

ACS MS-014

JUL 27 1982

Docket Nos. 50-266
and 50-301

Mr. C. W. Fay
Assistant Vice President
Wisconsin Electric Power Company
231 West Michigan Street
Milwaukee, Wisconsin 53201

Dear Mr. Fay:

The Commission has issued the enclosed Amendment No. 62 to Facility Operating License No. DPR-24 and Amendment No. 67 to Facility Operating License No. DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in partial response to your application transmitted by letter dated April 27, 1982 as modified by letters dated May 5, 1982 and June 30, 1982.

These amendments modify the limiting conditions for operation of the Technical Specifications with respect to auxiliary feedwater pump operation. Specifically, they modify the following sections of the Point Beach Nuclear Plant Technical Specifications:

1. Specification 15.3.4.A.2 is revised to require that both motor-driven AFW pumps and the associated turbine-driven pump be operable before a unit may be taken critical.
2. Specification 15.3.4.C is revised to require the associated unit to be shut down if a turbine-driven AFW pump is taken out of service and cannot be restored to service within 72 hours. The revised specification also requires that both units be shut down if a motor-driven AFW pump is taken out of service and cannot be restored to service within seven days.

By our letter of April 21, 1982 we issued our Safety Evaluation (SER) regarding NUREG-0737 Item II.E.1.1, "Auxiliary Feedwater System Evaluation", for the Point Beach Nuclear Plant, Units 1 and 2. Our resolution of this item for your facilities was contingent upon your commitment to submit proposed Technical Specification changes to resolve the NRC staff's concerns regarding limiting conditions for operation of the auxiliary feedwater pumps. We requested these changes be submitted within 45 days receipt of our April 21, 1982 letter.

Your application for amendments dated April 27, 1982 is responsive to our requests. The staff has reviewed your proposed changes to the Technical Specifications and finds them acceptable. The reasons for our acceptance are documented in our April 21, 1982 SER which constitutes our Safety Evaluation on this matter.

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In your May 5, 1982 letter you stated that the modifications to your originally proposed Technical Specification change request result in the same restrictions on pump availability as did those originally proposed but were more desirable because they simplified the overall specification. While the staff agrees with you that the format of the May 5, 1982 proposed Technical Specification changes more closely duplicate the format of your Technical Specifications than do those originally proposed in your April 27, 1982 letter, we do not agree that they place the same restrictions on pump availability. Therefore, as discussed with and agreed to by your staff by telephone, we are approving your original change request, which the staff feels satisfies our requirements.

Your letter of April 27, 1982 requested that these amendments be made effective no earlier than 120 days from the date of your application in order to allow you to procure a sufficient inventory of spare parts for pump repairs. As discussed previously with members of your staff by telephone conversations, this is acceptable to the NRC staff.

As further discussed with members of your staff, the proposed Technical Specification changes regarding testing of automatic actuation circuitry for the auxiliary feedwater pumps will be addressed in separate correspondence following completion of the circuitry modifications described in your June 30, 1982 submittal. The staff requires that you inform us in writing of the completion of these modifications in order that the appropriate Technical Specifications may be issued. Please reference this letter.

We have evaluated the potential for environmental impact of plant operation in accordance with the enclosed amendments and have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

The amendments do not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. They do not involve a significant increase in the probability or consequences of an accident previously evaluated, do not create the possibility of an accident of a type different from any evaluated previously and do not involve a significant reduction in a margin of

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Mr. C. W. Fay

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safety, the amendments do not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

A copy of the Notice of Issuance is enclosed.

Sincerely,

Original signed by

Timothy G. Colburn, Project Manager
Operating Reactors Branch #3
Division of Licensing

Enclosures:

- 1. Amendment No. 6 2 to DPR-24
- 2. Amendment No. 6 7 to DPR-27
- 3. Notice of Issuance

cc: w/enclosures
See next page

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→ AMENDMENT

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Wisconsin Electric Power Company

cc:

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Town Chairman
Town of Two Creeks
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Ms. Kathleen M. Falk
General Counsel
Wisconsin's Environmental Decade
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U. S. Environmental Protection Agency
Federal Activities Branch
Region V Office
ATTN: Regional Radiation
Representative
230 S. Dearborn Street
Chicago, Illinois 60604

cc w/enclosure(s) and incoming
dtd: 4/27/82, 5/5/82, 6/30/82

Chairman
Public Service Commission of Wisconsin
Hills Farms State Office Building
Madison, Wisconsin 53702

Regional Administrator
Nuclear Regulatory Commission, Region III
Office of Executive Director for Operations
799 Toosevelt Road
Glen Ellyn, Illinois 60137



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 62
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated April 27, 1982, as supplemented June 30, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

DESIGNATED ORIGINAL

Certified By

J. J. Noonan

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 62, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 60 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 27, 1982



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 67
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated April 27, 1982, as supplemented June 30, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

DESIGNATED ORIGINAL

Certified By

J. J. Noonan

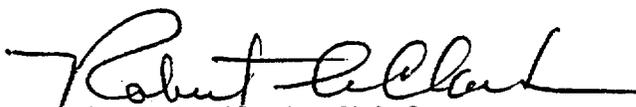
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 67, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 60 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 27, 1982

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 62 TO FACILITY OPERATING LICENSE NO. DPR-24

AMENDMENT NO. 67 TO FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NOS. 50-266 AND 50-301

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove*

15.3.4-1
15.3.4-2
15.3.4-2a

Insert

15.3.4-1
15.3.4-2
15.3.4-2a

*Changes on the revised TS pages do not become effective until 60 days after the date of issuance. In the interim, the existing TS pages are in effect and should be retained.

15.3.4 STEAM AND POWER CONVERSION SYSTEM

Applicability

Applies to the operating status of steam and power conversion system.

Objective

To define conditions of the steam and power conversion system steam-relieving capacity. Auxiliary Feedwater System and Service Water System operation is necessary to ensure the capability to remove decay heat from the core.

Specification

- A. When the reactor coolant is heated above 350°F the reactor shall not be taken critical unless the following conditions are met:
1. A minimum steam-relieving capability of eight (8) main steam safety valves available, except for low power physics testing.
 2. Auxiliary Feedwater System
 - a. Two Unit Operation - All four auxiliary feedwater pumps together with their associated flow paths and essential instrumentation shall be operable.
 - b. Single Unit Operation - Both motor driven auxiliary feedwater pumps and the turbine driven auxiliary feedwater pump associated with that unit together with their associated flow paths and essential instrumentation shall be operable.

Unit 1 - Amendment No. 62

Unit 2 - Amendment No. 67

15.3.4-1

3. A minimum of 10,000 gallons of water per operating unit in the condensate storage tanks and an unlimited water supply from the lake via either leg of the plant Service Water System.
 4. System piping and valves required to function during accident conditions directly associated with the above components operable.
- B. The iodine-131 activity on the secondary side of the steam generator shall not exceed 1.2 $\mu\text{Ci/cc}$.
- C. During power operation the requirements of 15.3.4.A.2.a and b may be modified to allow the following components to be inoperable for a specified time. If the system is not restored to meet the requirements of 15.3.4.A.2.a and b within the time period specified, the specified action must be taken. If the requirements of 15.3.4.A.2.a and b are not satisfied within an additional 48 hours, the appropriate reactor(s) shall be cooled down to less than 350°F.
1. Two Unit Operation - One of the four operable auxiliary feedwater pumps may be out-of-service for the below specified times. A turbine driven auxiliary feedwater pump may be out of service for up to 72 hours. If the turbine driven auxiliary feedwater pump cannot be restored to service within the 72 hour time period the associated reactor shall be shut down and in hot shutdown within the next 12 hours. A motor driven auxiliary feedwater pump may be out of service for up to 7 days. If the inoperable motor driven auxiliary feedwater pump cannot be restored to service within the 7 day time period both of the reactors shall be shut down and in hot shutdown within the next 12 hours.

Unit 1 - Amendment No. 28, 62

Unit 2 - Amendment No. 31, 67

15.3.4-2

2. Single Unit Operation - The turbine driven auxiliary feedwater pump may be out-of-service for up to 72 hours. If the turbine driven auxiliary feedwater pump cannot be restored to service within that 72 hour time period, the reactor shall be shut down and in hot shutdown within the next 12 hours. Either one of the two motor driven auxiliary feedwater pumps may be out-of-service for up to 7 days. If the motor driven auxiliary feedwater pump cannot be restored to service within that 7 day period the operating unit shall be shut down and in hot shutdown within the next 12 hours.

Basis

A reactor shutdown from power requires removal of core decay heat. Immediate decay heat removal requirements are normally satisfied by the steam by pass to the condenser. Therefore, core decay heat can be continuously dissipated via the steam bypass to the condenser as feedwater in the steam generator is converted to steam by heat absorption. Normally, the capability to return feedwater flow to the steam generators is provided by operation of the turbine cycle feedwater system.

The eight main steam safety valves have a total combined rated capability of 6,664,000 lbs/hr. The total full power steam flow is 6,620,000 lbs/hr, therefore eight (8) main steam safety valves will be able to relieve the total full-power steam flow if necessary.

In the unlikely event of complete loss of electrical power to the station, decay heat removal would continue to be assured for each unit by the availability of either the steam-driven auxiliary feedwater pump or one of the two motor-driven auxiliary steam generator feedwater pumps, and steam discharge to the atmosphere via the main steam safety valves or atmospheric relief valves. One motor-driven auxiliary feedwater pump can supply sufficient feedwater for removal of decay heat from a unit. The minimum amount of water in the condensate storage tanks is the amount needed for 25 minutes of operation/unit, which allows sufficient time for operator action.

An unlimited supply is available from the lake via either leg of the plant service water system for an indefinite time period.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-266 AND 50-301WISCONSIN ELECTRIC POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

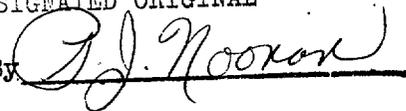
The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 62 to Facility Operating License No. DPR-24, and Amendment No. 67 to Facility Operating License No. DPR-27 issued to Wisconsin Electric Power Company (the licensee), which revised Technical Specifications for operation of Point Beach Nuclear Plant, Unit Nos. 1 and 2 (the facilities) located in the Town of Two Creeks, Manitowoc County, Wisconsin. The amendments are effective 60 days from date of issuance.

The amendments modify the limiting conditions for operation of the Technical Specifications with respect to the auxiliary feedwater pump operation.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

DESIGNATED ORIGINAL

Certified By



- 2 -

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated April 27, 1982, as supplemented June 30, 1982, (2) Amendment Nos. 62 and 67 to License Nos. DPR-24 and DPR-27, and (3) the Commission's letter dated July 27, 1982. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. 20555, and at the Joseph Mann Library, 1516 16th Street, Two Rivers, Wisconsin 54241. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 27th day of July, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing