

Docket

APR 7 1977

Dockets Nos. 50-266
and 50-301

Wisconsin Electric Power Company
Wisconsin Michigan Power Company
ATTN: Mr. Sol Burstein
Executive Vice President
231 West Michigan Street
Milwaukee, Wisconsin 53201

Gentlemen:

The Commission has issued the enclosed Amendment No. 29 to Facility Operating License No. DPR-27 for the Point Beach Nuclear Plant Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application dated December 9, 1976, as supplemented by letters dated February 1, March 18, and April 1, 1977.

The amendment consists of changes in the Technical Specifications that will allow operation of Unit No. 2 in core Cycle 4 by eliminating the fuel residence time limit, and modifying the core power distribution limits.

In addition, the Commission has approved the reevaluation of ECCS cooling performance for both Point Beach Units Nos. 1 and 2 that you submitted on October 27, 1976, and supplemental letter dated January 6, 1977, in response to the Commission's Order for Modification of License dated August 27, 1976.

Copies of the Safety Evaluation and the Federal Register Notice are also enclosed.

We would like to draw your attention to one additional point. In your letter of February 1, 1977, you stated that you would not submit a report of startup test results per our request of January 19, 1977. The requested information is needed to enable us to verify the accuracy of the fuel supplier's computer codes and to identify potential problems that might not otherwise be apparent from other information we receive.

OFFICE >						
SURNAME >						
DATE >						

APR 7 1977

Wisconsin Electric Power Company - 2 -

Consequently, we believe that your position not to supply this information is, in the long run, contrary to the best interest of all concerned with safe operation of the facility. We therefore urge you to reconsider your position, and we reiterate our previous request that you submit the startup test results for Point Beach Unit No. 2, core Cycle 4.

Sincerely,

Original signed by

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 29
- 2. Safety Evaluation
- 3. Federal Register Notice

cc: See next page

*SEE PREVIOUS YELLOW FOR CONCURRENCES

OFFICE >	ORB #3	ORB #3	OELD	ORB #3	AD:DOR
SURNAME >	*CParrish	*JWetmore:acr	*	*GLear	*KRGoller
DATE >	4/1/77	4/1/77	4/1/77	4/7/77	4/6/77

position, and we reiterate our previous request that you submit the startup test results for Point Beach Unit No. 2, core Cycle 4.

Sincerely,

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Operating Reactors Branch #3
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OFFICE	ORB #3	ORB #3	OELD	ORB #3	AD:DOR
SURNAME	*CParrish	*JWetmore:acr		GLear 6	KRGoller
DATE	4/1/77	4/1/77	4/1/77	4/7/77	4/6/77

Regulatory Guide on the submittal of core reload information. It is already included in a "Branch Technical Position" now being reviewed for publication. Consequently, we believe that your position not to supply this information is, in the long run, contrary to the best interest of all concerned with safe operation of the facility. We therefore urge you to reconsider your position, and we reiterate our previous request that you submit the startup test results for Point Beach Unit No. 2, core Cycle 4.

Sincerely,

George Lear, Chief
 Operating Reactors Branch #3
 Division of Operating Reactors

Enclosures:

1. Amendment No. 29
2. Safety Evaluation
3. Federal Register Notice

cc: See next page

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*with charge indicated on p.1
 wetmore concurred 4/11/77*

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DATE →	<i>4/1</i> / 177	<i>4/1</i> / 177	<i>3/1</i> / 177	3/ / 177	<i>4/6</i> / 77

Wisconsin Electric Power Company - 2 -
Wisconsin Michigan Power Company

cc:

Mr. Bruce Churchill, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N. W.
Washington, D. C. 20036

Mr. Norman Clapp, Chairman
Public Service Commission
of Wisconsin
Hill Farms State Office Building
Madison, Wisconsin 53702

Mr. Arthur M. Fish
Document Department
University of Wisconsin -
Stevens Point Library
Stevens Point, Wisconsin 54481

Wisconsin Electric Power Company
ATTN: Mr. Glen Reed
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Chief, Energy Systems Analysis Branch (AW-459)
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U. S. Environmental Protection Agency
Room 645, East Tower
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Washington, D. C. 20460

U. S. Environmental Protection Agency
Federal Activities Branch
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Chicago, Illinois 60604

Walter L. Meyer
Town Chairman
Town of Two Creeks, Wisconsin
Route 3, Two Rivers, Wisconsin 54241



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY
WISCONSIN MICHIGAN POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 29
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company and Wisconsin Michigan Power Company (the licensees) dated December 9, 1976, as supplemented by letter dated February 1, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-27 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 29, are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Karl R. Goller

Karl R. Goller, Assistant Director
for Operating Reactors
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 7, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 29

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NO. 50-301

Replace pages 15.2.1-1, 15.2.1-3, 15.3.10-2 and Figure 15.3.10-3 with the attached revised pages.

15.2.0 SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

15.2.1 SAFETY LIMIT, REACTOR CORE

Applicability:

Applies to the limiting combinations of thermal power, reactor coolant system pressure, and coolant temperature during operation.

Objective:

To maintain the integrity of the fuel cladding.

Specification:

1. The combination of thermal power level, coolant pressure, and coolant temperature shall not exceed the limits shown in Figure 15.2.1-1. The safety limit is exceeded if the point defined by the combination of reactor coolant system average temperature and power level is at any time above the appropriate pressure line.

Additional peaking factors to account for local peaking due to fuel rod axial gaps and reduction in fuel pellet stack length as well as a penalty to account for rod bowing, have been included in the calculation of the curves shown in Figure 15.2.1-1. These curves are based on an $F_{\Delta N}^N$ of 1.58, cosine axial flux shape, and a DNB analysis as described in Section 4.3 of WCAP-8050, "Fuel Densification, Point Beach Nuclear Plant Unit 1 Cycle 2", (including the effects of fuel densification and flattened cladding).

Figure 15.2.1-1 also includes an allowance for an increase in the enthalpy rise hot channel factor at reduced power based on the expression:

$$F_{\Delta H}^N = 1.58 [1 + 0.2 (1-P)] \text{ where } P \text{ is a fraction of rated power}$$

$$\text{when } P \leq 1.0. \quad F_{\Delta H}^N = 1.58 \text{ when } P > 1.0.$$

An additional rod bow penalty is applied for the Point Beach cores to limit the radial peaking factor $F_{\Delta H}$ to a more conservative value of 1.55 instead of 1.58. This additional penalty is based on new data (plus appropriate conservatism) which shows that the bowing model in WCAP-8386, "An Evaluation of Fuel Rod Bowing" underestimates the extent of fuel rod bowing.

The hot channel factors are also sufficiently large to account for the degree of malpositioning of full-length rods that is allowed before the reactor trip setpoints are reduced and rod withdrawal block and load runback may be required. Rod withdrawal block and load runback occur before reactor trip setpoints are reached. The Reactor Control and Protective System is designed to prevent any anticipated combination of transient conditions that would result in a DNB ratio of less than 1.30.

The fuel residence time during any given Cycle is limited to less than that at which clad flattening will occur to assure no clad flattening without prior review by the Regulatory Staff. The residence time is based on predicted minimum time to clad flattening for the appropriate cycle operating pressure. The basis for the calculation of clad flattening, time is given in WCAP 8377, "Revised Clad Flattening Model".

3. The part-length rods shall be fully withdrawn from the core, except for physics testing.
4. When the reactor is subcritical, except for physics tests, the critical rod position, i.e., the rod position at which criticality would be achieved if the control rods were withdrawn in normal sequence with no other reactivity changes, shall not be lower than the insertion limit for zero power.

B. Power Distribution Limits

1. a. Except during low power physics tests, the hot channel factors defined in the basis must meet the following limits:

$$F_Q(Z) \leq \frac{(2.32)}{P} \times K(Z) \quad \text{for } P > .5$$

$$F_Q(Z) \leq 4.64 \times K(Z) \quad \text{for } P \leq .5$$

$$F_{\Delta H}^N \leq 1.55 \times (1 + 0.2 (1-P))$$

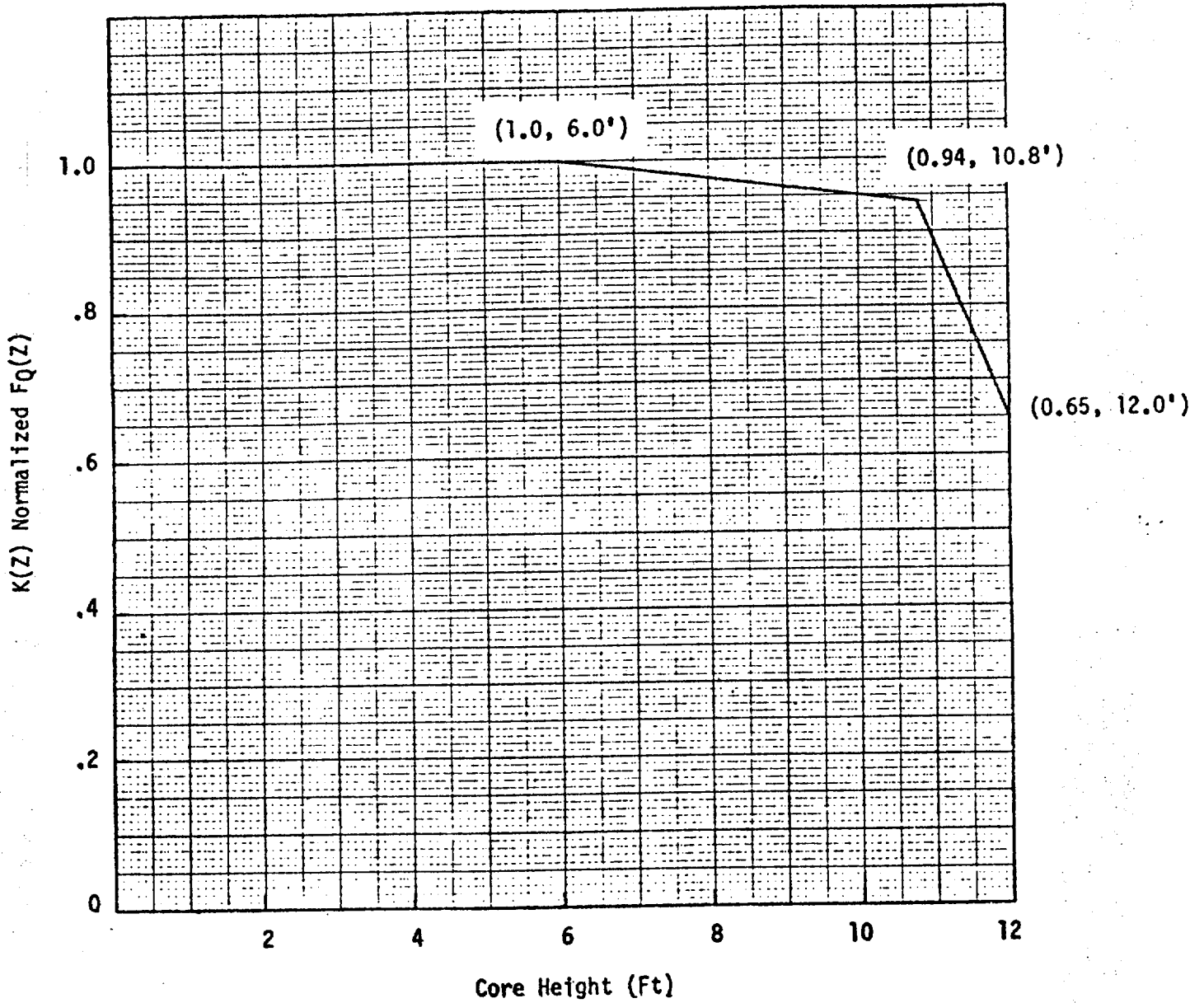
where P is the fraction of full power at which the core is operating, K(Z) is the function in Figure 15.3.10-3 and Z is the core height location of F_Q .

- b. Following core loading prior to exceeding 90% of rated power and at effective full power monthly intervals thereafter, power distribution maps using the movable incore detector system shall be made to confirm that the hot channel factor limits are satisfied. The measured hot channel factors shall be increased in the following way:

- (1) The measurement of total peaking factor, F_Q^{Meas} , shall be increased by three percent to account for manufacturing tolerances and further increased by five percent to account for measurement error.

FIGURE 15.3.10-3

POINT BEACH UNIT 1 AND UNIT 2
HOT CHANNEL FACTOR NORMALIZED OPERATING ENVELOPE





UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 29 TO LICENSE DPR-27

AND

SUPPORTING APPROVAL OF A REEVALUATION OF
ECCS COOLING PERFORMANCE FOR LICENSES DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY

WISCONSIN MICHIGAN POWER COMPANY

POINT BEACH NUCLEAR PLANT UNITS NOS. 1 AND 2

DOCKETS NOS. 50-266 AND 50-301

Introduction

By letter dated December 9, 1976, Wisconsin Electric Power Company (WEPCO) proposed changes to the Technical Specifications of Facility Operating License DPR-27 for Point Beach Unit No. 2. WEPCO supplied supplemental information to support the requested changes by letters dated February 1, March 18, and April 1, 1977. The proposed changes would allow operation of Unit No. 2 in core Cycle 4 by eliminating the fuel residence time limit, and modifying the core power distribution limits.

In addition, WEPCO submitted a reevaluation of ECCS cooling performance for both Point Beach Units Nos. 1 and 2 by letter dated October 27, 1976. WEPCO provided supplemental information by letter dated January 6, 1977. The reevaluation of ECCS cooling performance was submitted in compliance with the Commission's Order for Modification of License dated August 27, 1976. A similar Order was sent to the licensees of other Westinghouse designed plants. The Orders were issued after it was reported to the NRC that reactor vessel upper head water temperatures in excess of those assumed in previously approved ECCS analyses could exist in Westinghouse designed reactors. This higher upper head water temperature has the effect of increasing the calculated peak clad temperature in the event of a loss-of-coolant-accident (LOCA). The reevaluation of ECCS cooling performance for Point Beach Units Nos. 1 and 2 has shown that the total nuclear peaking factor limitations presently incorporated in the Technical Specifications for the facility are adequate to ensure that the ECCS acceptance criteria of 10 CFR 50.46 are met.

Our evaluation of the proposed changes to the Point Beach Unit No. 2 Technical Specifications to allow operation in core Cycle 4, and our evaluation of the licensee's calculated ECCS cooling performance follows.

Evaluation

A. Point Beach Unit No. 2 core cycle 4:

Core Cycle 4 fuel loading will consist of 1 Region 2 assembly, 48 Region 4 assemblies, 36 Region 5 assemblies, plus 36 new unirradiated Region 6 assemblies. The mechanical, thermal-hydraulic and chemical design of the new Region 6 assemblies is essentially the same as the other irradiated fuel assemblies that will remain in the core during Cycle 4.

Most of the core parameters determined for Cycle 4 fall within the range of values used in previously approved accident analyses and therefore most of the existing safety analyses for Cycle 3 continue to apply to Cycle 4. The only exception to this is the change to control rod worths and peaking factors which affect the results of the rod ejection accident analyses. Consequently, the licensee has reanalyzed the rod ejection accidents using a standard Westinghouse procedure (reference 4). The analysis was performed for beginning and end of cycle conditions, and assumed a conservatively high initial fuel average temperature. The results of the analysis indicate no fuel melting and an acceptable value of peak fuel enthalpy. Based on these results, we have concluded that the rod ejection accident analysis for core Cycle 4 is acceptable.

The Cycle 4 planned physics startup tests for Point Beach, Unit No. 2 were reviewed to check that: (1) all necessary tests would be performed, and (2) the acceptance criteria are reasonable. The startup tests will check the fuel loading and verify the calculational methods used to determine power distributions, shutdown margin and control rod worths. Core flux maps at various power levels will be taken and evaluated to verify power distribution predictions. This data will also be used in establishing the excore/incore calibration. The test proposed to verify shutdown margin and control rod worths consists of determining the differential and integral rod worths for control banks D and C. Based on our review, it is our position that the physics startup test program is acceptable only if the integral rod worths of control banks A and B are also determined and if the following conditions are met: If the worth of Bank D or C differs from the predicted value by more than 15%, or the sum of the worths of the control banks A, B, C and D differs from the predicted value by more than 10%, the first shutdown bank should be

measured. If the sum of the worths of banks A, B, C and D and the first shutdown bank differs from the predicted value by more than 10%, additional shutdown bank measurements should be performed to verify technical specification shutdown margin. Also, the power coefficient must be measured, with at least one measurement at a high power level (over 65% power). The licensee has agreed to these requirements.

The licensee has proposed changes to the Technical Specifications to allow reactor operation in core Cycle 4:

1. Fuel Residence Time Limit (Technical Specification 15.2.1.2)

The existing fuel residence time limit contained in Technical Specification 15.2.1.2 applies to core Cycle 3 and is based on the predicted time to clad flattening for the most limiting fuel in core Cycle 3. The licensee has proposed eliminating this limit for Cycle 4. The predicted time to clad flattening has been determined for the most limiting fuel in Cycle 4 using an approved Westinghouse procedure (reference 3). The results show that clad flattening will not occur for core Cycle 4, thus a fuel residence time limit is no longer necessary or required. Therefore, the proposed change to eliminate the fuel residence time limit is acceptable.

2. Core Power Distribution Limits (Technical Specification 15.3.10.B.1.a, and Figure 15.3.10-3)

The existing height dependent heat flux hot channel factor (FQ[Z]) apply to core Cycle 3 only. They were originally imposed because of clad flattening considerations in core Cycle 3. Clad flattening is not calculated to occur in core Cycle 4, so the licensee has proposed a relaxation of FQ[Z] limits back to the pre-Cycle 3 values. Based on our review of the ECCS analysis we have determined that the proposed change is acceptable.

The existing Hot Channel Factor Normalized Operating Envelope (Figure 15.3.10-3) applies to core Cycle 3. For core Cycle 4 the licensee has proposed a modification to the 10.8 to 12 foot core elevation line segment that results in a lower limit curve. The remainder of the envelope remains unchanged. This proposed change will provide an additional margin of safety; and thus, is acceptable.

B. Reevaluation of ECCS Cooling Performance for both Point Beach Units Nos. 1 and 2:

The Loss-of-coolant-accident has been reanalyzed^(5,8) with the October 1975 version ⁽⁶⁾ of the Westinghouse ECCS evaluation model to account for elevated reactor vessel upper head water temperature. The reanalysis was performed assuming:

1. The coolant temperature in the upper head region is equal to the reactor coolant system hot leg temperature;
2. The break spectrum in the generic evaluation ⁽⁷⁾ is conservative for Point Beach;
3. The double ended cold leg (DECL) break with $C_D=0.4$ is the worst case; and
4. 10% of the steam generator tubes are plugged.

Information supplied by the licensee⁽⁸⁾ verifies the conservatism of assumptions (2) and (3). The plant configuration assumed in the generic evaluation ⁽⁷⁾ is based on a reactor power higher than any operating two-loop plant, a core flooding rate lower than any operating two-loop plant, and a containment pressure during the transient that is lower than any operating two-loop plant. The result of these assumptions is a calculated peak clad temperature higher than would be calculated for an individual two-loop plant. Thus, the generic evaluation is conservative for Point Beach. Assumption (4) is conservative for Point Beach Unit No. 2, since the tube plugging level is now approximately 0.4%. It is also conservative for Point Beach Unit No. 1 because the tube plugging level is now approximately 5.5%.

The results of the reanalysis with the above assumptions and a total peaking factor (FQ) of 2.32 indicate a peak clad temperature of 1965°F, hot spot metal-water reaction of 3.5%, and total core metal-water reaction less than 0.3%. All of the above results are within the acceptance criteria of 10 CFR 50.46. Therefore, the reanalysis of the ECCS performance for Point Beach Units Nos. 1 and 2 is acceptable.

Environmental Finding

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that:

- (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
- (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 7, 1977

REFERENCES

1. Letter S. Burstein (WEPCo) to B. Rusche (NRC) dated December 9, 1976.
2. Letter S. Burstein (WEPCo) to B. Rusche (NRC) dated February 1, 1977.
3. George, R. A. ., et. al., "Revised Clad Flattening Model," WCAP-8377 (Proprietary) and WCAP-8381 (Non-Proprietary), July 1976.
4. Risher, D. H. Jr., "An evaluation of the Rod Ejection Accident in Westinghouse Pressurized Water Reactor Using Spatial Kinetics Method," WCAP-7588, Revisional, December 1971.
5. Letter S. Burstein (WEPCo) to B. Rusche (NRC) dated October 27, 1976.
6. "Westinghouse ECCS Evaluation Model October 1975 Version," WCAP-8622 (Proprietary), WCAP-8623 (Non-Proprietary), November 1975.
7. Del Signore, T., et. al., "Westinghouse ECCS Two Loop Plant Sensitivity Studies (14x14)" WCAP-8854 (Non-Proprietary), September 1976.
8. Letter S. Burstein (WEPCo) to B. Rusche (NRC) dated January 6, 1977.

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-266 AND 50-301

WISCONSIN ELECTRIC POWER COMPANY
WISCONSIN MICHIGAN POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE NO. DRR-27

AND

APPROVAL OF A REEVALUATION OF ECCS COOLING PERFORMANCE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 29 to Facility Operating License No. DPR-27 issued to Wisconsin Electric Power Company and Wisconsin Michigan Power Company which revised Technical Specifications for operation of the Point Beach Nuclear Plant Unit No. 1, located in the Town of Two Creeks, Manitowac County, Wisconsin. The amendment is effective as of its date of issuance.

The amendment consists of changes in the Technical Specifications that will allow operation of Unit No. 2 in core Cycle 4 by eliminating the fuel residence time limit and modifying the core power distribution limits.

In addition, the Commission has approved the reevaluation of ECCS cooling performance for both Point Beach Units Nos. 1 and 2 that was submitted by the licensees on October 27, 1976 and supplemented by letter dated January 6, 1977 in response to the Commissions Order for Modification of License dated August 27, 1976.

The application for the amendment to Point Beach Unit No. 2 (Docket No. 50-301) and the licensees' October 27, 1976 submittal of reevaluation of ECCS cooling performance for Point Beach Units Nos. 1 and 2 (Dockets Nos. 50-266 and 50-301) comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Notice of the Proposed Issuance of Amendment to Facility Operating License No. DPR-27 in connection with this action was published in the FEDERAL REGISTER on January 10, 1977 (42FR2141). Notice of Order for Modification of Licenses (Dockets Nos. 50-266 and 50-301) was published in the FEDERAL REGISTER on September 9, 1976 (41FR38236). No request for a hearing or petition for leave to intervene was filed following notice of the proposed action or notice of the Order for Modifications for Licenses.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement and negative declaration or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated December 9, 1976, as supplemented by letters dated February 1, March 18, and April 1, 1977, (2) Amendment No. 29 to License No. DPR-27, (3) the licensees' October 27, 1976 response to the Commission's

Order for Modification of License dated August 27, 1976, and supplemental information dated January 6, 1977, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the Document Department - University of Wisconsin, Stevens Point Library, ATTN: Mr. Arthur M. Fish, Stevens Point, Wisconsin 54481.

A copy of items (2) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 7th day of April 1977.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script that reads "George Lear". The signature is written in black ink and is positioned above the typed name and title.

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

MAR 2 1977

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Dockets Nos. 50-266
and 50-301

Wisconsin Electric Power Company
Wisconsin Michigan Power Company
ATTN: Mr. Sol Burstein
Executive Vice President
231 West Michigan Street
Milwaukee, Wisconsin 53201

Gentlemen:

On September 23, 1976, we issued Amendment Nos. 20 and 25 to License Nos. DPR-24 and DPR-27 for Point Beach Units Nos. 1 and 2. Some of the copies of these amendments were minus Figure 15.4.10-1 which was to be added to the Technical Specifications for the facilities. Enclosed are copies of Figure 15.4.10-1 in case your copies of the amendments were minus this figure.

Sincerely,

Original signed by

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosure:
Figure 15.4.10-1 (2 copies)

cc: See page 2

OFFICE >	ORB #2 <i>JW</i>	ORB #3				
SURNAME >	JWetmore:jf	GLear <i>G</i>				
DATE >	3/1/77	3/2/77				

Wisconsin Electric Power Company
Wisconsin Michigan Power Company

- 2 -

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Wisconsin Electric Power Company
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Plant Superintendent
Point Beach Plant
231 West Michigan Street
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