August 2, 1976

Dockets Nos. 50-266 and 50-301

> Wisconsin Electric Power Company Wisconsin Michigan Power Company ATTN: Mr. Sol Burstein Executive Vice President 231 West Michigan Street Milwaukee, Wisconsin 53201

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Gentlemen:

The Commission has issued the enclosed Amendments Nos. 19 and 24 to Facility Operating Licenses Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Units Nos. 1 and 2. The amendments consist of changes to the Technical Specifications and are in accordance with your applications dated June 1 and 29, 1976.

These amendments consist of changes in the Technical Specifications that involve changes to the administrative controls section, excluding any substantive changes to the Plant Reports Requirements, and were submitted in response to our request of October 18, 1974. Changes to the Plant Reporting Requirements, Technical Specification 15.6.9, were issued in a separate licensing action on January 9, 1976.

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

Stello^{*}s office checked, per Pat OK - 8/3/76

			George Le Operating Division	ear, Chief Reactors Bra of Operating	nch #3 Reactors)	
Enclo	sures:		-	A Start			
1. A	mendment No.	19 to License	DPR-24	a gast			
2. A	2. Amendment No. 24 to License DPR-27						
3. S	afety Evaluat:	10 0					
4. F	ederal Regist	er Notice	~ ~ ~ ~ ~				
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Form AEC-318 (Rev. 9-53) AECM 0240

A U. S. GOVERNMENT PRINTING OFFICE: 1974-526-166

. Wisconsin Electric Power ompany - 2 Wisconsin Michigan Power Company

Mr. Bruce Churchill, Esquire Shaw, Pittman, Potts and Trowbridge 1800 M. Street, N. W. Washington, D. C. 20036

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Mr. Norman Clap, Chairman Public Service Commission of Wisconsin Hill Farms State Office Building Madison, Wisconsin 53702

Mr. Arthur M. Fish Document Department University of Wisconsin -Stevens Point Library Stevens Point, Wisconsin 54481

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WISCONSIN ELECTRIC FONER COMPANY WISCONSIN MICHIGAN PONER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

ABSECTATE OF PACEDONY OF TRADENCE I CERTON

Amendment No. 19 License No. DPR-24

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Wisconsin Electric Power Company and Wisconsin Michigan Power Company (the licensees) dated June 1 and 24, 1976, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amondment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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George toar, Chief Operating Resetors Drovch #3 Division of Operating Resources

Attachment: Changes to the Technical Specifications

Date of Issuance: August 2, 1976

- 2 -

ATTACHMENT TO LICENSE AMENDMENT NO. 19

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TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-24

DOCKET NO. 50-266

Replace in its entirety Section 15.6 - Administrative Controls (consisting of pages 15.6.1-1 through 15.6.6-3) with the attached revised pages.

15.6 AD

ADMINISTRATIVE ONTROLS

15.6.1 RESPONSIBILITY

15.6.1.1 The Manager - Nuclear Power Division shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during absences from the Point Beach Nuclear Plant area of greater than 48 hours and where ready contact by telephone or other means is not assured.

15.6.2 ORGANIZATION

OFFSITE

15.6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 15.6.2-1.

FACILITY STAFF

15.6.2.2 The Facility organization shall be as shown on Figure 15.6.2-2 and:

- a. Each on-duty shift shall normally be composed of at least the minimum shift crew composition as noted in Figure 15.6.2-2.
- b. At least one licensed Operator shall be in the control room when fuel is in either reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures ['] shall be on site when fuel is in either reactor.
- e. ALL CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

15.6.1/2-1



MANAGEMENT ORGANIZATION CHART

15.6.2-1



Control Operator (RO) One per shift - one unit two per shift - two units Auxiliary Operator two per shift - one unit three per shift - two units

2. Health Physicist has optional reporting route to Manager on radiological health and safety matters.

minimum requirements.

3. SRO - AEC Senior Reactor Operator License

Group shift makeup may be less than the minimum

taken to restore the shift makeup to within the

requirements for a period of time not to exceed 2

hours in order to accommodate unexpected absence of on

duty shift crew members, provided immediate action is

RO - AEC Reactor Operator License

CONDUCT OF PLANT OPERATIONS CHART Figure 15.6.2-2

15.6.3 FACILITY STAFF QUALIFICATIONS

15.6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions.

15.6.4 TRAINING

15.6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

15.6.5 REVIEW AND AUDIT

15.6.5.1 Duty and Call Superintendents

- a. To assist and counsel the Shift Supervisor in case of Significant Operating Events, a Duty and Call Superintendent Group has been established. The Duty and Call Superintendent Group shall consist of any qualified person designated by the Manager - Nuclear Power Division.
- b. In the event of a reportable occurrence, the Shift Supervisor shall communicate with at least one Duty and Call Superintendent before taking other than the immediate onthe-spot action required. One Duty and Call Superintendent will be assigned to be "on call" at all times. The Duty and Call Superintendent provides continuously available counsel, call out backups, and review to the Shift Supervisor.

15.6.5.2 Manager's Supervisory Staff

FUNCTION

15.6.5.2.1 The Manager's Supervisory Staff (MSS) shall function to advise the Manager - Nuclear Power Division on all matters related to nuclear safety.

15.6.3/4/5-1

COMPOSITION,

15.6.5.2.2 The Manager's Supervisory Staff shall be composed of the:

Chairman:	Manager - Nuclear Power Division						
Member:	Operations Superintendent						
Member:	Maintenance Superintendent						
Member:	Instrument and Control Engineer						
Member:	Reactor Engineer						
Member:	Radiochemical Engineer						
Member:	Health Physicist						
Member:	Assistant to the Manager - Nuclear Power Division						

ALTERNATES

15.6.5.2.3 Alternate members shall be appointed in writing by the MSS Chairman to serve on a temporary basis; however, no more than two alternates shall participate in MSS activities at any one time.

MEETING FREQUENCY

15.6.5.2.4 The MSS shall meet at least once per calendar month and as convened by the MSS Chairman.

QUORUM

15.6.5.2.5 A quorum of the MSS shall consist of the Chairman and four members including alternates.

RESPONSIBILITIES

15.6.5.2.6 The Manager's Supervisory Staff shall:

a) Review existing and proposed normal, abnormal and emergency operating procedures. Review maintenance procedures and proposed changes to these procedures and other procedures or changes thereto as determined by the Manager to affect plant operational safety. (Re: Section 15.6.7 for area of review.)

- b) Review all proposed tests and experiments related to Safety and the results thereof when applicable.
- c) Review all proposed changes to Technical Specifications.
- Review all proposed changes or modifications to plant systems or equipment where changes would require a change in operating or emergency procedures or that affect nuclear safety.
- Periodically review plant operations for industrial and nuclear safety hazards.
- f) Investigate violations or suspected violations of Technical Specifications, such investigations to include reports, evaluations, and recommendations to prevent recurrence, to the Vice President - Nuclear Plant and to the Chairman of the Off-Site Review Committee.
- g) Perform special reviews and investigations and prepare reports thereon as requested by the Chairman of the Off-Site Review Committee.
- h) Investigate, review, and report on all reportable occurrences.
- Cause to be conducted periodic drills on emergency procedures, including evacuation (partial or complete) of the site and check adequacy of communications with off-site support groups.

AUTHORITY

- 15.6.5.2.7
- 2.7 a) The Supervisory Staff shall serve as advisory to the Manager -Nuclear Power Division.
 - b) The Supervisory Staff shall recommend to the Manager approval or disapproval of proposals under items a) through d) above.
 In the event of disagreement between a majority of the

15.6.5-3

Supervisory Staff and decisions by the Manager, the course of action will be determined by the Manager and the disagreement recorded in the Staff minutes. Records of the disagreement will be included in the minutes sent for review to the Off-Site Review Committee and the Vice President - Nuclear Plant.

- c) The Supervisory Staff shall make tentative recommendations as to whether or not proposals considered by the Staff involve unreviewed safety questions. These recommendations shall be subject to review and further recommendations by the Off-Site Review Committee. Minutes shall be kept of all meetings of the Staff and copies shall be sent to the Vice President -Nuclear Plant and to the Chairman of the Off-Site Review Committee.
- d) The Supervisory Staff shall review and approve the contents
 of a report for each reportable occurrence.
 This report shall include an evaluation of the cause of the
 occurrence and recommendations for appropriate action to
 prevent or reduce the probability of a recurrence. Copies
 of all such reports shall be submitted to the Vice President Nuclear Plant and to the Chairman of the Off-Site Review
 Committee.

RECORDS

15.6.5.2.8 The Manager's Supervisory Staff shall maintain written minutes of each meeting and copies shall be provided to the Vice President - Nuclear Plant and Chairman, Off-Site Review Committee. 15.6.5.3 OFF-SITE REVIEW COMMITTEE (OSRC)

FUNCTION

15.6.5.3.1 The Off-Site Review Committee shall function to provide independent review and audit of designated activities in

the areas of:

- a) nuclear power plant operations
- b) nuclear engineering
- c) chemistry and radiochemistry
- d) metallurgy
- e) instrumentation and control
- f) radiological safety
- g) mechanical and electrical engineering
- h) guality assurance practices
- i) environmental monitoring

COMPOSITION

15.6.5.3.2 The Off-Site Review Committee is made up of a minimum of five regular members appointed by the Vice President - Nuclear Plant and one ex-officio member. Of the five or more regular members, at least two will be persons not directly employed by WEPCo. or WMPCo. All members will be experienced in one or more aspects of the nuclear industry. The ex-officio member will be the Vice President - Nuclear Plant of Wisconsin Michigan Power Company.

ALTERNATES

15.6.5.3.3 Alternate members may be appointed in writing by the OSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in OSRC activities at any one time.

CONSULTANTS

15.6.5.3.4 Consultants shall be utilized as determined by the OSRC Chairman to provide expert advice to the OSRC.

MEETING FREQUENCY

15.6.5.3.5 The OSRC shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least twice per year at approximately six month intervals thereafter. QUORUM

15.6.5.3.6 A quorum of OSRC shall consist of the Chairman or his designated alternate and three members. No more than a minority of the quorum shall have line responsibility for operation of the facility.

REVIEW

15.6.5.3.7 The OSRC shall review:

- a) The safety evaluations for 1) changes to procedures, equipment or systems, and 2) tests or experiments completed under the provision of 10 CFR, Section 50.59, to verify that such actions did not constitute an unreviewed safety question.
- b) Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in 10
 CFR, Section 50.59.
- c) Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR Section 50.59.
- d) Proposed changes in Technical Specifications or Licenses.
- e) Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f) Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g) Reportable Occurrences requiring 24 hour notification to the Commission.

Amendment No. 19

15.6.5-6

- h) Any in cation of an unanticipated deficency in some aspect of design or operation of safety related structures, systems, or components.
- Reports and meeting minutes of the Manager's Supervisory Staff.

AUDITS

15.6.5.3.8 Audits of facility activities shall be performed under the cognizance of the OSRC. These audits shall encompass:

- a) The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per year.
- b) The performance, training and qualifications of the licensed operating staff at least once per year.
- c) The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least twice per year at approximately six month intervals.
- d) The results of quarterly audits by the Quality Assurance and Technical Services Department on the performance of activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per two years.
- Emergency Plan and implementing procedures at least once per 2 years.
- f) Any other area of facility operation considered appropriateby the Vice President Nuclear Plant.

AUTHORITY

15.6.5.3.9 The OSRC shall report to and advise the Vice President - Nuclear Plant on those areas of responsibility specified in Section 15.6.5.3.7 and 15.7.6.3.8.

15.6.5-7

RECORDS

15.6.5.3.10 Records of OSRC activities shall be prepared, approved and distributed as indicated below:

- a) Minutes of each OSRC meeting shall be prepared, approved and forwarded to the Vice President - Nuclear Plant within 14 days following each meeting.
- b) Reports of reviews encompassed by Section 15.6.5.3.7.e, f and g above, shall be prepared, approved and forwarded to the Vice President - Nuclear Plant within 14 days following completion of the review.
- c) Audit reports encompassed by Section 15.6.5.3.8 above, shall be forwarded to the Vice President - Nuclear Plant and to the management positions responsible for the areas audited within 30 days after completion of the audit.

15.6.6 REPORTABLE OCCURRENCE ACTION

Specification

The following action shall be taken for REPORTABLE OCCURRENCES

- A. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 15.6.9.2.
- B. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the Manager's Supervisory Staff (MSS) and submitted to the Off Site Review Committee (OSRC) and the Vice President - Nuclear Plant.

15.6.6-1

15.6.7 ACTION TO BE TAKEN IF A SAFETY LIMIT IS EXCEEDED

Specification

- A. If a safety limit is exceeded, the affected reactor shall be shut down and reactor operation shall not be resumed until approval is received from the NRC.
- B. An immediate report shall be made to the Vice President Nuclear Plant and the Chairman of the Off-Site Review Committee.
- C. The Vice President Nuclear Plant shall report the circumstances to the NRC.
- D. A Safety Limit Violation Report including a complete analysis of the circumstances leading to and resulting from the occurrence, effects upon facility components, systems or structures, together with recommendations to prevent a recurrence, shall be prepared. This report shall be submitted to the Vice President - Nuclear Plant and the Chairman of the Off-Site Review Committee. A Safety Limit Violation Report shall be submitted to the NRC by the Vice President -Nuclear Plant within 10 days of the occurrence.

15.6.8 PLANT OPERATING PROCEDURES

15.6.8.1

The plant shall be operated and maintained in accordance with approved procedures. <u>Major</u> procedures, supported by appropriate <u>minor</u> procedures (such as checkoff lists, operating instructions, data sheets, alarm responses, chemistry analytical procedures, etc.) shall be provided for the following operations where these operations involve nuclear safety of the plant:

 Normal sequences of startup, operation and shutdown of components, systems and overall plant.

2. Refueling.

- 3. Specific and foreseen potential malfunctions of systems or components including abnormal reactivity changes.
- 4. Security Plan Implementation
- 5. Emergencies which could involve release of radioactivity.

6. Nuclear core testing.

7. Surveillance and Testing of safety related equipment.

15.6.8.2 Approval of Procedures

- A. All <u>major</u> procedures of the categories listed in 15.6.8.1 (except 15.6.8.1.4) and 15.6.11.1, and modifications to the intent thereof, shall be reviewed by the Manager's Supervisory Staff and approved by the Manager - Nuclear Power Division prior to implementation.
- B. <u>Minor</u> procedures (checkoff lists, operating instructions, data sheets, alarm responses, chemistry analytical procedures, technical instructions, special and routine maintenance procedures, laboratory manuals, etc.) shall, prior to initial use, be reviewed by the Manager's Supervisory Staff and approved by the Manager - Nuclear Power Division.

15.6.8-1

15.6.8.3 Changes to Procedures

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Temporary changes to <u>major</u> procedures, of the categories listed in 15.6.8.1 (except 15.6.8.1.4) and 15.6.11.1, which do not change the intent of the original or subsequent approved procedure, may be made provided such changes to operating procedures are approved by the Duty Shift Supervisor and one of the Duty and Call Superintendents.

For temporary changes to <u>major</u> procedures under the jurisdiction of Maintenance, Instrumentation and Control, Reactor Engineering, or Chemistry and Health Physics which do not change the intent, changes may be made upon approval of the cognizant group head and a Duty and Call Superintendent. All Temporary changes to <u>major</u> procedures (made by the Duty Shift Supervisor and a Duty and Call Superintendent or a cognizant group head) shall subsequently be reviewed by the Manager's Supervisory Staff and approved by the Manager - Nuclear Power Division within 2 weeks; and shall only become permanent changes after these Manager's Staff review and Manager's approval steps.

B. All temporary or permanent changes to <u>minor</u> operating procedures, (checkoff lists, alarm responses, data sheets, operating instructions, etc.) shall be approved by the Duty Shift Supervisor, and shall be subsequently reviewed and approved by the Operations Superintendent. All temporary or permanent changes to other <u>minor</u> procedures under the jurisdiction of Maintenance, Instrumentation and Control, Reactor Engineering, or Chemistry and Health Physics, shall be approved by a supervisor of the cognizant group and shall be subsequently reviewed and approved by the group head of the cognizant group.

15.6.8-2

15.6.9 Plant Reporting Requirement

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following program for reporting of operating information shall be followed. Reports should be addressed to the Director, Directorate of Regulatory Operations, Region III unless otherwise noted.

15.6.9.1 Routine Reports

- A. Startup Report
 - 1. A summary report of plant startup and power escalation testing which addresses each of the tests identified in the FFDSAR and includes a general description of the measured values obtained during the test program and a comparison of these values with design predictions and specifications must be submitted under the following conditions:
 - a. Receipt of an operating license
 - Amendment to the license involving a planned increase in power level.
 - c. Installation of fuel that has a different design or has been manufactured by a different fuel supplier.
 - Modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

Any corrective actions that were required to obtain satisfactory operation shall also be described.

 This report shall be submitted within the earliest time frame of the following:

Amendment No. 19

90 days following completion the startup tests.

 b. 90 days following resumption or commencement of commercial power operation.

c. 9 months following initial criticality.

- B. Annual Operating Report
 - A report covering the operation of the units during the previous calendar year shall be submitted prior to March 1 of each year.

2. The report shall include:

- A narrative summary of operating experience during the report period relating to safe operation of the facility, including safety-related maintenance.
- b. A description of each outage or forced reduction in power of over 20 percent of design power level where the reduction extends for greater than four hours. This description shall include the proximate cause, system or major component involved, reference to any pertaining abnormal occurrences, corrective action and maintenance undertaken, operating time lost, and a report of any release of radioactivity or radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

tabulation on an annual basis . the number of station, utility, and other personnel receiving exposures greater than 100 mrem/year and their associated man-rem exposure according to work and job functions. The dose assignments to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20 percent of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

d. Indications of failed fuel resulting from irradiated fuel examinations, including eddy current tests, ultrasonic tests, or visual examinations completed during the report period.

C. Monthly Operating Reports

c.

- Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis.
- 2. Completed reports shall be sent to the Director of Regulatory Operations, U. S. Nuclear Regulatory Commission, Washington, D. C., 20555 with a copy to RO Region III by the tenth of each month following the calendar month covered by the report.

15.6.9.2 Reportable Occurrences

A. Prompt Notification with Written Followup

The types of events listed in items 1 through 9 below shall be reported as expeditiously as possible within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director, Regulatory Operations, Region III, or his designate, no later than the first working day following the event. A written followup report must be submitted within two weeks. This written followup report shall include a completed copy of the licensee event report form, and may include additional narrative material to provide complete explanation of the circumstances surrounding the event.

- Failure of the reactor protection system or other systems subject to limiting safety-system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety-system setting in the technical specifications or failure to complete the required protective function.
- 2. Operation of the unit or affected systems when any parameter or operation subject to a limiting condition for operation is less conservative than the least conservative aspect of the limiting condition for operation established in the Technical Specifications.
- 3. Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

- 4. Reacti 'ty anomalies, involving disagre 'ent with the predicted value of reactivity balance under steady state conditions during power operation, greater than or equal to 1%Δk/k; a calculated reactivity balance indicating a shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds, or, if sub-critical, an unplanned reactivity insertion of more than 0.5%Δk/k or occurrence of any unplanned criticality.
- 5. Failure or malfunction of one or more components which prevents, or could prevent by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the FFDSAR.
- 6. Peronnel error or procedural inadequacy which prevents, or could prevent by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the FFDSAR.
- 7. Conditions arising from natural or manmade events that, as a direct result of the event, require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.
- 8. Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.

- 9. Performance of structures, systems, or concents that requires remedial action or corrective measures to prevent operation in a manner less conservative than that assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.
- B. Thirty-Day Written Reports

The types of events listed in items 1 through 4 below have lesser immediate importance. These events shall be the subject of written reports to the Director, Regulatory Operations, Region III within 30 days of the occurrence of the event. The written report shall include, as a minimum, a completed copy of the licensee event report form, and may be supplemented, as needed to provide complete explanation of the circumstances surrounding the event.

- Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.
- 2. Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

- 3. Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.
- 4. Abnormal degradation of systems other than those specified in 15.6.8.2.A.3 above designed to contain radioactive material resulting from the fission process.

15.6.9.2 UNIQUE REPORTING REQUIREMENTS

The following written reports shall be submitted to the Director, Office of Nuclear Reactor Regulation, USNRC:

- A. Each integrated leak test shall be the subject of a summary technical report, including results of the local leak rate tests and isolation valve leak rate tests since the last report. The report shall include analysis and interpretations of the results which demonstrate compliance with specified leak rate limits.
- B. At the end of the first five years of the in-service inspection period, a review of the inspection program will be conducted. This review will evaluate the results obtained to date in view of possible modifications to the inspection program. These modifications may increase or decrease surveillance requirements as experience dictates. The results of these evaluations will be reported to the Nuclear Regulatory Commission.
- C. Submission of a report within 60 days after January 1 and after July 1 each year for the six-month period or fraction thereof, ending June 30 and December 31 containing:

,15.6.9-7

1. Information relative to the quantities of liquid, gaseous and solid radioactive effluents released from the facility, and effluent volumes used in maintaining the releases within the limits of 10 CFR 20 shall be provided (summarized on a monthly basis) as follows:

a. Liquid Releases

- Total radioactivity (in curies) released, other than tritium, and average concentration at outfall of discharge canal.
- (2) Total tritium (in curies) discharged, and average concentration at outfall of discharge canal.
- (3) Total volume (in gallons) of liquid waste released into circulating water discharge.
- (4) Total volume (in gallons) of dilution water used.
- (5) The maximum concentration released (averaged over the period of a single release).
- (6) Estimated total radioactivity (in curies) released, by nuclide (other than tritium), based on representative isotopic analyses performed.
- (7) Percent of applicable limits released, basedon nuclide identification performed.

- b. Gaseous Releases
 - (1) Total radioactivity (in curies) released of:
 - (a) Noble gases.
 - (b) Halogens.
 - (c) Particulates with half-lives greater than eight days.
 - (d) Tritium oxide.
 - (2) Maximum release rate (for any one-hour period).
 - (3) Estimated total radioactivity (in curies) released by nuclide (other than tritium) based on representative isotopic analyses performed.
 - (4) Percent of applicable limit released, based on nuclide identification performed.
- c. Solid Waste
 - (1) The total amount of solid waste packaged (in cubic feet).
 - (2) Estimated total radioactivity (in curies) involved.
 - (3) The dates of shipment and disposition (if shipped off-site).
- 2. New and Spent Fuel Receipts and Shipments
 - a. Number and type of new fuel assemblies received during the reporting period, if any.
 - b. Number of spent fuel assemblies shipped off site during the reporting period, if any.
- 3. Environmental Monitoring
 - A narrative summary of pertinent environmental monitoring activities performed during the reporting period, including:

- The number and types of samples taken and the measurements made on the samples; e.g., gross beta gamma scan, etc.
- (2) Any changes made in sample types or locations during the reporting period, and criteria for these changes.
- b. A summary of survey results during the reporting period.

4. Leak Testing of Sources

Results of required leak tests performed on seal sources if the tests reveal the presence of 0.005 microcuries or more of removable contamination.

15.6.10 PLANT OPERATING RECORDS

Specification

Records and logs relative to the following items shall be retained for six (6) years unless a longer period is required by applicable regulations.

- A. Records of normal plant operation, including power levels and periods of operation at each power level.
- B. Records of principal maintenance activities, including inspection, repair, substitution or replacement of principal items of equipment pertaining to nuclear safety.

C. Records of reportable occurrences.

- D. Records of periodic checks, inspections and calibrations performed to verify that surveillance requirements are being met.
- E.* Records of new and spent fuel inventory and assembly histories.
- F.* Records of changes made to the plant and to plant drawings as described in the FFDSAR.
- G.* Records of plant radiation and contamination surveys.
- H.* Records of off-site environmental monitoring surveys.
- I.* Records of radiation exposure of all plant personnel, including all contractor personnel and visitors who enter radiation control areas in the plant.
- J.* Records of radioactivity levels in liquid and gaseous wastes released to the environment and dilution of these wastes.

K. Records of any special reactor tests or experiments.

L. Records of changes made in the Operating Procedures.

15.6. 10-1

- M. Test results, in units of microcuries, for leak test performed pursuant to Specification 15.4.12.
- N. Record of annual physical inventory verifying accountability of sources subject to Specification 15.4.12.
- 0. *Records of training and qualification for current plant NRC licensed staff and key personnel.
- P. *Records of in-service inspections performed pursuant to these technical specifications.
- Q. *Records of Quality Assurance activities required by the QA Manual.
- R. *Records of reviews performed pursuant to 10 CFR 50.59.
- S. *Records of meetings of the Manager's Supervisory Staff and the Off-Site Review Committee.

*Starred items will be permanently retained.

15.6.10-2

15.6.11 RADIATION PROTECTION PROGRAM

Specification

Radiological control procedures shall be written and made available to all station personnel, and shall state permissible radiation exposure levels. The radiation protection program shall meet the requirements of 10 CFR 20, with the exception of the following:

Paragraph 20.203 - Caution signs, labels and signals

In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2), each radiation area in which the intensity of radiation is <u>greater than 100 mrem/hr</u> shall be barricaded and conspicuously posted as a High Radiation Area, and entrance thereto shall be controlled in accordance with the Point Beach Nuclear Plant Radiation Protection Manual, Section 2.9, Radiation Work Permit. A person or persons permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area. In addition, each High Radiation Area outside the containment

In addition, each high Kadiation kiele outside the concernment building in which the intensity of radiation is greater than <u>1000 mrem/hr</u> shall be provided with locked barricades to prevent unauthorized entry into these areas, and the keys to these locked barricades shall be maintained under the administrative control of the Duty Shift Supervisor. 15.6.12

RESPIRATORY PROTECTION PROGRAM

15.6.12.1 ALLOWANCE

Pursuant to 10 CFR 20.103(c)(1) and (3), allowance may be made for the use of respiratory protective equipment in conjunction with activities authorized by the operating license for this facility in determining whether individuals in restricted areas are exposed to concentrations in excess of the limits specified in Appendix B, Table I, Column 1, of 10 CFR 20, subject to the following conditions and limitations:

- a. The limits provided in Section 20.103(a) and (b) shall not be exceeded.
 - b. If the radioactive material is of such form that intake through the skin or other additional route is likely, individual exposures to radioactive material shall be controlled so that the radioactive uptake by any critical organ from all routes of intake averaged over 7 consecutive days does not exceed that which would result from inhaling such radioactive material for 40 hours at the pertinent concentration values provided in Appendix B, Table-I, Column 1, of 10 CFR 20.
- c. For radioactive materials designated "Sub" in the "Isotope" column of Appendix B, Table I, Column 1 of 10 CFR20, the concentration value specified shall be based upon exposure to the material as an external radiation source. Individual

15.6.12 -1

exposures to these materials shall be accounted for as part of the limitation on individual dose in \$20.101.

15.6.12.2 PROTECTION PROGRAM

In all operations in which adequate limitation of the inhalation of radioactive material by the use of process or other engineering controls is impracticable, the licensee may permit an individual in a restricted area to use respiratory protective equipment to limit the inhalation of airborne radioactive material, provided:

The limits specified in 15.6.12.1 above, are not exceeded. Respiratory protective equipment is selected and used, b. . based on the highest concentrations of airborne radioactive material expected or determined by the air sampling program, to ensure that concentrations inhaled by an individual wearing the equipment do not exceed the pertinent 10 CFR 20 limits. For the purpose of this subparagraph, the concentration of radioactive material that is inhaled when respirators are worn may be determined by dividing the ambient airborne concentration by the protection factor specified in Table 15.6.12-1 for the respirator protective equipment worn. If the intake of radioactivity is later determined by other measurements to have been different than that initially estimated, the later quantity shall be used in evaluating the exposures.

- c. The licensee advises each respirator user that he may ask to leave the area at any time for relief from respirator use in case of equipment malfunction, physical or psychological distress, or any other condition that might cause reduction in the protection afforded the wearer.
- d. A respiratory protection program, described in the FFDSAR, Section 11.2.3, shall be maintained to ensure that the objectives of this specification are met. Such a program shall include:
 - Air sampling and other surveys sufficient to identify the hazard, to evaluate individual exposures, and to permit proper selection of respiratory protective equipment.
 - Written procedures to assure proper selection, supervision, and training of personnel using such protective equipment.
 - Written procedures to assure the adequate fitting of respirators and the checking of respiratory protective equipment for operability, immediately prior to use.
 - Written procedures for maintenance to assure full
 effectiveness of respiratory protective equipment, including
 issuance, cleaning and decontamination, inspection, repair,
 and storage.
 - 5. Written operational and administrative procedures for proper use of respiratory protective equipment including provisions for planned limitations on working times as necessitated by operational conditions.

. -15.6.12-3
Bioassays and/or whole body counts of individuals (and other surveys, as appropriate) to evaluate individual exposures and to assess protection actually provided.

- e. The licensee shall use equipment approved by the U. S. Bureau of Mines (USBOM) or the National Institute for Occupational Safety and Health (NIOSH) under appropriate approval schedules when available. Protection factors for equipment not approved under USBOM or NIOSH approval schedules shall be used only if the licensee has evaluated the equipment and can demonstrate by testing, or on the basis of reliable test information that the material and performance characteristics of the equipment are at least equal to those afforded by USBOM or NIOSH approved equipment of the same or similar type, as specified in Table 15.6.12-1.
 - Unless otherwise authorized by the Commission, the licensee shall not assign protection factors in excess of those specified in Table 15.6.12-1 in selecting and using respiratory protective equipment.

15.6.12.3 REVOCATION

The specifications of Section 15.6.12 shall be revoked in their entirety upon adoption of the proposed change to 10 CFR 20, Section 20.103, which would make such provisions unnecessary.

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TABLE 15.6.12-1

PROTECTION FACTORS FOR RESPIRATORS

	DESCRIPTION	MODES (1)	PROTECTION FACTORS (2) PARTICULATES AND VAPORS AND GASES EXCEPT TRITIUM OXIDE (3)	GUIDES TO SELECTION OF EQUIPMENT: BUREAU OF MINES APPROVAL SCHEDULES* FOR EQUIPMENT CAPABLE OF PROVIDING AT LEAST EQUIVALENT PROTECTION FACTORS
I.	AIR-PURIFYING RESPIRATORS			
•	Facepiece, half-mask ^(4,7) Facepiece, full ⁽⁷⁾	NP	5 100	30 CFR Part 11 Subpart K
			en e	
II.	ATMOSPHERE-SUPPLYING RESPIRATOR	-	· ·	
· · ·.	<pre>1. Airline respirator Facepiece, half-mask Facepiece, full Facepiece, full(7) Facepiece, full Hood Suit</pre>	CF CF D PD CF CI	100 1,000 100 1,000 (5,6) (5,6)	30 CFR Part 11 Subpart J
	2. <u>Self-contained breathing</u> apparatus (SCBA)	<u>7</u>	100	30 CER Part 11
	Facepiece, full Facepiece, full Facepiece, full	D PD R	100 1,000 100	Subpart H
II.	COMBINATION RESPIRATOR			
	Any combination of air- purifying and atmosphere- supplying respirator		Protection factor for type and mode of operation as	~9 CFR Part 11 s11.63(b)
			Listen anove	

1, 2, 3, 4, 5, 6, 7 [These notes are on the following pages.]

* Or schedule superseding for equipment of type listed.

Amendment No. 19

15.6.12-5

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TABLE 15.6.12-1 (Continued)

1 See the following symbols:

CF: continuous flow

D: demand

.2

- NP: negative pressure (i.e., negative phase during inhalation)
- PD: pressure demand (i.e., always positive pressure)

R: recirculating (closed circuit)

(a) For purposes of this specification the protection factor is a measure of the degree of protection afforded by a respirator, defined as the ratio of the concentration of airborne radio-active material outside the respiratory protective equipment to that inside the equipment (usually inside the facepiece) under conditions of use. It is applied to the ambient airborne concentration to estimate the concentration inhaled by the wearer according to the following formula:

Concentration Inhaled = <u>Imbient Airborne Concentration</u> Protection Factor

(b) The protection factors apply:

- (i) only for trained individuals wearing properly fitted respirators used and maintained under supervision in a well-planned respiratory protective program.
- (ii) for air-purifying respirators only when high efficiency (above 99.9% removal efficiency by dioctyl phthalate (DOP)
 test) particulate filters and/or sorbents appropriate to the hazard are used in atmospheres not deficient in oxygen.
- (iii) for atmosphere-supplying respirators only when supplied with adequate respirable air.

Excluding radioactive contaminants that present an absorption or submersion hazard. For tritium oxide approximately half of the intake occurs by absorption through the skin so that an overall protection factor of not more than approximately 2 is appropriate when atmosphere-supplying respirators are used to protect against tritium oxide. Air-purifying respirators are not recommended for use against tritium oxide. See also footnote ⁵, below, concerning supplied-air suits and hoods.

. 15.6.12-6

TABLE 15.6.12-1 (Continued)

Under chin type only. Not recommended for use where ambient airborne concentration could reach instantaneous values greater than 50 times the pertinent values in Appendix B, Table I, Column 1 of 10 CFR Part 20.

⁵ Appropriate protection factors must be determined taking account of the design of the suit or hood and its permeability to the contaminant under conditions of use. No protection factor greater than 1,000 shall be used except as authorized by the Commission.

No approval schedules currently available for this equipment. Equipment must be evaluated on the basis of testing, or on the basis of reliable test information.

Only for shaven faces.

NOTE 1:

Protection factors for respirators, as may be approved by the USBOM or NIOSH according to approval schedules for respirators to protect against airborne radionuclides, may be used to the extent that they do not exceed the protection factors listed in this Table. The protection factors in this Table may not be appropriate to circumstances where chemical or other respiratory hazards exist in addition to radioactive hazards. The selection and use of respirators for such circumstances should take into account approvals of the USBOM or NIOSH in accordance with applicable schedules.

NOTE 2:

Radioactive contaminants for which the concentration values in Appendix B, Table I of 10 CFR, Part 20, are based on internal dose due to inhalation may, in addition, present external exposure hazards at higher concentrations. Under such circumstances, limitations on occupancy may have to be governed by external dose limits.



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WISCONSIN ELECTRIC POWER COMPANY WISCONSIN MICHIGAN POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMERDMENT TO PACILITY OFFICATING LECENSE

Amendment No. 24 License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:

- A. The applications for amendment by Wisconsin Electric Power Company and Wisconsin Michigan Power Company (the licensees) dated June 1 and 29, 1976, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
- B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
- C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
- D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
- E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

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FOR THE NUCLEAR REGULATORY COMMISSION

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George Lear, Chief Operating Reactors Branch #3 Division of Operating Reactors

Attachment: Changes to the Technical Specifications

Date of Issuance: August 2, 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 24

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NO. 50-301

Replace in its entirety Section 15.6 - Administrative Controls (consisting of pages 15.6.1-1 through 15.6.6-3) with the attached revised pages. 15.6

ADMINISTRATIVE

15.6.1 RESPONSIBILITY

15.6.1.1 The Manager - Nuclear Power Division shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during absences from the Point Beach Nuclear Plant area of greater than 48 hours and where ready contact by telephone or other means is not assured.

15.6.2 ORGANIZATION

OFFSITE

15.6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 15.6.2-1.

FACILITY STAFF

15.6.2.2 The Facility organization shall be as shown on Figure 15.6.2-2 and:

- a. Each on-duty shift shall normally be composed of at least the minimum shift crew composition as noted in Figure 15.6.2-2.
- b. At least one licensed Operator shall be in the control room when fuel is in either reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures
 ⁴ shall be on site when fuel is in either reactor.
- e. ALL CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

15.6.1/2-1



MANAGEMENT ORGANIZATION CHART

15.6.2-1



- One per shift one unit two per shift - two units Auxiliary Operator two per shift - one unit three per shift - two units
 - <u>hift two_units_</u>'

- 1. The Operations Group shift makeup is the minimum size for operation in all modes except cold shutdown of Point Beach Units Nos. 1 and 2. [The Operations Group shift makeup may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members, provided immediate action is taken to restore the shift makeup to within the minimum requirements.
- 2. Health Physicist has optional reporting route to Manager on radiological health and safety matters.
- SRO AEC Senior Reactor Operator License RO - AEC Reactor Operator License

Control Operator (RO)

CONDUCT OF PLANT OPERATIONS CHART Figure 15.6.2-2

15.6.3 FACILITY STAFF QUALIFICATIONS

15.6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions.

15.6.4 TRAINING

15.6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

15.6.5 REVIEW AND AUDIT

15.6.5.1 Duty and Call Superintendents

- a. To assist and counsel the Shift Supervisor in case of Significant Operating Events, a Duty and Call Superintendent Group has been established. The Duty and Call Superintendent Group shall consist of any qualified person designated by the Manager - Nuclear Power Division.
- b. In the event of a reportable occurrence, the Shift Supervisor shall communicate with at least one Duty and Call Superintendent before taking other than the immediate onthe-spot action required. One Duty and Call Superintendent will be assigned to be "on call" at all times. The Duty and Call Superintendent provides continuously available counsel, call out backups, and review to the Shift Supervisor.

15.6.5.2 Manager's Supervisory Staff

FUNCTION

15.6.5.2.1 The Manager's Supervisory Staff (MSS) shall function to advise the Manager - Nuclear Power Division on all matters related to nuclear safety.

COMPOSITION

15.6.5.2.2 The Manager's Supervisory Staff shall be composed of the:

Chairman:	Manager - Nuclear Power Division			
Member:	: Operations Superintendent			
Member:	Maintenance Superintendent			
Member:	Instrument and Control Engineer			
Member:	Reactor Engineer			
Member:	Radiochemical Engineer			
Member:	Health Physicist			
1ember:	Assistant to the Manager - Nuclear Power Division			
	-			

ALTERNATES

15.6.5.2.3 Alternate members shall be appointed in writing by the MSS Chairman to serve on a temporary basis; however, no more than two alternates shall participate in MSS activities at any one time.

MEETING FREQUENCY

15.6.5.2.4 The MSS shall meet at least once per calendar month and as convened by the MSS Chairman.

QUORUM

15.6.5.2.5 A quorum of the MSS shall consist of the Chairman and four members including alternates.

RESPONSIBILITIES

15.6.5.2.6 The Manager's Supervisory Staff shall:

a) Review existing and proposed normal, abnormal and emergency operating procedures. Review maintenance procedures and
 ; proposed changes to these procedures and other procedures or changes thereto as determined by the Manager to affect plant operational safety. (Re: Section 15.6.7 for area of review.)

- b) Review all proposed tests and experiments related to Safety and the results thereof when applicable.
- c) Review all proposed changes to Technical Specifications.
- Review all proposed changes or modifications to plant systems or equipment where changes would require a change in operating or emergency procedures or that affect nuclear safety.
- e) Periodically review plant operations for industrial and nuclear safety hazards.
- f) Investigate violations or suspected violations of Technical Specifications, such investigations to include reports, evaluations, and recommendations to prevent recurrence, to the Vice President - Nuclear Plant and to the Chairman of the Off-Site Review Committee.
- g) Perform special reviews and investigations and prepare reports thereon as requested by the Chairman of the Off-Site Review Committee.
- h) Investigate, review, and report on all reportable occurrences.
- Cause to be conducted periodic drills on emergency procedures, including evacuation (partial or complete) of the site and check adequacy of communications with off-site support groups.

AUTHORITY

- 15.6.5.2.7
- a) The Supervisory Staff shall serve as advisory to the Manager Nuclear Power Division.
 - b) The Supervisory Staff shall recommend to the Manager approval or disapproval of proposals under items a) through d) above.In the event of disagreement between a majority of the

Supervisory Staff and decisions by the Manager, the course of action will be determined by the Manager and the disagreement recorded in the Staff minutes. Records of the disagreement will be included in the minutes sent for review to the Off-Site Review Committee and the Vice President - Nuclear Plant.

- c) The Supervisory Staff shall make tentative recommendations as to whether or not proposals considered by the Staff involve unreviewed safety questions. These recommendations shall be subject to review and further recommendations by the Off-Site Review Committee. Minutes shall be kept of all meetings of the Staff and copies shall be sent to the Vice President -Nuclear Plant and to the Chairman of the Off-Site Review Committee.
- d) The Supervisory Staff shall review and approve the contents of a report for each reportable occurrence. This report shall include an evaluation of the cause of the occurrence and recommendations for appropriate action to prevent or reduce the probability of a recurrence. Copies of all such reports shall be submitted to the Vice President -Nuclear Plant and to the Chairman of the Off-Site Review Committee.

RECORDS

15.6.5.2.8 The Manager's Supervisory Staff shall maintain written minutes of each meeting and copies shall be provided to the Vice President - Nuclear Plant and Chairman, Off-Site Review Committee.

15.6.5.3 OFF-SITE REVIEW COMMITTEE (OSRC)

FUNCTION

- 15.6.5.3.1 The Off-Site Review Committee shall function to provide independent review and audit of designated activities in the areas of:
 - a) nuclear power plant operations
 - b) nuclear engineering
 - c) chemistry and radiochemistry
 - d) metallurgy
 - e) instrumentation and control
 - f) radiological safety
 - g) mechanical and electrical engineering
 - h) quality assurance practices
 - i) environmental monitoring

COMPOSITION

15.6.5.3.2 The Off-Site Review Committee is made up of a minimum of five regular members appointed by the Vice President - Nuclear Plant and one ex-officio member. Of the five or more regular members, at least two will be persons not directly employed by WEPCo. or WMPCo. All members will be experienced in one or more aspects of the nuclear industry. The ex-officio member will be the Vice President - Nuclear Plant of Wisconsin Michigan Power Company.

ALTERNATES

15.6.5.3.3 Alternate members may be appointed in writing by the OSRC Chairman to serve on a temporary basis; however, no more than two alter-

nates shall participate in OSRC activities at any one time.

CONSULTANTS

15.6.5.3.4 Consultants shall be utilized as determined by the OSRC Chairman to provide expert advice to the OSRC.

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MEETING FREQUENCY

15.6.5.3.5 The OSRC shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least twice per year at approximately six month intervals thereafter. QUORUM

15.6.5.3.6 A quorum of OSRC shall consist of the Chairman or his designated alternate and three members. No more than a minority of the quorum shall have line responsibility for operation of the facility.

REVIEW

15.6.5.3.7 The OSRC shall review:

- a) The safety evaluations for 1) changes to procedures, equipment or systems, and 2) tests or experiments completed under the provision of 10 CFR, Section 50.59, to verify that such actions did not constitute an unreviewed safety question.
- b) Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in 10
 CFR, Section 50.59.
- c) Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR Section 50.59.
- d) Proposed changes in Technical Specifications or Licenses.
- e) Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f) Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g) Reportable Occurrences requiring 24 hour notification to the Commission.

Amendment No. 24

- h) Any in_cation of an unanticipated defi_ency in some aspect of design or operation of safety related structures, systems, or components.
- Reports and meeting minutes of the Manager's Supervisory Staff.

AUDITS

15.6.5.3.8 Audits of facility activities shall be performed under the cognizance of the OSRC. These audits shall encompass:

- a) The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per year.
- b) The performance, training and qualifications of the licensed operating staff at least once per year.
- c) The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least twice per year at approximately six month intervals.
- d) The results of quarterly audits by the Quality Assurance and Technical Services Department on the performance of activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per two years.
- Emergency Plan and implementing procedures at least once per 2 years.
- f) Any other area of facility operation considered appropriateby the Vice President Nuclear Plant.

AUTHORITY

15.6.5.3.9 The OSRC shall report to and advise the Vice President - Nuclear Plant on those areas of responsibility specified in Section 15.6.5.3.7 and

15.7.6.3.8.

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RECORDS

15.6.5.3.10 Records of OSRC activities shall be prepared, approved and distributed as indicated below:

- a) Minutes of each OSRC meeting shall be prepared, approved and forwarded to the Vice President - Nuclear Plant within 14 days following each meeting.
- b) Reports of reviews encompassed by Section 15.6.5.3.7.e, f and g above, shall be prepared, approved and forwarded to the Vice President - Nuclear Plant within 14 days following completion of the review.
- c) Audit reports encompassed by Section 15.6.5.3.8 above, shall be forwarded to the Vice President - Nuclear Plant and to the management positions responsible for the areas audited within 30 days after completion of the audit.

Specification

The following action shall be taken for REPORTABLE OCCURRENCES

- A. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification .5.6.9.2.
- B. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the Manager's Supervisory Staff (MSS) and submitted to the Off Site Review Committee (OSRC) and the Vice President - Nuclear Plant.

15.6.7. ACTION TO BE TAKEN IF A SAFETY LIMIT IS EXCEEDED

Specification

- A. If a safety limit is exceeded, the affected reactor shall be shut down and reactor operation shall not be resumed until approval is received from the NRC.
- B. An immediate report shall be made to the Vice President Nuclear Plant and the Chairman of the Off-Site Review Committee.
- C. The Vice President Nuclear Plant shall report the circumstances to the NRC.
- D. A Safety Limit Violation Report including a complete analysis of the circumstances leading to and resulting from the occurrence, effects upon facility components, systems or structures, together with recommendations to prevent a recurrence, shall be prepared. This report shall be submitted to the Vice President - Nuclear Plant and the Chairman of the Off-Site Review Committee. A Safety Limit Violation Report shall be submitted to the NRC by the Vice President -Nuclear Plant within 10 days of the occurrence.

15.6.8 PLANT OPERATING PROCEDURES

- 15.6.8.1 The plant shall be operated and maintained in accordance with approved procedures. <u>Major</u> procedures, supported by appropriate <u>minor</u> procedures (such as checkoff lists, operating instructions, data sheets, alarm responses, chemistry analytical procedures, etc.) shall be provided for the following operations where these operations involve nuclear safety of the plant:
 - Normal sequences of startup, operation and shutdown of components, systems and overall plant.
 - 2. Refueling.
 - 3. Specific and foreseen potential malfunctions of systems or components including abnormal reactivity changes.
 - 4. Security Plan Implementation
 - 5. Emergencies which could involve release of radioactivity.

6. Nuclear core testing.

7. Surveillance and Testing of safety related equipment.

15.6.8.2 Approval of Procedures

- A. All <u>major</u> procedures of the categories listed in 15.6.8.1 (except 15.6.8.1.4) and 15.6.11.1, and modifications to the intent thereof, shall be reviewed by the Manager's Supervisory Staff and approved by the Manager - Nuclear Power Division prior to implementation.
- B. <u>Minor</u> procedures (checkoff lists, operating instructions, data sheets, alarm responses, chemistry analytical procedures, technical instructions, special and routine maintenance procedures, laboratory manuals, etc.) shall, prior to initial use, be reviewed by the Manager's Supervisory Staff and approved by the Manager - Nuclear Power Division.

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15.6.8.3 Changes to Procedures

A. Temporary changes to <u>major</u> procedures, of the categories listed in 15.6.8.1 (except 15.6.8.1.4) and 15.6.11.1, which do not change the intent of the original or subsequent approved procedure, may be made provided such changes to operating procedures are approved by the Duty Shift Supervisor and one of the Duty and Call Superintendents.

For temporary changes to <u>major</u> procedures under the jurisdiction of Maintenance, Instrumentation and Control, Reactor Engineering, or Chemistry and Health Physics which do not change the intent, changes may be made upon approval of the cognizant group head and a Duty and Call Superintendent. All Temporary changes to <u>major</u> procedures (made by the Duty Shift Supervisor and a Duty and Call Superintendent or a cognizant group head) shall subsequently be reviewed by the Manager's Supervisory Staff and approved by the Manager - Nuclear Power Division within 2 weeks; and shall only become permanent changes after these Manager's Staff review and Manager's approval steps.

B. All temporary or permanent changes to <u>minor</u> operating procedures, (checkoff lists, alarm responses, data sheets, operating instructions, etc.) shall be approved by the Duty Shift Supervisor, and shall be subsequently reviewed and approved by the Operations Superintendent. All temporary or permanent changes to other <u>minor</u> procedures under the jurisdiction of Maintenance, Instrumentation and Control, Reactor Engineering, or Chemistry and Health Physics, shall be approved by a supervisor of the cognizant group and shall be subsequently reviewed and approved by the group head of the cognizant group.

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15.6.9 Plant Reporting Requirement

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following program for reporting of operating information shall be followed. Reports should be addressed to the Director, Directorate of Regulatory Operations, Region III unless otherwise noted.

15.6. .1 Routine Reports

- A. Startup Report
 - 1. A summary report of plant startup and power escalation testing which addresses each of the tests identified in the FFDSAR and includes a general description of the measured values obtained during the test program and a comparison of these values with design predictions and specifications must be submitted under the following conditions:
 - a. Receipt of an operating license
 - b. Amendment to the license involving a planned increase in power level.
 - c. Installation of fuel that has a different design
 or has been manufactured by a different fuel
 supplier.
 - Modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

Any corrective actions that were required to obtain satisfactory operation shall also be described.

2. This report shall be submitted within the earliest time frame of the following:

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- c. tabulation on an annual basis _: the number of station, utility, and other personnel receiving exposures greater than 100 mrem/year and their associated man-rem exposure according to work and job functions. The dose assignments to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20 percent of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.
- d. Indications of failed fuel resulting from irradiated fuel examinations, including eddy current tests, ultrasonic tests, or visual examinations completed during the report period.

C. Monthly Operating Reports

- Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis.
- 2. Completed reports shall be sent to the Director of Regulatory Operations, U. S. Nuclear Regulatory Commission, Washington, D. C., 20555 with a copy to RO Region III by the tenth of each month following the calendar month covered by the report.

- 90 days following completion the startup tests.
 b. 90 days following resumption or commencement of commercial power operation.
- c. 9 months following initial criticality.
- B. Annual Operating Report
 - A report covering the operation of the units during the previous calendar year shall be submitted prior to March 1 of each year.
 - 2. The report shall include:
 - A narrative summary of operating experience during the report period relating to safe operation of the facility, including safety-related maintenance.
 - b. A description of each outage or forced reduction in power of over 20 percent of design power level where the reduction extends for greater than four hours. This description shall include the proximate cause, system or major component involved, reference to any pertaining abnormal occurrences, corrective action and maintenance undertaken, operating time lost, and a report of any release of radioactivity or radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

15.6.9.2 Reportable Occurrences

A. Prompt Notification with Written Followup

The types of events listed in items 1 through 9 below shall be reported as expeditiously as possible within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director, Regulatory Operations, Region III, or his designate, no later than the first working day following the event. A written followup report must be submitted within two weeks. This written followup report shall include a completed copy of the licensee event report form, and may include additional narrative material to provide complete explanation of the circumstances surrounding the event.

- Failure of the reactor protection system or other systems subject to limiting safety-system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety-system setting in the technical specifications or failure to complete the required protective function.
- 2. Operation of the unit or affected systems when any parameter or operation subject to a limiting condition for operation is less conservative than the least conservative aspect of the limiting condition for operation established in the Technical Specifications.
- 3. Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

- 4. Reactir ty anomalies, involving disagreement with the predicted value of reactivity balance under steady state conditions during power operation, greater than or equal to 1%Δk/k; a calculated reactivity balance indicating a shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds, or, if sub-critical, an unplanned reactivity insertion of more than 0.5%Δk/k or occurrence of any unplanned criticality.
- 5. Failure or malfunction of one or more components which prevents, or could prevent by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the FFDSAR.
- 6. Peronnel error or procedural inadequacy which prevents, or could prevent by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the FFDSAR.
- 7. Conditions arising from natural or manmade events that, as a direct result of the event, require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.
- 8. Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.

- 9. Performant of structures, systems, or consents that requires remedial action or corrective measures to prevent operation in a manner less conservative than that assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.
- B. Thirty-Day Written Reports

The types of events listed in items 1 through 4 below have lesser immediate importance. These events shall be the subject of written reports to the Director, Regulatory Operations, Region III within 30 days of the occurrence of the event. The written report shall include, as a minimum, a completed copy of the licensee event report form, and may be supplemented, as needed to provide complete explanation of the circumstances surrounding the event.

- Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.
- 2. Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

- 3. Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.
- 4. Abnormal degradation of systems other than those specified in 15.6.8.2.A.3 above designed to contain radioactive material resulting from the fission process.

15.6.9.2 UNIQUE REPORTING REQUIREMENTS

The following written reports shall be submitted to the Director, Office of Nuclear Reactor Regulation, USNRC:

- A. Each integrated leak test shall be the subject of a summary technical report, including results of the local leak rate tests and isolation valve leak rate tests since the last report. The report shall include analysis and interpretations of the results which demonstrate compliance with specified leak rate limits.
- B. At the end of the first five years of the in-service inspection period, a review of the inspection program will be conducted. This review will evaluate the results obtained to date in view of possible modifications to the inspection program. These modifications may increase or decrease surveillance requirements as experience dictates. The results of these evaluations will be reported to the Nuclear Regulatory Commission.
- C. Submission of a report within 60 days after January 1 and after July 1 each year for the six-month period or fraction thereof, ending June 30 and December 31 containing:

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1. Information relative to the quantities of liquid, gaseous and solid radioactive effluents released from the facility, and effluent volumes used in maintaining the releases within the limits of 10 CFR 20 shall be provided (summarized on a monthly basis) as follows:

a. Liquid Releases

- Total radioactivity (in curies) released, other than tritium, and average concentration at outfall of discharge canal.
- (2) Total tritium (in curies) discharged, and average concentration at outfall of discharge canal.
- (3) Total volume (in gallons) of liquid waste released into circulating water discharge.
- (4) Total volume (in gallons) of dilution water used.
- (5) The maximum concentration released (averaged over the period of a single release).
- (6) Estimated total radioactivity (in curies) released, by nuclide (other than tritium), based on representative isotopic analyses performed.
- (7) Percent of applicable limits released, based on nuclide identification performed.

- b. Gaseous Releases
 - (1) Total radioactivity (in curies) released of:
 - (a) Noble gases.
 - (b) Halogens.
 - (c) Particulates with half-lives greater than eight days.
 - (d) Tritium oxide.
 - (2) Maximum release rate (for any one-hour period).
 - (3) Estimated total radioactivity (in curies) released by nuclide (other than tritium) based on representative isotopic analyses performed.
 - (4) Percent of applicable limit released, based on nuclide identification performed.
- c. Solid Waste
 - (1) The total amount of solid waste packaged (in cubic feet).
 - (2) Estimated total radioactivity (in curies) involved.
 - (3) The dates of shipment and disposition (if shipped off-site).
- 2. New and Spent Fuel Receipts and Shipments
 - a. Number and type of new fuel assemblies received during the reporting period, if any.
 - b. Number of spent fuel assemblies shipped off site during the reporting period, if any.
- 3. Environmental Monitoring
 - a. A narrative summary of pertinent environmental monitoring activities performed during the reporting period, including:

- The number and types of samples taken and the measurements made on the samples; e.g., gross beta gamma scan, etc.
- (2) Any changes made in sample types or locations during the reporting period, and criteria for these changes.
- b. A summary of survey results during the reporting period.

4. Leak Testing of Sources

Results of required leak tests performed on seal sources if the tests reveal the presence of 0.005 microcuries or more of removable contamination.

15.6.10 PLANT OPERATING RECORDS

Specification

Records and logs relative to the following items shall be retained for six (6) years unless a longer period is required by applicable regulations. A. Records of normal plant operation, including power levels and periods

of operation at each power level.

B. Records of principal maintenance activities, including inspection, repair, substitution or replacement of principal items of equipment pertaining to nuclear safety.

C. Records of reportable occurrences.

- D. Records of periodic checks, inspections and calibrations performed to verify that surveillance requirements are being met.
- E.* Records of new and spent fuel inventory and assembly histories.
- F.* Records of changes made to the plant and to plant drawings as described in the FFDSAR.
- G.* Records of plant radiation and contamination surveys.
- H.* Records of off-site environmental monitoring surveys.
- I.* Records of radiation exposure of all plant personnel, including all contractor personnel and visitors who enter radiation control areas in the plant.
- J.* Records of radioactivity levels in liquid and gaseous wastes released to the environment and dilution of these wastes.

K. Records of any special reactor tests or experiments.

L. Records of changes made in the Operating Procedures.

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- M. Test results, in units of microcuries, for leak test performed pursuant to Specification 15.4.12.
- N. Record of annual physical inventory verifying accountability of sources subject to Specification 15.4.12.
- 0. *Records of training and qualification for current plant NRC licensed staff and key personnel.
- P. *Records of in-service inspections performed pursuant to these technical specifications.
- Q. *Records of Quality Assurance activities required by the QA Manual.
- R. *Records of reviews performed pursuant to 10 CFR 50.59.
- S. *Records of meetings of the Manager's Supervisory Staff and the Off-Site Review Committee.

*Starred items will be permanently retained.

Specifica n

Radiological control procedures shall be written and made available to all station personnel, and shall state permissible radiation exposure levels. The radiation protection program shall meet the requirements of 10 CFR 20, with the exception of the following:

Paragraph 20.203 - Caution signs, labels and signals

In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2), each radiation area in which the intensity of radiation is <u>greater than 100 mrem/hr</u> shall be barricaded and conspicuously posted as a High Radiation Area, and entrance thereto shall be controlled in accordance with the Point Beach Nuclear Plant Radiation Protection Manual, Section 2.9, Radiation Work Permit. A person or persons permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.

In addition, each High Radiation Area outside the containment building in which the intensity of radiation is greater than <u>1000 mrem/hr</u> shall be provided with locked barricades to prevent unauthorized entry into these areas, and the keys to these locked barricades shall be maintained under the administrative control of the Duty Shift Supervisor.

15.6.11-1

RESPIRATORY PROTECTION PROGRAM

15.6.12.1 ALLOWANCE

15.6.12

Pursuant to 10 CFR 20.103(c)(1) and (3), allowance may be made for the use of respiratory protective equipment in conjunction with activities authorized by the operating license for this facility in determining whether individuals in restricted areas are exposed to concentrations in excess of the limits specified in Appendix B, Table I, Column 1, of 10 CFR 20, subject to the following conditions and limitations:

- a. The limits provided in Section 20.103(a) and (b) shall not
 be exceeded.
- b. If the radioactive material is of such form that intake through the skin or other additional route is likely, individual exposures to radioactive material shall be controlled so that the radioactive uptake by any critical organ from all routes of intake averaged over 7 consecutive days does not exceed that which would result from inhaling such radioactive material for 40 hours at the pertinent concentration values provided in Appendix B, Table I, Column 1, of 10 CFR 20.
 - For radioactive materials designated "Sub" in the "Isotope" column of Appendix B, Table I, Column 1 of 10 CFR20, the concentration value specified shall be based upon exposure to the material as an external radiation source. Individual

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exposures to these materials shall be accounted for as part of the limitation on individual dose in §20.101.

15.6.12.2 PROTECTION PROGRAM

In all operations in which adequate limitation of the inhalation of radioactive material by the use of process or other engineering controls is impracticable, the licensee may permit an individual in a restricted area to use respiratory protective equipment to limit the inhalation of airborne radioactive material, provided:

The limits specified in 15.6.12.1 above, are not exceeded. Respiratory protective equipment is selected and used, **b**... based on the highest concentrations of airborne radioactive material expected or determined by the air sampling program, to ensure that concentrations inhaled by an individual wearing the equipment do not exceed the pertinent 10 CFR 20 limits. For the purpose of this subparagraph, the concentration of radioactive material that is inhaled when respirators are worn may be determined by dividing the ambient airborne concentration by the protection factor specified in Table 15.6.12-1 for the respirator protective equipment worn. If the intake of radioactivity is later determined by other measurements to have been different than that initially estimated, the later quantity shall be used in evaluating the exposures.

The licensee advises each respirator user that he may ask to leave the area at any time for relief from respirator use in case of equipment malfunction, physical or psychological distress, or any other condition that might cause reduction in the protection afforded the wearer.

Section 11.2.3, shall be maintained to ensure that the objectives of this specification are met. Such a program shall include:

> Air-sampling and other surveys sufficient to identify the hazard, to evaluate individual exposures, and to permit proper selection of respiratory protective equipment.

> Written procedures to assure proper selection, supervision, and training of personnel using such protective equipment.
> Written procedures to assure the adequate fitting of respirators and the checking of respiratory protective equipment for operability, immediately prior to use.

4. Written procedures for maintenance to assure full
effectiveness of respiratory protective equipment, including
issuance, cleaning and decontamination, inspection, repair,
and storage.

Written operational and administrative procedures for proper use of respiratory protective equipment including provisions for planned limitations on working times as necessitated by operational conditions.

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6. Bioassays and/or whole body counts of individuals (and other surveys, as appropriate) to evaluate individual exposures and to assess protection actually provided.

e. The licensee shall use equipment approved by the U. S. Bureau of Mines (USEOM) or the National Institute for Occupational Safety and Health (NIOSH) under appropriate approval schedules when available. Protection factors for equipment not approved under USBOM or NIOSH approval schedules shall be used only if the licensee has evaluated the equipment and can demonstrate by testing, or on the basis of reliable test information that the material and performance characteristics of the equipment ', are at least equal to those afforded by USBOM or NIOSH approved equipment of the same or similar type, as specified in Table 15.6.12-1.

Unless otherwise authorized by the Commission, the licensee shall not assign protection factors in excess of those specified in Table 15.6.12-1 in selecting and using respiratory protective equipment.

15.6.12.3 REVOCATION

The specifications of Section 15.6.12 shall be revoked in their entirety upon adoption of the proposed change to 10 CFR 20, Section 20.103, which would make such provisions unnecessary.

15.6.12-4

✓ TABLE 15.6.12-1

PROTECTION FACTORS FOR RESPIRATORS

	DESCRIPTION	MODES (1.)	PROTECTION FACTORS (2) PARTICULATES AND VAPORS AND GASES EXCEPT TRITIUM OXIDE (3)	GUIDES TO SELECTION OF EQUIPMENT: BUREAU OF MINES APPROVAL SCHEDULES* FOR EQUIPMENT CAPABLE OF PROVIDING AT LEAST EQUIVALENT PROTECTION FACTORS
I.	AIR-PURIFYING RESPIRATORS	•		
	Facepiece, half-mask ^(4,7) Facepiece, full ⁽⁷⁾	NP NP	5 1.00	30 CFR Part 11 Subpart K
	· · .		· · · · · · · · · · · · ·	. • • •
II.	ATMOSPHERE-SUPPLYING	-		· · · · · · · ·
	RESPIRATOR		: ·	
	7 Admitium and amitanticas			
· · ·	Facepiece half-mask	ĊŦ	100	20 CEP Part 11
	Facepiece, full	CF	1.000	Subpart I
	Facepiece, full(7)	D	100	Subpart 5
	Facepiece, full	PD	1.000	
	Hood	CF	(5,6)	
	Suit	CI	(5,6)	
		х. С	· · · · · · · · · · · · · · · · · · ·	÷
	2. Self-contained breathin	ıg		· .
	apparatus (SCBA)			· · · · ·
	Facepiece, full (7)	D .	100	30 CFR Part 11
	Facepiece, full	PD	1,000	Subpart H
	Facepiece, full	R	100	
		· · ·		
. 4. 4. 4	COMBINATION RESPIRATOR	•		
	my commutation of all-		Protection factor) CFR Part 11
	cupplying respirator	•	. for type and mode	911.03(D)
	Suppry mid respirator		listed above	
			JISted above	
	•		1. S.	· · · ·
		•	•	
		•		
٦.	2. 3. 4. 5. 6. 7	•	•.	
1	[These note	es are on the fol	llowing pages.]	

* Or schedule superseding for equipment of type listed.

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TABLE 15.6.12-1 (Continued)

¹ See the following symbols:

CF: continuous flow

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- NP: negative pressure (i.e., negative phase during inhalation)
- PD: pressure demand (i.e., always positive pressure)
- R: recirculating (closed circuit)

(a) For purposes of this specification the protection factor is a measure of the degree of protection afforded by a respirator, defined as the ratio of the concentration of airborne radioactive material outside the respiratory protective equipment to that inside the equipment (usually inside the facepiece) under conditions of use. It is applied to the ambient airborne concentration to estimate the concentration inhaled by the wearer according to the following formula:

Ambient Airborne Concentration

Concentration Inhaled = Protection Factor

(b) The protection factors apply:

- (i) only for trained individuals wearing properly fitted respirators used and maintained under supervision in a well-planned respiratory protective program.
- (ii) for air-purifying respirators only when high efficiency
 (above 99.9% removal efficiency by dioctyl phthalate (DOP)
 test) particulate filters and/or sorbents appropriate to the
 hazard are used in atmospheres not deficient in oxygen.
- (iii) for atmosphere-supplying respirators only when supplied with adequate respirable air.

Excluding radioactive contaminants that present an absorption or submersion hazard. For tritium oxide approximately half of the intake occurs by absorption through the skin so that an overall protection factor of not more than approximately 2 is appropriate when atmosphere-supplying respirators are used to protect against tritium oxide. Air-purifying respirators are not recommended for use against tritium oxide. See also footnote ⁵, below, concerning supplied-air suits and hoods. TABLE 15.6.12-1 (Continued)

Under chin type only. Not recommended for use where ambient airborne concentration could reach instantaneous values greater than 50 times the pertinent values in Appendix B, Table I, Column 1 of 10 CFR Part 20.

^b Appropriate protection factors must be determined taking account of the design of the suit or hood and its permeability to the contaminant under conditions of use. No protection factor greater than 1,000 shall be used except as authorized by the Commission.

No approval schedules currently available for this equipment. Equipment must be evaluated on the basis of testing, or on the basis of reliable test information.

' Only for shaven faces.

NOTE 1:

Protection factors for respirators, as may be approved by the USBOM or NIOSH according to approval schedules for respirators to protect against airborne radionuclides, may be used to the extent that they do not exceed the protection factors listed in this Table. The protection factors in this Table may not be appropriate to circumstances where chemical or other respiratory hazards exist in addition to radioactive hazards. The selection and use of respirators for such circumstances should take into account approvals of the USBOM or NIOSH in accordance with applicable schedules.

NOTE 2:

Radioactive contaminants for which the concentration values in Appendix B, Table I of 10 CFR, Part 20, are based on internal dose due to inhalation may, in addition, present external exposure hazards at higher concentrations. Under such circumstances, limitations on occupancy may have to be governed by external dose limits.



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 19 AND 24 TO FACILITY LICENSES DPR-44 AND 27

WISCONSIN ELECTRIC POWER COMPANY WISCONSIN MICHIGAN POWER COMPANY

POINT BEACH UNITS NOS. 1 AND 2

DOCKETS NOS. 50-266 AND 50-301

Introduction

By letters dated June 1 and 29, 1976, Wisconsin Electric Power Company (WEPCO) proposed changes to the Technical Specifications appended to Facility Operating Licenses Nos. DPR-24 and 27 for Point Beach Units Nos. 1 and 2. The proposed amendments involve changes to the administrative controls section of the Technical Specifications excluding any substantive changes to the Plant Reporting Requirements, and were submitted in response to our request of October 18, 1974. Changes to the Plant Reporting Requirements, Technical Specification 15.6.9, were issued in a separate licensing action on January 9, 1976.

Evaluation

The proposed changes would be administrative in nature and would affect the conduct of operation. The proposed changes are intended to provide uniform license requirements, and are consistent with requirements being incorporated in the Technical Specifications of newly licensed facilities.

Areas covered by the uniform specifications include licensee staffing qualifications and management procedures associated with operating the reactors. Identifying minimum acceptable qualifications for facility personnel will provide assurance of capable performance from the facility staff. Other administrative requirements also restated by the specifications assure uniformity and conformance to the desired features of the plant staffing and procedures. In addition, a radiation protection program delineates use of respiratory equipment in the event personnel are to be exposed to concentrations in excess of 10 CFR Part 20 concentrations. Similar changes have been approved for other operating reactor licensees, so all licensees will have the same requirements presented in a uniform manner.

During our review of the proposed changes we found that certain modifications to the proposal were necessary to have conformance with the desired regulatory position. These changes were discussed with and concurred in by the licensee, and have been incorporated into the proposed amendment. We have determined that the proposed changes, as modified, improve the licensee's program for evaluating plant performance and the reporting of the operating information needed by the Commission to assess safety related activities; and thus, is acceptable.

Environmental Findings

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR S51.5(d)(4) that an environmental statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: August 2, 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-266 AND 50-301

WISCONSIN ELECTRIC POWER COMPANY WISCONSIN MICHIGAN POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 19 and 24 to Facility Operating Licenses Nos. DFR-24 and DFR-27 issued to Wisconsin Electric Power Company and Wisconsin Michigan Power Company, which revised Technical Specifications for operation of the Point Beach Nuclear Plant Units Nos. 1 and 2, located in the town of Two Creeks, Manitowoc County, Wisconsin. The amendments are effective as of the date of issuance.

These amendments consist of changes in the Technical Specifications that involve changes to the administrative controls section, excluding any substantive changes to the Plant Reporting Requirements and were submitted in response to our request of October 18, 1974. Changes to the Plant Reporting Requirements, Technical Specification 15.6.9, were issued in a separate licensing action on January 9, 1976.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration. The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of these amendments.

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For further details with respect to this action, see (1) the applications for amendment dated June 1 and 29, 1976, (2) Amendment No. 19 to License No. DPR-24, (3) Amendment No. 24 to License No. DPR-27, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street N. W., Washington, D. C. and at the University of Wisconsin -Stevens Point Library, ATTN: Mr. Arthur M. Fish, Stevens Point, Wisconsin 54481.

A copy of items (2), (3) and (4) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland this 2 day of August 1976.

FOR THE NUCLEAR REGULATORY COMMISSION

George Légr, Chief Operating Reactors Branch #3 Division of Operating Reactors