuclear Operations  
Wisconsin Electric Power Company  
Point Beach Nuclear Plant  
6610 Nuclear Road  
Two Rivers, Wisconsin 54241

Mr. Gordon Blaha  
Town Chairman  
Town of Two Creeks  
Route 3  
Two Rivers, Wisconsin 54241

Ms. Kathleen M. Falk  
General Counsel  
Wisconsin's Environmental Decade  
114 N. Carroll Street  
Madison, Wisconsin 53703

U. S. Environmental Protection Agency  
Federal Activities Branch  
Region V Office  
ATTN: Regional Radiation  
Representative  
230 S. Dearborn Street  
Chicago, Illinois 60604

Chairman  
Public Service Commission of Wisconsin  
Hills Farms State Office Building  
Madison, Wisconsin 53702

Regional Administrator  
Nuclear Regulatory Commission, Region III  
Office of Executive Director for Operations  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 70  
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated October 21, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

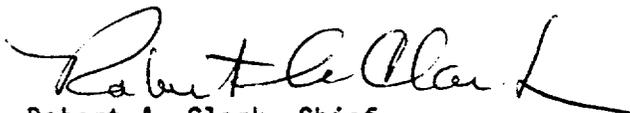
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 70, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief  
Operating Reactors Branch #3  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 1, 1983



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 75  
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated October 21, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I:
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 75, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief  
Operating Reactors Branch #3  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 1, 1983

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 70 TO FACILITY OPERATING LICENSE NO. DPR-24

AMENDMENT NO. 75 TO FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NOS. 50-266 AND 50-301

Revise Appendix A as follows:

Remove Pages

15.4.4-4  
15.4.4-5  
15.4-4-6

Insert Pages

15.4.4-4  
15.4.4-5  
15.4.4-6

acceptance criteria in Section 1.B above, a Type "A" test shall be performed at each plant shutdown for refueling, or approximately every 18 months, whichever occurs first. The accelerated test schedule shall continue until two consecutive Type "A" tests pass, after which time the retest schedule in I.C.1 may be resumed.

D. Report of Test Results

1. Each Type "A" leakage rate test will be the subject of a summary technical report, which will include summaries of Type "B" and "C" tests (Items II and III below) that were performed since the last Type "A" test.

II. Type "B" Tests

A Type "B" test measures leakage across individual and/or portions of pressure containing or leakage-limiting boundaries of primary reactor containment penetrations as defined in II.A.5.

A. Test

1. Type "B" tests shall be performed at intervals specified in II.C. below.
2. With the exception of the airlock door seal test, for the purposes of the 3 day test requirement, Type "B" tests shall be performed at a pressure of not less than  $P_a$ .
3. Testing of the airlock door seals, in lieu of the full pressure airlock test, may be used to fulfill the 3 day airlock testing requirement specified in II.C.1.d below. This airlock door seal test shall be performed with a pressure differential across the door seals of at least 10" of mercury. This pressure differential may be established via the use of a positive pressure or a vacuum. Airlock door seal testing shall not be substituted for the 6 month test of the entire air lock.
4. Acceptable methods of testing are halogen leak detection, pressure decay and fluid flow using air or nitrogen. Another method may be used if it can be shown to have equivalent sensitivity.
5. The local leakage shall be measured for each of the following components:
  - a. Containment penetrations that employ resilient seals, gaskets or sealant compounds, piping penetrations fitted with expansion bellows and electrical penetrations fitted with flexible metal seal assemblies.

- b. Airlock and equipment door seals, including operating mechanism and penetrations with resilient seals which are part of the containment boundary in the airlock structure.
- c. Fuel transfer tube flange seal.
- d. Other containment components which require leak repair in order to meet the acceptance criterion for any integrated leakage rate test.

B. Acceptance Criterion

- 1. The total leakage from items II.A.5 and III.A.3 shall not exceed  $0.6 L_a$ .
  - a. If at any time it is determined that  $0.6 L_a$  is exceeded, repairs shall be initiated immediately. After repair, a retest to confirm conformance to the acceptance criterion of II.B. is required.
  - b. If repairs are not completed and conformance to the acceptance criterion of II.B. is not demonstrated within 48 hours, the reactor shall be taken to cold shutdown conditions until repairs are effected and the local leakage meets this acceptance criterion.
- 2. The leakage from the airlock doors seal test, resulting from the 3 day testing requirement in II.C.1.d, shall be considered acceptable if the leakage sum from the worst door in each airlock, extrapolated to  $P_a$ , and added to the total of items II.A.5 and III.A.3, is less than  $0.6 L_a$ .
  - a. If the total identified in II.B.2, above, exceeds  $0.6 L_a$ , then the airlock containing the worst door shall be full pressure tested to determine the actual leakage performance.

C. Test Frequency

- 1. Individual penetrations shall be tested during each shutdown for major fuel reloading except as specified in a and b below. In no case shall the interval be greater than two years.
  - a. The containment equipment hatch flange seals and the fuel transfer tube flange seals shall be tested at each shutdown for major fuel reloading or after each time used, if that be sooner.

Unit 1  
Unit 2

15.4.4-5

Amendment No. 61, 70  
Amendment No. 66, 75

- b. The air locks shall be tested at 6-month intervals at test pressure not less than  $P_a$ .
- c. Personnel airlocks shall be tested at a pressure of no less than  $P_a$  following periods when containment integrity is defeated through the use of the airlock.
- d. Personnel airlocks opened during periods when containment integrity is established shall be tested within 3 days after being opened. Personnel airlocks opened more frequently than once every 3 days shall be tested at least once every 3 days during the period of frequent openings.

### III. Type "C" Tests

A Type "C" test measures the leakage across an individual valve or across a group of valves used to isolate an individual penetration through the primary reactor containment as defined in III.A.3.

#### A. Test

1. Type "C" tests shall be performed at intervals specified in III.D below and at a pressure of not less than  $P_a$ .
2. Acceptable methods of testing are by local pressurization and the methods described in II.A.4 above. The pressure shall be applied in the same direction as that when the valve would be required to perform its safety function, unless it can be determined that the results from the tests for a pressure applied in a different direction will provide equivalent or more conservative results. Each valve to be tested shall be closed by normal operation and without any preliminary exercising or adjustments.

Unit 1  
Unit 2

15.4.4-6

Amendment No. 61, 70  
Amendment No. 66, 75



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 70 AND 75 TO

FACILITY OPERATING LICENSE NOS. DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-266 AND 50-301

Introduction

By letter dated June 25, 1982 the NRC staff transmitted copies of an Exemption and Safety Evaluation regarding Wisconsin Electric Power Company's (licensee) requested exemption to 10 CFR Part 50 Appendix J. For denied exemptions, the staff requested that the licensee submit Technical Specification (TS) changes for Point Beach Units 1 and 2 within 45 days receipt of our evaluation to provide compliance with Appendix J leakage testing requirements.

By letter dated August 13, 1982 the licensee requested a delay for submission of proposed TS in order to complete development of a methodology for reduced pressure testing of containment airlock doors and to await results of an industry study on reduced duration (less than 24 hours) Type "A" containment integrated leak rate testing. By letter dated October 21, 1982 the licensee submitted proposed TS for containment airlock door testing.

Discussion and Evaluation

The licensee's proposed TS changes propose to test the air lock door, "O"-ring seal by drawing a vacuum between the seals and extrapolating the measured leakage to the design pressure. This test would be performed to satisfy the requirement for testing the air lock door seals three days after an air lock opening. In addition to this test, the air locks would be tested every six months at the design pressure of 60 psig.

The staff agrees that the licensee can extrapolate the leakage rate from the vacuum test result to accident pressure, if the following equation is used for the extrapolation:

$$\frac{m_a}{m_t} = \frac{(P_a + P_{at})^2 - (P_{at})^2}{(P_{at})^2 - (P_{at} - P_t)^2}$$

Where:

- $m_a$  = leakage rate at the accident pressure ( $P_a$ )
- $m_t$  = leakage rate at the vacuum pressure
- $P_a$  = pressure resulting from an accident
- $P_t$  = vacuum pressure of leakage test
- $P_{at}$  = atmospheric pressure (Reference (1)).

The licensee's proposed testing meets the testing requirements in 10 CFR Part 50 Appendix J for airlock door testing. We therefore consider the proposed change acceptable

#### Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendments.

#### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of an accident previously evaluated, do not create the possibility of an accident of a type different from any evaluated previously, and do not involve a significant reduction in a margin of safety, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: April 1, 1983

#### Reference

- (1) Conversion of Reduced Pressure Air Leakage Measurements to Equivalent Full Pressure Air Leakage: July 17, 1980, Dr. G. P. Wachtell, Franklin Research Center

Principal Contributors:

P. Hearn, CSB/DSI  
T. G. Colburn, ORB#3/DL

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-266 AND 50-301WISCONSIN ELECTRIC POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY  
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 70 and 75 to Facility Operating License Nos. DPR-24 and DPR-27 issued to Wisconsin Electric Power Company (the licensee), which revised Technical Specifications for operation of Point Beach Nuclear Plant, Unit Nos. 1 and 2 (the facilities) located in the Town of Two Creeks, Manitowoc County, Wisconsin. The amendments are effective as of the date of issuance.

The amendments modify the requirements for periodic leakage rate testing of the containment air lock doors.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

- 2 -

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated October 21, 1982, (2) Amendment Nos. 70 and 75 to License Nos. DPR-24 and DPR-27, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. 20555, and at the Joseph Mann Library, 1516 16th Street, Two Rivers, Wisconsin 54241. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 1st day of April, 1983.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief  
Operating Reactors Branch #3  
Division of Licensing