

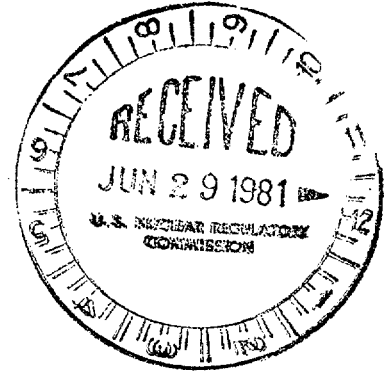


UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

June 24, 1981

Docket Nos. 50-266
and 50-301

Mr. Sol Burstein
Executive Vice President
Wisconsin Electric Power Company
231 West Michigan Street
Milwaukee, Wisconsin 53201



Dear Mr. Burstein:

The Commission has issued the enclosed Amendment No. 50 to Facility Operating License No. DPR-24 and Amendment No. 54 to Facility Operating License No. DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated September 19, 1980.

These amendments consist of a revised definition of operability and several new specifications addressing the limiting conditions for operation. They also address inoperability of safety-related systems due solely to loss of their normal emergency power supplies. The above changes were requested by our letter to you dated April 10, 1980. The acceptability of these changes is contained in that letter which constitutes our Safety Evaluation of this matter.

Also contained in these amendments is a specification addressing the momentary loss of normal or emergency power resulting in immediate corrective action as it relates to minimum operable channels or minimum degree of redundancy. You state that this specification serves to clarify the intent of Table 15.3.5-2 of the Technical Specifications. We have reviewed this specification and find it acceptable.

The remainder of changes in these amendments are administrative changes, the first, the inclusion of the Reactor Engineer as part of the Manager's Supervisory Staff and the second, a specification relating to the exercising of positive control over individuals entering or working in radiation areas. The first is a relatively minor change over that previously approved and is acceptable. The second change you assert is in response to suggestions received during an NRC inspection. We find that it conforms with NRC standard Technical Specifications and is therefore acceptable.

8107010 065

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

A copy of the Notice of Issuance is also enclosed.

Sincerely,

A handwritten signature in dark ink, appearing to read "Robert A. Clark", with a long horizontal flourish extending to the right.

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Enclosures:

1. Amendment No. 50 to DPR-24
2. Amendment No. 54 to DPR-27
3. Notice of Issuance

cc: w/enclosures
See next page

Wisconsin Electric Power Company

cc:

Mr. Bruce Churchill, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N. W.
Washington, D. C. 20036

Mr. William Guldemon
USNRC Resident Inspectors Office
6612 Nuclear Road
Two Rivers, Wisconsin 54241

Joseph Mann Library
1516 Sixteenth Street
Two Rivers, Wisconsin 54241

Mr. Glenn A. Reed, Manager
Nuclear Operations
Wisconsin Electric Power Company
Point Beach Nuclear Plant
6610 Nuclear Road
Two Rivers, Wisconsin 54241

Mr. Gordon Blaha
Town Chairman
Town of Two Creeks
Route 3
Two Rivers, Wisconsin 54241

Ms. Kathleen M. Falk
General Counsel
Wisconsin's Environmental Decade
302 E. Washington Avenue
Madison, Wisconsin 53703

Director, Criteria and Standards Division
Office of Radiation Programs (ANR-460)
U.S. Environmental Protection Agency
Washington, D. C. 20460

U. S. Environmental Protection Agency
Federal Activities Branch
Region V Office
ATTN: EIS COORDINATOR
230 S. Dearborn Street
Chicago, Illinois 60604

cc w/enclosure(s) and incoming
dtd: 9/19/80

Chairman
Public Service Commission of Wisconsin
Hills Farms State Office Building
Madison, Wisconsin 53702



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 50
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated September 19, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

8107010 069

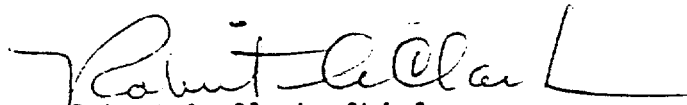
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 50, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 24, 1981



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY
DOCKET NO. 50-301
POINT BEACH NUCLEAR PLANT, UNIT NO. 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 56
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated September 19, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.


2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 56, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 24, 1981

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 50 TO FACILITY OPERATING LICENSE NO. DPR-24

AMENDMENT NO. 56 TO FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NOS. 50-266 AND 50-301

Revise Appendix A as follows:

Remove Pages

15-i
15.1-2

15.6.4/5-1
15.6.5-2
15.6.11-1

Insert Pages

15-i
15.1-2
15.3.0-1
15.3.0-2
15.3.0-3
15.3.0-4
15.3.0-5
15.6.4/5-1 (overleaf)
15.6.5-2
15.6.11-1

TABLE OF CONTENTS

<u>Section</u>	<u>Title</u>	<u>Page</u>
15	TECHNICAL SPECIFICATIONS AND BASES	
15.1	Definitions	15.1-1
15.2.0	Safety Limits and Limiting Safety System Settings	15.2.1-1
15.2.1	Safety Limit, Reactor Core	15.2.1-1
15.2.2	Safety Limit, Reactor Coolant System Pressure	15.2.2-1
15.2.3	Limiting Safety System Settings, Protective Instrumentation	15.2.3-1
15.3	Limiting Conditions for Operation	15.3-0
15.3.0	General Considerations	15.3.0-1
15.3.1	Reactor Coolant System	15.3.1-1
15.3.2	Chemical and Volume Control System	15.3.2-1
15.3.3	Emergency Core Cooling System, Auxiliary Cooling Systems, Air Recirculation Fan Coolers, and Containment Spray	15.3.3-1
15.3.4	Steam and Power Conversion System	15.3.4-1
15.3.5	Instrumentation System	15.3.5-1
15.3.6	Containment System	15.3.6-1
15.3.7	Auxiliary Electrical Systems	15.3.7-1
15.3.8	Refueling	15.3.8-1
15.3.9	Effluent Releases	15.3.9-1
15.3.10	Control Rod and Power Distribution Limits	15.3.10-1
15.3.11	Movable In-Core Instrumentation	15.3.11-1
15.3.12	Control Room Emergency Filtration	15.3.12-1
15.3.13	Shock Suppressors (Snubbers)	15.3.13-1
15.3.14	Fire Protection System	15.3.14-1
15.3.15	Overpressure Mitigating System	15.3.15-1
15.3.16	Reactor Coolant System Pressure Isolation Valves	15.3.16-1
15.4	Surveillance Requirements	15.4-1
15.4.1	Operational Safety Review	15.4.1-1
15.4.2	In-Service Inspection of Primary System Components	15.4.2-1
15.4.3	Primary System Testing Following Opening	15.4.3-1
15.4.4	Containment Tests	15.4.4-1
15.4.5	Emergency Core Cooling System and Containment Cooling System Tests	15.4.5-1
15.4.6	Emergency Power System Periodic Tests	15.4.6-1
15.4.7	Main Steam Stop Valves	15.4.7-1
15.4.8	Auxiliary Feedwater System	15.4.8-1
15.4.9	Reactivity Anomalies	15.4.9-1
15.4.10	Operational Environmental Monitoring	15.4.10-1
15.4.11	Control Room Emergency Filtration	15.4.11-1
15.4.12	Miscellaneous Radioactive Materials Sources	15.4.12-1
15.4.13	Shock Suppressors (Snubbers)	15.4.13-1
15.4.14	Surveillance of Auxiliary Building Crane	15.4.14-1
15.4.15	Fire Protection System	15.4.15-1
15.4.16	Reactor Coolant System Pressure Isolation Valves Leakage Tests	15.4.16-1

b. Quadrant Power Tilt

Quadrant to average power tilt is expressed in percent as defined by the following equation:

$$100 \times \left(\frac{\text{power in any core quadrant}}{\text{average for all quadrants}} - 1 \right)$$

c. Operability

A system, subsystem, train, component, or device shall be operable or have operability when it is capable of performing its function(s) as analyzed in the safety analysis report. Implicit in this definition is the assumption that necessary instrumentation, controls, normal and emergency electrical power sources, cooling or seal water, lubrication or other auxiliary equipment required for the system, subsystem, train, component or device to perform its function(s) are capable of performing their related support function(s).

d. Containment Integrity*

Containment integrity is defined to exist when:

- 1) All non-automatic containment isolation valves and blind flanges are closed as required.
- 2) The equipment hatch is properly closed.
- 3) At least one door in each personnel air lock is properly closed.
- 4) All automatic containment isolation valves are operable or are secured closed.
- 5) The uncontrolled containment leakage satisfies Specification 15.4.4.

e. Protective Instrumentation Logic

1) Analog Channel

An analog channel is an arrangement of components and modules as required to generate a single protective action signal when required by a plant condition. An analog channel loses its identity where single action signals are combined.

*Containment isolation valves are discussed in FFDSAR Section 5.2.

Unit 1 - Amendment No. 43, 50

Unit 2 - Amendment No. 48, 56 15.1-2

15.3 Limiting Conditions for Operation

15.3.0 General Considerations

- A. Many of the Limiting Conditions for Operation (LCO) presented in these specifications provide a temporary relaxation of the single failure criterion, consistent with overall reliability considerations, to allow time periods during which corrective action may be taken to restore the system to full operability. If the situation has not been corrected within the specified time period, and the LCO prescribes no other specific action, an affected unit, which is critical, shall be placed in the hot shutdown condition within three hours. In the event an LCO cannot be satisfied because of equipment failures or limitations beyond those specified in the permissible conditions of the LCO, the affected unit, which is critical, shall be placed in the hot shutdown condition within three hours of discovery of the situation.
- B. If the conditions which prompted the shutdown required by 15.3.0.A cannot be corrected, many LCOs specify an additional time period until the unit must be placed in the cold shutdown condition. If no such time period is specified, the unit shall be put into the cold shutdown condition within 48 hours of discovering the situation.
- C. When a system, subsystem, train, component or device is determined to be inoperable solely because its emergency power source is inoperable, or solely because its normal power source is inoperable, the system, subsystem, train, component or device may be considered operable for the purpose of satisfying the requirements of the applicable Limiting Condition for Operation, provided: (1) the alternate power source (normal or emergency) is operable and (2) all redundant system(s), subsystem(s), train(s), component(s) and device(s) are operable.

If either condition (1) or (2) cannot be met, specifications 15.3.0.A and 15.3.0.B become applicable. This specification is not applicable during cold shutdown or refueling shutdown conditions.

- D. A momentary loss of normal or emergency power resulting in immediate corrective or required action in accordance with Table 15.3.5-2, i.e., placing associated Channels into the trip condition or shutdown of the unit, shall not be interpreted as causing a violation of the specification with respect to minimum operable channels or minimum degree of redundancy, unless said loss is the result of operator error or procedural violation.

Bases

Specifications 15.3.0.A and 15.3.0.B delineate the action to be taken for circumstances not directly provided for in the action statements of the LCO and whose occurrence would violate the intent of the specification. For example, Specification 15.3.3.A.2.e permits a single Reactor Coolant System accumulator to be isolated for up to one hour during power operations. Under the terms of Specifications 15.3.0.A and 15.3.0.B, if more than one accumulator is isolated or inoperable, the unit is required to be in hot shutdown within 3 hours of discovery of the condition and in the cold shutdown condition within the following 45 hours unless corrective measures are completed. As a further example, Specification 15.3.3.B.2.b permits one Containment Spray Pump to be out-of-service for up to 48 hours during power operations. Under the terms of these Specifications, if both of the required Containment Spray Pumps are inoperable, the unit is required to be in hot shutdown within 3 hours, and in the cold shutdown condition within the next 45 hours. It is assumed the unit is brought to the required condition within the required times by promptly initiating and carrying out the appropriate statement.

Specification 15.3.0.C delineates additional conditions which must be satisfied to permit operation to continue, consistent with the Limiting Condition for Operation statements for power sources, when a normal or emergency power source is not operable. It specifically prohibits operation when one system, subsystem, train, component or device is inoperable because its normal or emergency power source is inoperable and a redundant system, subsystem, train, component or device is inoperable for another reason.

The provisions of this specification permit the action statements associated with individual systems, subsystems, trains, components, or devices to be consistent with the action statements of the associated electrical power source. It allows operation to be governed by the time limits of the action statement associated with the Limiting Condition for Operation for the normal or emergency power source, not the individual action statements for each system, subsystem, train, component or device determined to be inoperable solely because of the inoperability of its normal or emergency power source.

For example, Specification 15.3.7.A.1.e allows a 7 day out-of-service time for one emergency diesel generator. If the definition of operable were applied without consideration of Specification 15.3.0.C, all systems, subsystems, trains, components or devices supplied by the inoperable emergency power source would also be inoperable. This would invoke the applicable action statements for each of the applicable Limiting Conditions for Operation. However, the provisions of Specification 15.3.0.C permit the time limits for continued operation to be consistent with the statement for the inoperable emergency diesel generator instead, provided the other specified conditions are satisfied. In this case, the corresponding normal power source must be operable, and all redundant systems, subsystems, trains, components, and devices must be operable, or otherwise satisfy Specification 15.3.0.C (i.e., be capable of performing their design function and have at least one normal or one emergency power source operable). If these conditions are not satisfied, shutdown is required in accordance with Specification 15.3.0.A.

As a further example, Specification 15.3.7.A.1.d requires in parts that 4160 volt buses A03 and A04 be energized for the unit to be taken critical. Specification 15.3.7.B.1.d permits either bus A03 or A04 to be taken out-of-service for up to 7 days provided both diesel generators are operable and the associated diesel generator is operating and providing power to the engineered safeguard bus normally supplied by the out-of-service bus. If the definition of operable were applied without consideration of Specification 15.3.0.C, all systems, subsystems, trains, components and devices supplied by the inoperable normal power sources (i.e., the out-of-service bus A03 or A04) would also be inoperable. This would invoke the applicable action statements for each of the applicable LCOs. However, the provisions of this Specification 15.3.0.C permit the time limit for continued operation to be consistent with the action statement for the inoperable normal power source, in this case 7 days, provided the other specified conditions are satisfied. These conditions are that for the engineered safeguards systems on one bus the emergency power source must be operable (as must be the components supplied by the emergency power source) and all redundant systems, subsystems, trains, components and devices in the other engineered safeguards systems must be operable, or likewise satisfy Specification 15.3.0.C (i.e., be capable of performing their design function and have an emergency power source operable). In other words, both emergency power sources must be operable and all redundant systems, subsystems, trains, components and devices in both divisions of engineered safeguards systems must also be operable. If these conditions are not satisfied, shutdown is required in accordance with this specification.

In the cold shutdown and refueling shutdown conditions, Specification 15.3.0.C is not applicable, and thus the individual action statements for each applicable Limiting Condition for Operation in these conditions must be adhered to.

Specification 15.3.0.D addresses the momentary loss of power to a component when immediate action is initiated resulting in reenergization from an alternate source, tripping the channel of logic or initiating operator action as specified in Table 15.3.5-2. Such a situation does

not constitute an unsafe condition. During the short period of the corrective or required action, the operator is sensitive to the condition of the unit and the possible effects of the logic systems, therefore the occurrence of such an event should not constitute a violation of the specification with respect to minimum operable channels or minimum degree of redundancy.

15.6.4 TRAINING

15.6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

15.6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Fire Protection Supervisor and shall meet or exceed the requirements of Section 27 of the NFPA Code-1976, except that the meeting frequency may be quarterly.

15.6.5 REVIEW AND AUDIT

15.6.5.1 Duty and Call Superintendents

- a. To assist and counsel the Shift Supervisor in case of significant operating events, a Duty and Call Superintendent Group has been established. The Duty and Call Superintendent Group shall consist of any qualified person designated by the Manager - Nuclear Operations.
- b. In the event of a reportable occurrence, the Shift Supervisor shall communicate with at least one Duty and Call Superintendent before taking other than the immediate on-the-spot action required. One Duty and Call Superintendent will be assigned to be "on call" at all times. The Duty and Call Superintendent provides continuously available counsel, call out backups, and review to the Shift Supervisor.

15.6.5.2 Manager's Supervisory Staff

FUNCTION

15.6.5.2.1 The Manager's Supervisory Staff (MSS) shall function to advise the Manager - Nuclear Operations on all matters related to nuclear safety.

COMPOSITION

15.6.5.2.2 The Manager's Supervisory Staff shall be composed of the:

Chairman: Manager - Nuclear Operations
Member: Superintendent - Operations
Member: Superintendent - Maintenance & Construction
Member: Instrument and Control Engineer
Member: Superintendent - Technical Services
Member: Radiochemical Engineer
Member: Health Physicist
Member: Assistant to the Manager - Nuclear Operations
Member: Reactor Engineer

ALTERNATES

15.6.5.2.3 Alternate members shall be appointed in writing by the MSS Chairman to serve on a temporary basis; however, no more than two alternates shall participate in MSS activities at any one time.

MEETING FREQUENCY

15.6.5.2.4 The MSS shall meet at least once per calendar month and as convened by the MSS Chairman.

QUORUM

15.6.5.2.5 A quorum of the MSS shall consist of the Chairman and four members including alternates.

RESPONSIBILITIES

15.6.5.2.6 The Manager's Supervisory Staff shall:

- a) Review existing and proposed normal, abnormal and emergency operating procedures. Review maintenance procedures and proposed changes to these procedures and other procedures or changes thereto as determined by the Manager to affect plant operational safety.

(Re: Section 15.6.7 for area of review.)

15.6.11 RADIATION PROTECTION PROGRAM

Specification

Radiological control procedures shall be written and made available to all station personnel, and shall state permissible radiation exposure levels. The radiation protection program shall meet the requirements of 10 CFR 20, with the exception of the following:

Paragraph 20.203 - Caution signs, labels and signals

In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mRem/hr but less than 1000 mRem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit* as described in Point Beach Nuclear Plant Health Physics Administrative Control Policies and Procedures Manual, Section 2.7. Any individual or group of individuals permitted to enter such areas shall be provided with at least one of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of these conditions.
- c. Coverage by an individual qualified in radiation protection procedures who is equipped with a radiation dose rate monitoring device. This individual shall be responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance as specified by the facility Health Physics Supervisor on the Radiation Work Permit.

The requirements above shall also apply to each high radiation area in which the intensity of radiation is greater than 100 mRem/hr. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Duty Shift Supervisor and the plant Health Physicist.

*Health Physics qualified personnel or personnel escorted by Health Physics personnel shall be exempt from the Radiation Work Permit issuance requirement during the performance of their assigned radiation protection duties, provided they comply with approved radiation protection procedures as they relate to entry into high radiation areas.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-266 AND 50-301WISCONSIN ELECTRIC POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 50 to Facility Operating License No. DPR-24, and Amendment No. 56 to Facility Operating License No. DPR-27 issued to Wisconsin Electric Power Company (the licensee), which revised Technical Specifications for operation of Point Beach Nuclear Plant, Unit Nos. 1 and 2 (the facilities) located in the Town of Two Creeks, Manitowoc County, Wisconsin. The amendments are effective as of the date of issuance.

The amendments consist of a revised definition of operability and new specifications addressing limiting conditions for operation and inoperability of safety related systems due solely to loss of their normal or emergency power supplies. They also make various administrative changes to the Technical Specifications.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration. 8107010 073

- 2 -

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated September 19, 1980, (2) Amendment Nos. 50 and 56 to License Nos. DPR-24 and DPR-27, and (3) letter of April 10, 1980 from Darrell G. Eisenhower to all power reactor licensees. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. 20555, and at the Joseph Mann Library, 1516 16th Street, Two Rivers, Wisconsin 54241. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 24th day of June, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing