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Docket Nos. 50-266
and 50-301

Mr. Sol Burstein
Executive Vice President
Wisconsin Electric Power Company
231 West Michigan Street
Milwaukee, Wisconsin 53201

Dear Mr. Burstein:

The Commission has issued the enclosed Amendment No. 52 to Facility Operating License No. DPR-24 and Amendment No. 58 to Facility Operating License No. DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated April 6, 1981 and modified by letter dated June 22, 1981 and are in partial response to your application dated February 4, 1981.

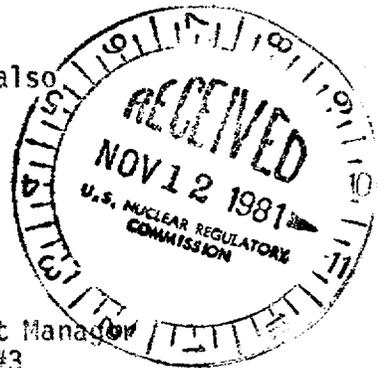
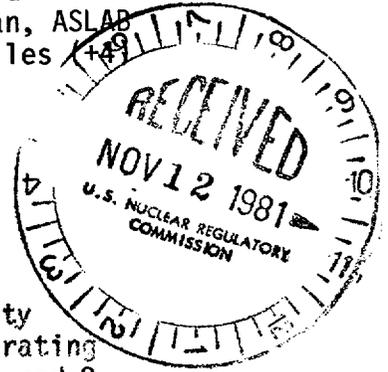
These amendments contain miscellaneous minor administrative changes which correct or clarify certain portions of the Technical Specifications and approve a revised organizational structure for the Point Beach facilities.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by:

Timothy G. Colburn, Project Manager
Operating Reactors Branch #3
Division of Licensing



Enclosures:

1. Amendment No. 52 to DPR-24
2. Amendment No. 58 to DPR-27
3. Safety Evaluation
4. Notice of Issuance

cc: w/enclosures
See next page

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CMT 8/20/81

FR NOTICE + AMENDMENT ONLY

OFFICE	ORB#3: Dp	ORB#2: DL	ORB#3: DL	AD-OR: DL	OELD		
SURNAME	PKreutzer	TColburn/pn	RAClark	TMNovak			
DATE	8/5/81	8/5/81	8/5/81	8/7/81	8/12/81		

Wisconsin Electric Power Company

cc:

Mr. Bruce Churchill, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N. W.
Washington, D. C. 20036

Mr. William Guldemon
USNRC Resident Inspectors Office
6612 Nuclear Road
Two Rivers, Wisconsin 54241

Joseph Mann Library
1516 Sixteenth Street
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Mr. Glenn A. Reed, Manager
Nuclear Operations
Wisconsin Electric Power Company
Point Beach Nuclear Plant
6610 Nuclear Road
Two Rivers, Wisconsin 54241

Mr. Gordon Blaha
Town Chairman
Town of Two Creeks
Route 3
Two Rivers, Wisconsin 54241

Ms. Kathleen M. Falk
General Counsel
Wisconsin's Environmental Decade
302 E. Washington Avenue
Madison, Wisconsin 53703

U. S. Environmental Protection Agency
Federal Activities Branch
Region V Office
ATTN: EIS COORDINATOR
230 S. Dearborn Street
Chicago, Illinois 60604

cc w/enclosure(s) and incoming
dtd: 4/6/81, 6/22/81, 2/4/81

Chairman
Public Service Commission of Wisconsin
Hills Farms State Office Building
Madison, Wisconsin 53702



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 52
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Wisconsin Electric Power Company (the licensee) dated February 4, 1981 and April 6, 1981 as modified by letter dated June 22, 1981, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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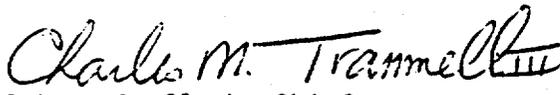
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 52, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for 
Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 20, 1981



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 58
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Wisconsin Electric Power Company (the licensee) dated February 4, 1981 and April 6, 1981 as modified by letter dated June 22, 1981, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 58, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for 
Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 20, 1981

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 52 TO FACILITY OPERATING LICENSE NO. DPR-24

AMENDMENT NO. 58 TO FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NOS. 50-266 AND 50-301

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
15.2.3-6	15.2.3-6
15.3.2-3	15.3.2-3
15.4.9-1	15.4.9-1
15.4.9-2	15.4.9-2
15.5.1-1	15.5.1-1
Figure 15.6.2-1	Figure 15.6.2-1
Figure 15.6.2-2	Figure 15.6.2-2
Figure 15.6.2-3	Figure 15.6.2-3
Figure 15.6.2-4	Figure 15.6.2-4
15.6.3-1	15.6.3-1
-	15.6.3-2
15.6.5-2	15.6.5-2

power distribution, the reactor trip limit, with allowance for errors, (2) is always below the core safety limit as shown on Figure 15.2.1-1. If axial peaks are greater than design, as indicated by difference between top and bottom power range nuclear detectors, the reactor trip limit is automatically reduced. (6) (7)

The overpower, overtemperature and pressurizer pressure system setpoints have been revised to include effect of reduced system pressure operation (including the effects of fuel densification). The revised setpoints as given above will not exceed the revised core safety limits as shown in Figure 15.2.1-1.

The overpower limit criteria is that core power be prevented from reaching a value at which fuel pellet centerline melting would occur. The reactor is prevented from reaching the overpower limit condition by action of the nuclear overpower and overpower ΔT trips.

The high and low pressure reactor trips limit the pressure range in which reactor operation is permitted. The high pressurizer pressure reactor trip setting is lower than the set pressure for the safety valves (2485 psig) such that the reactor is tripped before the safety valves actuate. The low pressurizer pressure reactor trip trips the reactor in the unlikely event of a loss-of-coolant accident. (4)

The low flow reactor trip protects the core against DNB in the event of either a decreasing actual measured flow in the loops or a sudden loss of power to one or both reactor coolant pumps. The set point specified is consistent with the value used in the accident analysis. (8) The low loop flow signal is caused by a condition of less than 90% flow as measured by the loop flow instrumentation. The loss of power signal is caused by

D. During power operation the requirements of 15.3.2-B and C may be modified to allow the following components to be inoperable for a specified time. If the system is not restored to meet the requirements of 15.3.2-B or C within the time period specified, the appropriate reactor(s) except as otherwise noted, shall be placed in the hot shutdown condition. If the requirements of 15.3.2-B or C are not satisfied within an additional 48 hours, the appropriate reactor(s) shall be placed in the cold shutdown condition.

1. One of the two operable charging pumps associated with an operating reactor may be removed from service provided a charging pump associated with that same reactor is restored to operable status within 24 hours.
2. One of the boric acid transfer pumps designated in B.2 or C.2 may be out of service provided a pump is restored to operable status within 24 hours.
3. For the system piping and valve operability requirements (B.4 and C.4):
 - a. The flow path from the boric acid tank to a reactor coolant system may be out of service provided the flow path is restored to operable status within 24 hours.
 - b. The flow path from the refueling water storage tank to the reactor coolant system may be out of service provided the flow path is restored to operable status within one hour. If the flow path cannot be restored to operable status within one hour, the reactor shall be placed in cold shutdown within the next 30 hours.

Applicability

Applies to potential reactivity anomalies.

Objective

To require evaluation of reactivity anomalies within the reactor.

Specification

Following a normalization of the computed boron concentration as a function of burnup, the actual boron concentration of the coolant shall be periodically compared with the predicted value. If the difference between the observed and predicted steady-state concentrations reaches the equivalent of one percent in reactivity, an evaluation as to the cause of discrepancy shall be made and reported to the Nuclear Regulatory Commission.

Basis

To eliminate possible errors in the calculations of the initial reactivity of the core and the reactivity depletion rate, the predicted relation between fuel burn-up and the boron concentration necessary to maintain adequate control characteristics, must be adjusted (normalized) to accurately reflect actual core condition. When full power is reached initially, and with the control rod groups in the desired positions, the boron concentration is measured and the predicted curve is adjusted to this point. As power operation proceeds, the measured boron concentration is compared with the predicted concentration and the slope of the curve relating burn-up and reactivity is compared with that predicted. This process of normalization should be completed after about 10% of the total core burn-up. Thereafter,

actual boron concentration can be compared with prediction, and the reactivity status of the core can be continuously evaluated. Any reactivity anomaly greater than 1% would be unexpected, and its occurrence would be thoroughly investigated and evaluated.

The value of 1% is considered a safe limit since a shutdown margin of at least 1% with the most reactive rod in the fully withdrawn position is always maintained.

Reference

FSAR - Section 3.2.1

15.5 DESIGN FEATURES

15.5.1 SITE

Applicability

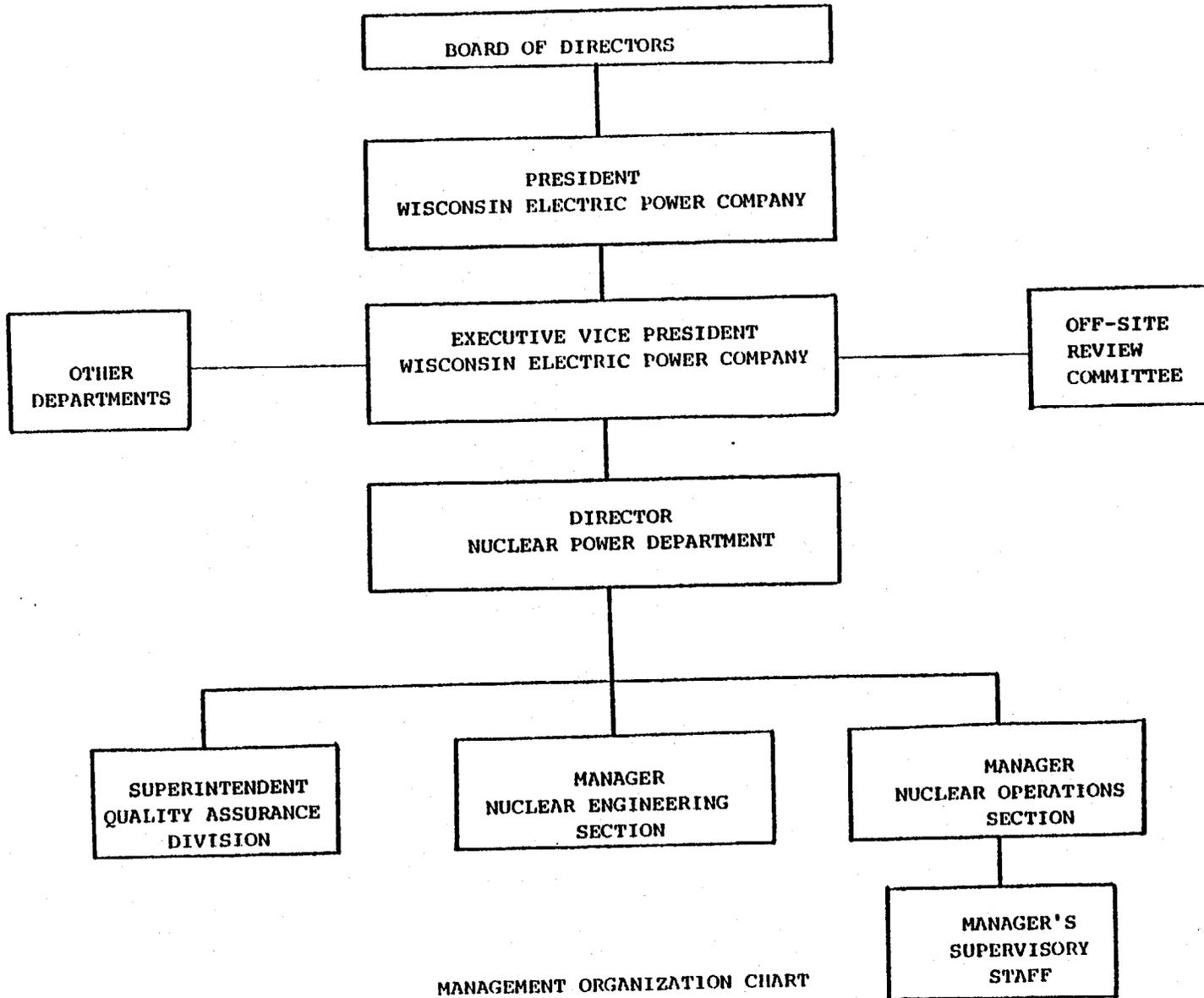
Applies to the location and extent of the reactor site.

Objective

To define those aspects of the site which affect the overall safety of the installation.

Specification

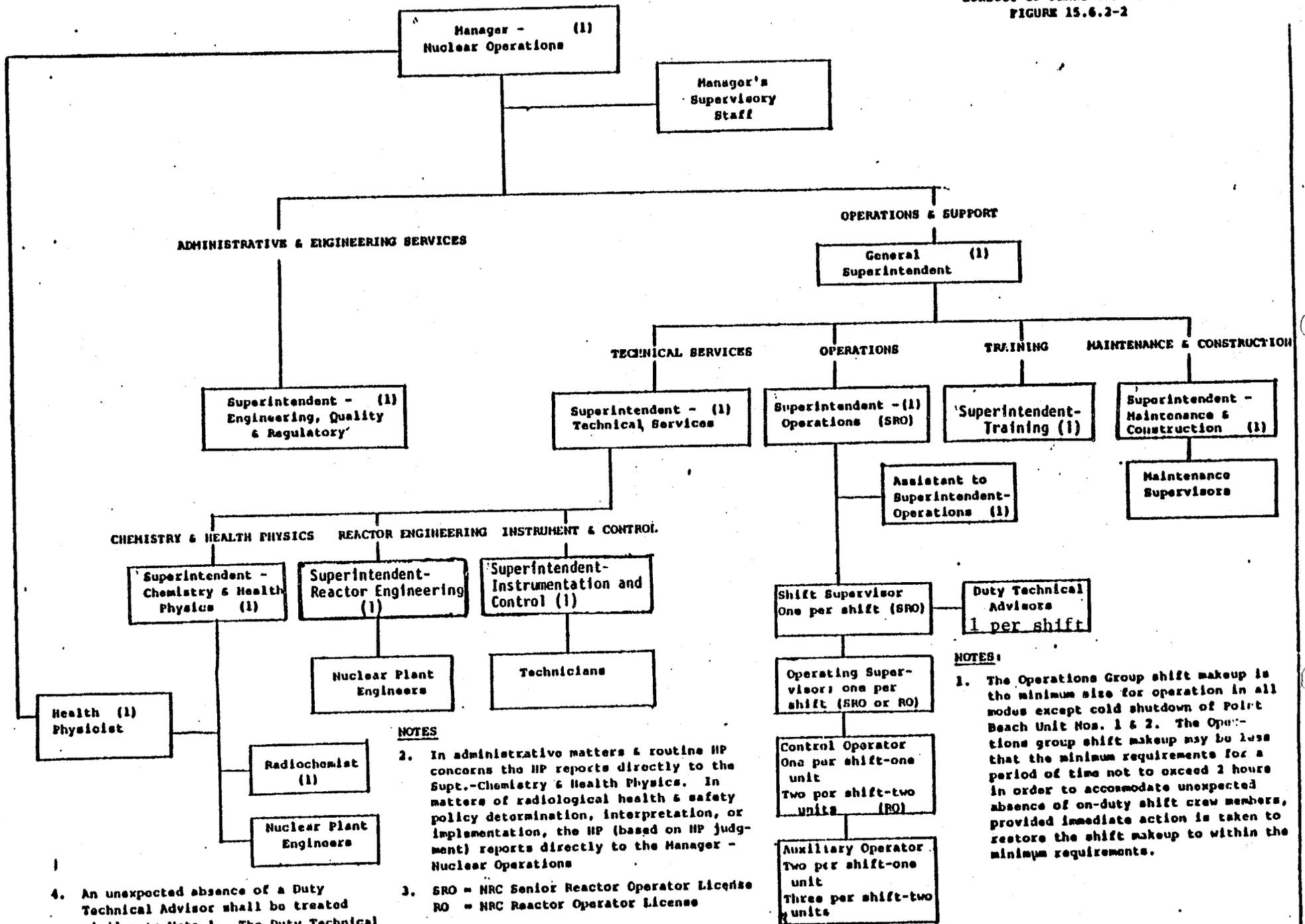
The Point Beach Nuclear Plant is located on property owned by Wisconsin Electric Power Company at a site on the shore of Lake Michigan, approximately 30 miles southeast of the city of Green Bay. The minimum distance from the reactor containment center line to the site exclusion boundary as defined in 10 CFR 100.3 is 1200 meters.



MANAGEMENT ORGANIZATION CHART

Figure 15.6.2-1

Unit No. 1 - Amendment No. 43, 52
 Unit No. 2 - Amendment No. 48, 58

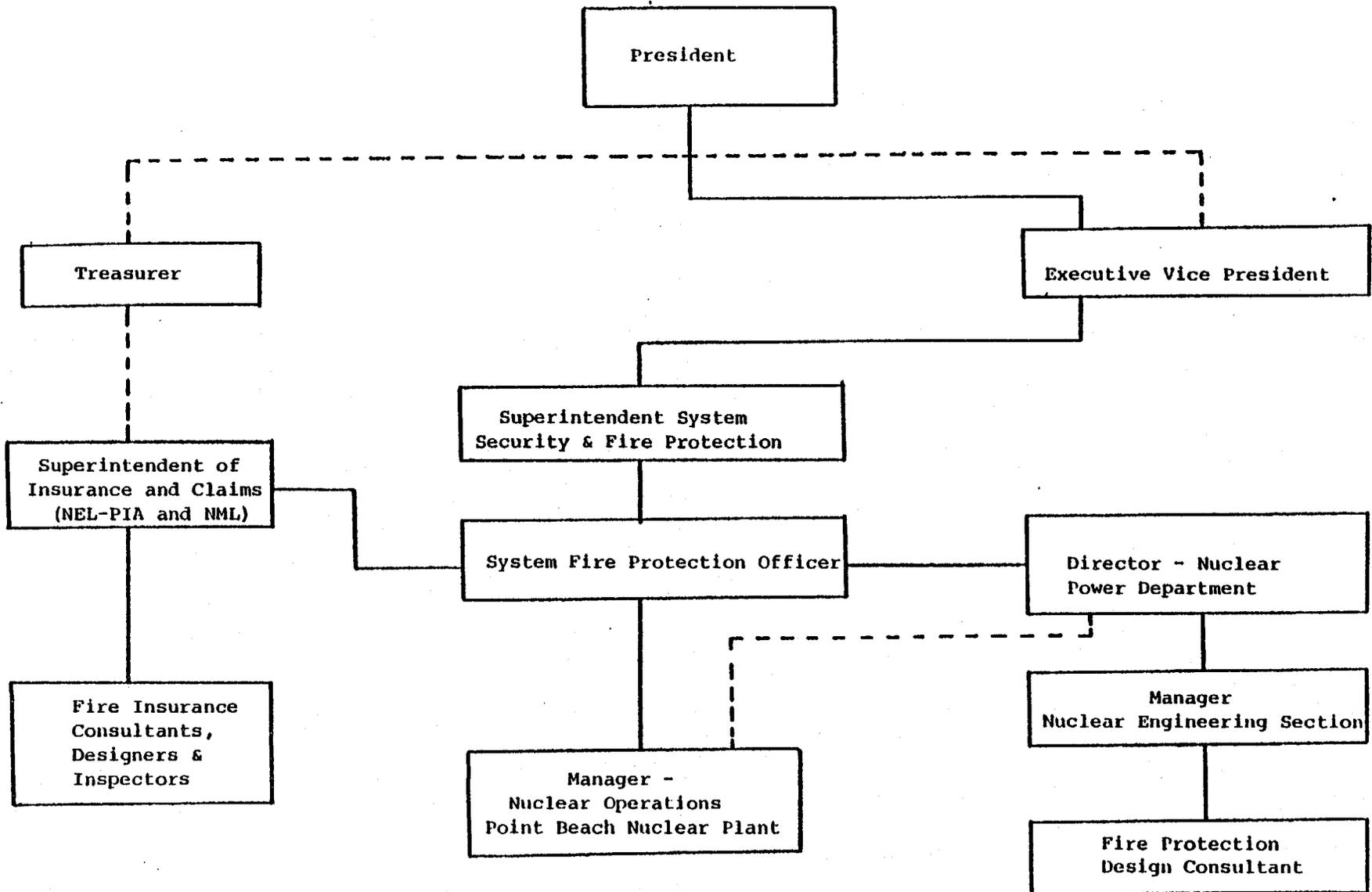


4. An unexpected absence of a Duty Technical Advisor shall be treated similar to Note 1. The Duty Technical Advisor is located on-site on ten minute call to the control room.

- NOTES**
- In administrative matters & routine HP concerns the HP reports directly to the Supt.-Chemistry & Health Physics. In matters of radiological health & safety policy determination, interpretation, or implementation, the HP (based on HP judgment) reports directly to the Manager - Nuclear Operations
 - SRO = NRC Senior Reactor Operator License
RO = NRC Reactor Operator License

NOTES:

- The Operations Group shift makeup is the minimum size for operation in all modes except cold shutdown of Point Beach Unit Nos. 1 & 2. The Operations group shift makeup may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members, provided immediate action is taken to restore the shift makeup to within the minimum requirements.



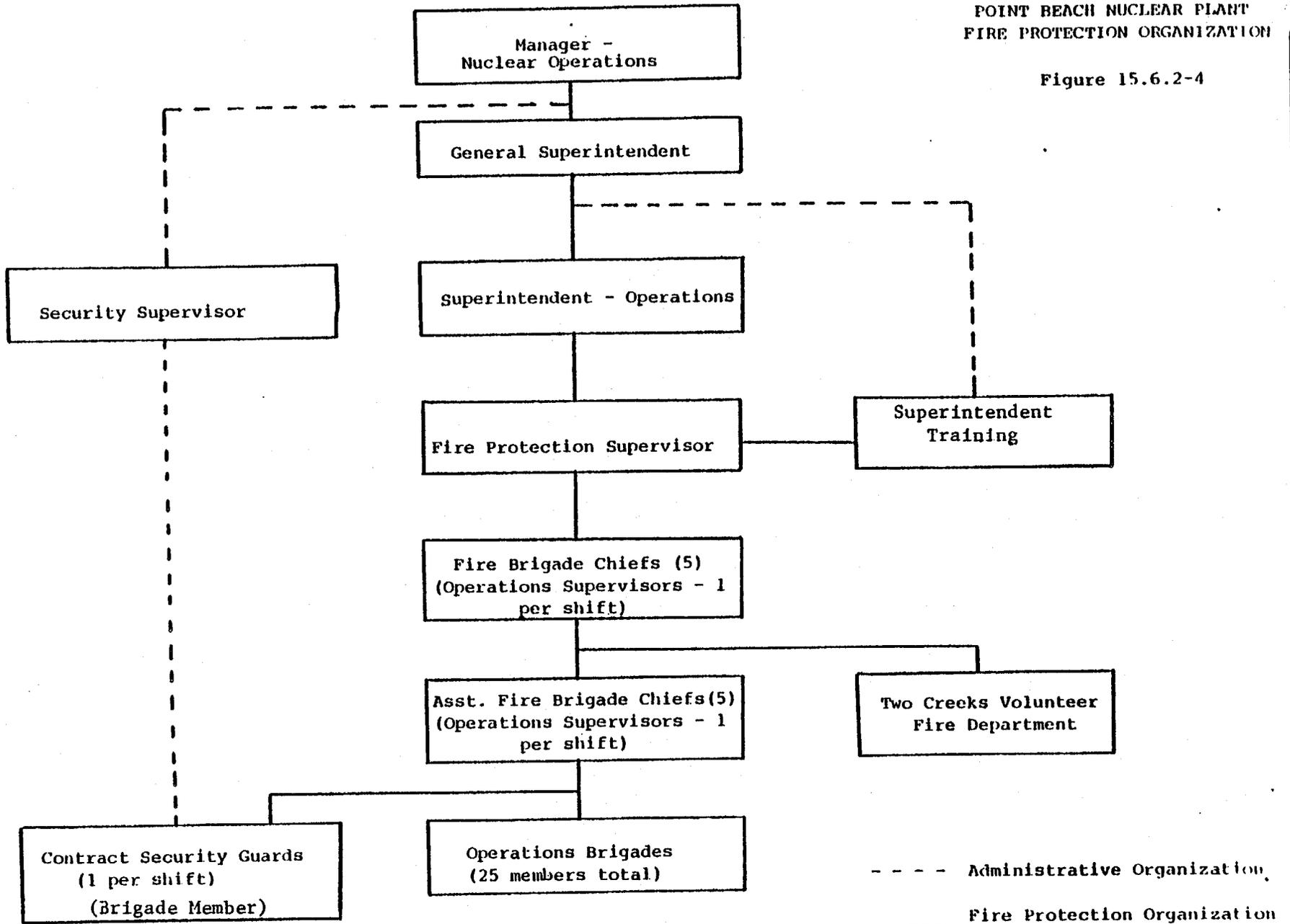
WISCONSIN ELECTRIC POWER COMPANY
 OFF-SITE MANAGEMENT
 FIRE PROTECTION ORGANIZATION

- - - - Administrative Organization
 _____ Fire Protection Organization

Figure 15.6.2-3

POINT BEACH NUCLEAR PLANT
FIRE PROTECTION ORGANIZATION

Figure 15.6.2-4



Unit No. 1 - Amendment No. 43, 52
 Unit No. 2 - Amendment No. 48, 58

15.6.3 Facility Staff Qualifications

15.6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions or as clarified in 15.6.3.2 through 15.6.3.4.

15.6.3.2 Except as provided in 15.6.3.3, either the Superintendent - Chemical and Health Physics or the Health Physicist shall meet the following requirements:

- a. The individual shall have a bachelor's degree or the equivalent in a science or engineering subject, including some formal training in radiation protection. For purposes of this paragraph, "equivalent" is as follows:
 - (1) Four years of formal schooling in science or engineering; or
 - (2) Four years of applied radiation protection experience at a nuclear facility; or
 - (3) Four years of operational or technical experience or training in nuclear power; or
 - (4) Any combination of the above totalling four years.
- b. Except as provided in d., below, the individual shall have at least five years of professional experience in applied radiation protection. A master's degree in a related field is equivalent to one year of experience and a doctor's degree in a related field is equivalent to two years of experience.
- c. Except as provided in d., below, at least three of the five years of experience shall be in applied radiation protection work in a nuclear facility dealing with radiological problems similar to those encountered in nuclear power plants.
- d. If the individual has a bachelor's degree specifically in health physics, radiological health, or radiation protection, at least three years of professional experience is required; if the individual has a master's or a doctor's degree specifically in health physics, radiological health, or radiation protection, at least two years of professional experience is required. This experience shall be in applied radiation protection in a nuclear facility dealing with radiological problems similar to those encountered in nuclear power plants.

15.6.3.3 In the event the position of Superintendent - Chemical and Health Physics or Health Physicist is vacated and neither the remaining individual nor the proposed replacement meets the qualifications of 15.6.3.2, but one of these individuals is determined to be otherwise well qualified, then concurrence of NRC shall be sought in approving the qualification of that individual.

15.6.3.4 The Duty Technical Advisor shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents. The Duty Technical Advisor shall also receive training in plant design and layout including the capabilities of instrumentation and controls in the control room.

COMPOSITION

15.6.5.2.2 The Manager's Supervisory Staff shall be selected from the following:

Chairman: Manager - Nuclear Operations
Member: General Superintendent
Member: Superintendent - Operations
Member: Superintendent - Maintenance & Construction
Member: Superintendent - Engineering, Quality and Regulatory
Member: Superintendent - Chemistry and Health Physics
Member: Superintendent - Technical Services
Member: Superintendent - Instrumentation and Control
Member: Health Physicist
Member: Superintendent - Reactor Engineering
Member: Superintendent - Training

ALTERNATES

15.6.5.2.3 Alternate members shall be appointed in writing by the MSS Chairman to serve on a temporary basis; however, no more than two alternates shall participate in MSS activities at any one time.

MEETING FREQUENCY

15.6.5.2.4 The MSS shall meet at least once per calendar month and as convened by the MSS Chairman.

QUORUM

15.6.5.2.5 A quorum of the MSS shall consist of the Chairman and four members including alternates.

RESPONSIBILITIES

15.6.5.2.6 The Manager's Supervisory Staff shall:

- a) Review existing and proposed normal, abnormal and emergency operating procedures. Review maintenance procedures and proposed changes to these procedures and other procedures or changes thereto as determined by the Manager to affect plant operational safety.

(Re: Section 15.6.7 for area of review.)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 52 TO FACILITY OPERATING LICENSE NO. DPR-24
AND AMENDMENT NO. 58 TO FACILITY OPERATING LICENSE NO. DPR-27

WISCONSIN ELECTRIC POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-266 AND 50-301

Introduction

By letter dated April 6, 1981 and modified by letter dated June 22, 1981, Wisconsin Electric Power Company (the licensee) submitted an application for amendments to the licenses for the Point Beach Nuclear Plant, Units 1 and 2. The amendments would make several administrative changes to the Technical Specifications. Previously, on February 4, 1981, the licensee had submitted an amendment request in response to Darrell Eisenhut's letter to all pressurized water reactor licensees dated July 2, 1980. Part of that submittal addressed the qualifications and duty status requirements for the Duty Technical Advisors. This issue is also addressed in these amendments.

Discussion and Evaluation

Many of the proposed changes involve minor clarifications and corrections and are acceptable. In addition, the licensee is proposing title changes to the corporate organization to best describe the management's function. The licensee also proposes that the plant staff organization be changed to provide for the addition of a General Superintendent reporting to the Manager-Nuclear Operations and having direct responsibility for plant operations and direct support. Further, several titles of plant staff personnel have been changed. The position of Duty Technical Advisor has also been included in the Operations Group shift makeup. This addition was originally proposed in licensee's amendment application to the NRC dated February 4, 1981 regarding Technical Specifications for TMI related changes filled at the Point Beach Nuclear Plant since January 1, 1980.

The licensee also proposes that certain plant staff positions are not critical key positions and wishes to make changes to the facility staff organization which affect these positions without prior NRC approval.

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After reviewing the proposed changes, we conclude that Figure 15.6.2-2 does not indicate that the following plant positions are critical:

- (1) General Superintendent
- (2) Superintendent of Technical Services
- (3) Superintendent - Training, and
- (4) Superintendent Engineering, Quality and Regulatory

We cannot agree that the positions mentioned above are not key. Confusion during the TMI incident was inherent because the Plant Manager and Technical Support Superintendent were very new to their jobs and the licensee was not required to have a qualified backup.

We believe these positions are important and that prior NRC approval is required before planned changes to these positions are made.

Therefore, we cannot accept this portion of the proposed Technical Specification change which deals with the designation of non-key plant personnel and their subsequent changes without prior NRC approval.

The addition of the Duty Technical Advisor to the staff organization is in accordance with the standard technical specifications for Westinghouse pressurized water reactors and is acceptable.

The proposed title changes to organizational staff are considered minor administrative changes and are acceptable.

Telephone conversations with members of the licensee's staff were held during the evaluation of the licensee's submittals to clarify certain of the proposals. The licensee approved the following minor changes to the proposed Technical Specifications:

1. "The Duty Technical Advisor shall also receive training in plant design and layout including the capabilities of instrumentation and controls in the control room." was added to section 15.6.3.4 to make it more closely conform to the standard technical specifications.
2. Section 15.6.9.2.A required no change from that previously approved.
3. Figure 15.6.2-2 has been modified to show a solid line in the reporting relationship between the Health Physicist and the Manager-Nuclear Operations in order to conform to previously approved technical specifications. Also the dotted line reporting relationship between the Duty Technical Advisors and the Superintendent-Training has been deleted. The licensee's staff has stated that these were in error of their intent.
4. Figure 15.6.2.2 has been modified to add the words "one per shift" under the position of Duty Technical Advisor in order to conform with the licensee's original February 4, 1981 submittal.

5. Figure 15.6.2-4 and section 15.6.5.2.2 were modified substituting "Superintendent - Training" for "Training Engineer" and "Superintendent-Instrumentation and Control" for "Instrumentation and Control Engineer" to conform with the licensee's June 22, 1981 submittal.
6. Figure 15.6.2-4 was also modified to include the designation of "brigade member" under the position of Contract Security Guards to conform with that previously approved by the NRC.
7. A typing error was corrected on page 15.4.9-1.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: August 20, 1981

Timothy G. Colburn ORB#3
James C. Snell LQB

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-266 AND 50-301WISCONSIN ELECTRIC POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 52 to Facility Operating License No. DPR-24, and Amendment No. 58 to Facility Operating License No. DPR-27 issued to Wisconsin Electric Power Company (the licensee), which revised Technical Specifications for operation of Point Beach Nuclear Plant, Unit Nos. 1 and 2 (the facilities) located in the Town of Two Creeks, Manitowoc County, Wisconsin. The amendments are effective as of the date of issuance.

The amendments contain miscellaneous minor administrative changes which correct or clarify certain portions of the Technical Specifications and approve a revised organizational structure for the Point Beach facilities.

The applications for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

- 2 -

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated February 4, 1981 and April 6, 1981 as modified by letter dated June 22, 1981, (2) Amendment Nos. 52 and 58 to License Nos. DPR-24 and DPR-27, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. 20555, and at the Joseph Mann Library, 1516 16th Street, Two Rivers, Wisconsin 54241. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 20th day of August, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION


Charles M. Trammell, Acting Chief
Operating Reactors Branch #3
Division of Licensing