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Docket Nos. 50-266
and 50-301

Mr. Sol Burstein
Executive Vice President
Wisconsin Electric Power Company
231 West Michigan Street
Milwaukee, Wisconsin 53201

Dear Mr. Burstein:

The Commission has issued the enclosed Amendment Nos. 47 and 52 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated July 28, 1977 as modified by letters dated August 24, 1979, January 17, 1980, April 17, 1980 and September 22, 1980.

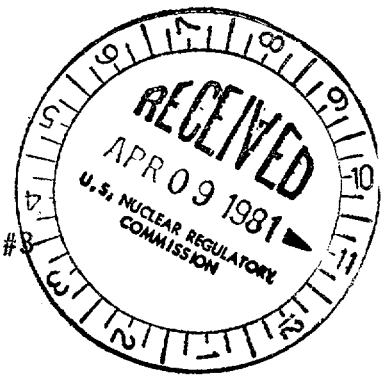
These amendments reflect modifications to plant equipment to provide additional assurance of the proper functioning of the plant's safety related electrical and emergency power systems.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by
Robert A. Clark

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing



Enclosures:
1. Amendment No. 47 to DPR-24
2. Amendment No. 52 to DPR-27
3. Safety Evaluation
4. Notice of Issuance

cc: w/enclosures
See next page

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OFFICE	ORB#3:DL	ORB#3:DL	ORB#3:DL	AD:OR:DL	OELD		
SURNAME	PKreutzer	TColburn	RAClark	TMNovak	BARTH		
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

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PMKreutzer

Docket No. 50-266 and 50-301

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: WISCONSIN ELECTRIC POWER COMPANY, Point Beach Nuclear Plant, Units No. 1 and 2.

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (12) of the Notice are enclosed for your use.

☐ Notice of Receipt of Application for Construction Permit(s) and Operating License(s).

☐ Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.

☐ Notice of Availability of Applicant's Environmental Report.

☐ Notice of Proposed Issuance of Amendment to Facility Operating License.

☐ Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.

☐ Notice of Availability of NRC Draft/Final Environmental Statement.

☐ Notice of Limited Work Authorization.

☐ Notice of Availability of Safety Evaluation Report.

☐ Notice of Issuance of Construction Permit(s).

☐ Notice of Issuance of Facility Operating License(s) or Amendment(s).

☒ Other: Amendment Nos. 47 and 52
Referenced documents have been provided PDR.

Division of Licensing, ORB#3
Office of Nuclear Reactor Regulation

Enclosure:
As Stated

ORB#3:DL
PMKreutzer/pn
/ /81

NRC FORM 102
7-79

Wisconsin Electric Power Company

cc:
Mr. Bruce Churchill, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N. W.
Washington, D. C. 20036

Joseph Mann Library
1516 Sixteenth Street
Two Rivers, Wisconsin 54241

Mr. Glenn A. Reed, Manager
Nuclear Operations
Wisconsin Electric Power Company
Point Beach Nuclear Plant
6610 Nuclear Road
Two Rivers, Wisconsin 54241

Mr. Gordon Blaha
Town Chairman
Town of Two Creeks
Route 3
Two Rivers, Wisconsin 54241

Ms. Kathleen M. Falk
General Counsel
Wisconsin's Environmental Decade
302 E. Washington Avenue
Madison, Wisconsin 53703

Director, Criteria and Standards Division
Office of Radiation Programs (ANR-460)
U.S. Environmental Protection Agency
Washington, D. C. 20460

U. S. Environmental Protection Agency
Federal Activities Branch
Region V Office
ATTN: EIS COORDINATOR
230 S. Dearborn Street
Chicago, Illinois 60604

cc w/enclosure(s) and incoming
dtd: 8/24/79, 1/17/80, 4/17/80, 9/22/80

Chairman
Public Service Commission of Wisconsin
Hills Farms State Office Building
Madison, Wisconsin 53702

Mr. William Guldemond
USNRC Resident Inspectors Office
6612 Nuclear Road
Two Rivers, Wisconsin 54241

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 47
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated July 28, 1977, as modified August 24, 1979, January 17, April 17 and September 22, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-24 is hereby amended to read as follows:

   (B) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 47 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance. Final implementation shall be not later than 60 days following the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 31, 1981

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-306

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 52
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Electric Power Company (the licensee) dated July 28, 1977 as modified August 24, 1979, January 17, April 17 and September 22, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-27 is hereby amended to read as follows:

   (B) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 52, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance. Final implementation shall be no later than 60 days following the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 31, 1981

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 47 TO FACILITY OPERATING LICENSE NO. DPR-24

AMENDMENT NO. 52 TO FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NOS. 50-266 AND 50-301

Revise Appendix A as follows:

Remove Pages	Insert Pages
Table 15.3.5-1	Table 15.3.5-1
Table 15.3.5-3	Table 15.3.5-3
Table 15.4.1-1	Table 15.4.1-1
Table 15.4.1-1 (continued)	Table 15.4.1-1 (continued)
15.4.6-1	15.4.6-1

Point Beach Unit 1
Point Beach Unit 2

TABLE 15.3.5-1

ENGINEERED SAFETY FEATURES INITIATION INSTRUMENT SETTING LIMITS

NO.	FUNCTIONAL UNIT	CHANNEL	SETTING LIMIT
1	High Containment Pressure (Hi)	Safety Injection*	$\leq$ 6 psig
2	High Containment Pressure (Hi-Hi)	a. Containment Spray b. Steam Line Isolation of Both Lines	$\leq$ 30 psig $\leq$ 20 psig
3	Pressurizer Low Pressure	Safety Injection*	$\geq$ 1715 psig
4	Low Steam Line Pressure	Safety Injection* Lead Time Constant Lag Time Constant	$\geq$ 500 psig $\geq$ 12 seconds $\leq$ 2 seconds
5	High Steam Flow in a Steam Line Coincident with Safety Injection and Low T_{avg}	Steam Line Isolation of Affected Line	d/p corresponding to $\leq$ 0.66 x 10^6 lb/hr at 1005 psig $\geq$ 540°F
6	High-high Steam Flow in a Steam Line Coincident with Safety Injection	Steam Line Isolation of Affected Line	$\leq$ d/p corresponding to 4 x 10^6 lb/hr at 806 psig
7	Degraded Voltage (4.16KV)	Disconnection of affected bus from offsite power	$\geq$ 3675 volts ±2% Time delay: 13.6 sec. ±5% at 0-95% of voltage setting
8	Loss of Voltage		
	a. 4.16 KV	Disconnection of affected bus from offsite power start Diesel	a. 2450 volts ±3% Time delay: 0.3 sec. ±5% at 0 volts 1.2 sec. ±5% at 90% of voltage setting
	b. 480V	Load Shedding	b. 256 volts ±3% Time delay: 0.75 sec. ±5% at 0 volts 3.5 sec. ±5% at 90% of voltage setting

Amendment No. 33, 47
Amendment No. 43, 52

* Initiates also containment isolation, feedwater line isolation and starting of all containment fans.

d/p means differential pressure

Point Beach - Unit 1
Point Beach - Unit 2

TABLE 15.3.5-3

EMERGENCY COOLING

NO.	FUNCTIONAL UNIT	NO. OF CHANNELS	NO. OF CHANNELS TO TRIP	MIN. OPERABLE CHANNELS	MIN. DEGREE OF REDUNDANCY	PERMISSIBLE BYPASS CONDITIONS	OPERATOR ACTION IF CONDITIONS OF COLUMN 3 OR 4 CANNOT BE MET
1	SAFETY INJECTION						
a.	Manual	2	1	1	1		Hot Shutdown***
b.	High Containment Pressure	3	2	2	1		Hot Shutdown***
c.	Steam Generator Low Steam Pressure/Loop	3	2	2	1	Primary Pressure is Less than 1800 psig	Hot Shutdown***
d.	Pressurizer Low Pressure	3	2	2	1	Primary Pressure is Less than 1800 psig	Hot Shutdown***
2.	CONTAINMENT SPRAY						
a.	Manual	2	2	2	--**		Hot Shutdown***
b.	Hi-Hi Containment Pressure (Containment Spray)	2 sets of 3	2 of 3 in each set	2 per set	1/set		Hot Shutdown***
3.	SAFETY RELATED ELECTRICAL BUSES						
a.	Degraded Voltage (4.16KV)	3/bus	2/bus	2/bus	1/bus		****
b.	Loss of Voltage (4.16KV)	2/bus	1/bus	1/bus	1		****
c.	Loss of Voltage (480V)	3/bus	2/bus	2/bus	1		Hot Shutdown***

** - Must actuate 2 switches simultaneously.

*** - If minimum conditions are not met within 24 hours, steps shall be taken on the affected unit to place the unit in cold shutdown conditions.

**** - Normal operation provided both diesel generators are available, and the associated diesel generator is operating and providing power to the affected safeguards bus. If minimum conditions are not met within 7 days, affected unit shall be placed in hot shutdown.

Amendment No. 39, 47
Amendment No. 39, 52

Point Beach Unit 1
Point Beach Unit 2

TABLE 15.4.1-1

MINIMUM FREQUENCIES FOR CHECKS, CALIBRATIONS AND TEST OF INSTRUMENT CHANNELS

	Channel Description	Check	Calibrate	Test	Remarks
1.	Nuclear Power Range	S(1)** M*(3)**	D (1)** Q*(3)**	B/W (2)**	(1) Heat Balance (2) Signal to ΔT; bistable action (permissive, rod stop, trips) (3) Upper and lower chambers for axial off-set
2.	Nuclear Intermediate Range	S (1)**	N.A.	P (2)	(1) Once/shift when in service (2) Log level; bistable action (permissive, rod stop, trips)
3.	Nuclear Source Range	S (1)	N.A.	P (2)	(1) Once/shift when in service (2) Bistable action (alarm, trips)
4.	Reactor Coolant Temperature	S	R	B/W (1)** (2)	(1) Overtemperature-Delta T (2) Overpower - Delta T
5.	Reactor Coolant Flow	S**	R	M**	
6.	Pressurizer Water Level	S**	R	M**	
7.	Pressurizer Pressure	S**	R	M**	
8.	4 Kv Voltage	N.A.	R	M**	Reactor protection circuits only
9.	Analog Rod Position	S (1)**	R	M**	(1) With step counters

* By means of the movable in-core detector system.

** Not required during periods of refueling shutdown, but must be performed prior to starting up if it has not been performed during the previous surveillance period. Tests of permissive and low power trip bistable set-points which cannot be done during power operations shall be conducted prior to startup if not done in the previous two weeks.

Amendment No. 43, 47
Amendment No. 48, 52

TABLE 15.4.1-1 (Continued)

	Channel Description	Check	Calibrate	Test	Remarks
24.	Containment Pressure	S	R	M**	Narrow range containment pressure (-3.0, +3 psig excluded)
25.	Steam Generator Pressure	S***	R	M***	
26.	Turbine First Stage Pressure	S**	R	M**	
27.	Emergency Plan Radiation Instruments	M	R	M	
28.	Environmental Monitors	M	N.A.	N.A.	
29.	Overpressure Mitigating System	S	R	****	
30.	Degraded 4.16KV Voltage	S	R	M**	
31.a	Loss of Voltage (4.16KV)	S	R	M**	
b	Loss of Voltage (480V)	S	R	M**	
32.	4160 V. Frequency	N.A.	R	N.A.	

S - Each Shift
D - Daily
W - Weekly
B/W - Biweekly
M - Monthly
P - Prior to each startup if not done previous week
R - Each Refueling Shutdown (But not to exceed 20 months, except for first core cycle)
NA - Not applicable

** Not required during periods of refueling shutdown, but must be performed prior to starting up if it has not been performed during the previous surveillance period.

*** Not required during periods of refueling shutdown if steam generator vessel temperature is greater than $70^{\circ}F$.

**** Each PORV shall be demonstrated operable by:
a. Performance of a channel functional test on the PORV actuation channel, but excluding valve operation, within 31 days prior to entering a condition in which the PORV is required operable and at least once per 31 days thereafter when the PORV is required operable.
b. Testing valve operation in accordance with the inservice test requirements of the ASME Boiler and Pressure Vessel Code, Section IX.

Amendment No. 38, 47
Amendment No. 50, 52

15.4.6 EMERGENCY POWER SYSTEM PERIODIC TESTS

Applicability

Applies to periodic testing and surveillance requirements of the emergency power system.

Objective

To verify that the emergency power system will respond promptly and properly when required.

Specification

The following tests and surveillance shall be performed as stated:

A. Diesel Generators

1. Manually-initiated start of the diesel generator, followed by manual synchronization with other power sources and assumption of load by the diesel generator shall not exceed 2850 KW. This test will be conducted monthly with a minimum running time of 30 minutes on each diesel generator. Normal plant operation will not be affected.

2. Automatic start of each diesel generator, load shedding, and restoration to operation of particular vital equipment, initiated by an actual interruption of normal A-C station service power supplies to associated engineered safety systems busses together with a simulated safety injection signal. In addition, after the diesel generator has carried its load for a minimum of 5 minutes, automatic load shedding and restoration of vital loads are tested again by manually tripping the diesel generator output breaker. This test will be conducted during reactor shutdown for major fuel reloading of each reactor to assure that the diesel-generator will start and assume required load in less than the time periods listed in the FFDSAR Section 8.2 after the initial starting signal. During this test a checkout of emergency lighting will be performed, including the changeover relay for DC lights.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 47 TO FACILITY OPERATING LICENSE NO. DPR-24

AND AMENDMENT NO. 52 TO FACILITY OPERATING LICENSE NO. DPR-27

WISCONSIN ELECTRIC POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-266 AND 50-301

Introduction

The criteria and staff positions pertaining to degraded grid voltage protection were transmitted to Wisconsin Electric Power Company (WEPCO) by NRC generic letter dated June 2, 1977. In response to this, by letters dated July 28, 1977, August 24, 1979, January 17, 1980, April 17, 1980 and September 22, 1980, the licensee proposed certain design modifications and changes to the Technical Specifications.

Discussion and Evaluation

The modifications consist of installing three undervoltage relays on each of the two Class 1E 4160-volt buses for each unit. These relays are arranged in a two-out-of-three coincident logic. These relays will monitor the voltage and initiate disconnection of the buses from offsite source if the voltage drops below 88% of nominal bus voltage for more than 13.6 seconds.

Load shedding is being maintained once the diesel generators are supplying power to their respective buses. The licensee's basis for this is that the setpoint initiating the load shed (256 volts + 3%) is low enough (53% of nominal voltage) to eliminate the need to block the load-shed feature.

The licensee has also requested changes in the Technical Specification surveillance testing requirements that will "simulate loss of offsite power in conjunction with a safety-injection actuation signal and will simulate interruption and subsequent reconnection of onsite power sources".

We have reviewed the licensee's proposed Technical Specification changes and find that:

The proposed Technical Specification changes to document relay setpoints, surveillance requirements and limiting conditions for operation are acceptable. The proposed modifications will protect the Class 1E equipment and systems from a sustained degraded voltage of the offsite power source, and the proposed changes to the Technical Specifications meet the staff's requirements. Therefore, we conclude that WEPCO's proposed design modification and changes to the Technical Specifications are acceptable.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 31, 1981

7590-01

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-266 AND 50-301

WISCONSIN ELECTRIC POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 47 to Facility Operating License No. DPR-24, and Amendment No. 52 to Facility Operating License No. DPR-27 issued to Wisconsin Electric Power Company (the licensee), which revised Technical Specifications for operation of Point Beach Nuclear Plant, Unit Nos. 1 and 2 (the facilities) located in the Town of Two Creeks, Manitowoc County, Wisconsin. The amendments are effective as of the date of issuance.

The amendments reflect modifications to plant equipment to provide additional assurance of the proper functioning of the plant's safety related electrical and emergency power systems.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated July 28, 1977 as modified August 24, 1979, January 17, April 17 and September 22, 1980, (2) Amendment Nos. 47 and 52 to License Nos. DPR-24 and DPR-27, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. 20555, and at the Joseph Mann Library, 1516 16th Street, Two Rivers, Wisconsin 54241. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 31st day of March, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing