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Dockets Nos. 50-266
and 50-301

Wisconsin Electric Power Company
Wisconsin Michigan Power Company
ATTN: Mr. Sol Burstein
Executive Vice President
231 West Michigan Street
Milwaukee, Wisconsin 53201

Gentlemen:

The Commission has issued the enclosed Amendments Nos. 17 and 22 to Facility Operating Licenses Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Units Nos. 1 and 2. The amendments consist of changes to the Technical Specifications and are in accordance with your application dated February 18, 1976.

The amendments consist of changes in the Technical Specifications that will add new shock suppressors (snubbers) limiting conditions for operation and surveillance requirements.

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:
1. Amendment No. 17 to License DPR-24
2. Amendment No. 22 to License DPR-27
3. Safety Evaluation
4. Federal Register Notice

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SURNAME ➤	CParrish	JWetmore:kmb	Ketchen	GLear	
DATE ➤	5/ 24 /76	5/ 21 /76	6/ 9 /76	5/ 18 /76	

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY
WISCONSIN MICHIGAN POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 17
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Electric Power Company and Wisconsin Michigan Power Company (the licensees) dated February 18, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the
Technical Specifications

Date of Issuance: June 18, 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 17

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-24

DOCKET NO. 50-266

Replace page 15.3.13-1, 15.3.13-2, Table 15.3.13-1, Table 15.3.13-1 (Continued), page 15.4.13-1 and page 15.4.13-2 with the attached revised pages.

15.3.13 SHOCK SUPPRESSORS (SNUBBERS)

Applicability

Applies to the operability of safety related snubbers.

Objective

To define those limiting conditions for operation for snubbers required to protect the primary coolant system and safety related systems.

Specification

1. During all modes of operation except Cold Shutdown and Refueling Shutdown all safety related snubbers as listed in Table 15.3.13-1 shall be operable except as noted in items 15.3.13.2 through 15.3.13.4 below.
2. If a snubber is determined to be inoperable, reactor operation may be continued no longer than 72 hours after the time the snubber was determined to be inoperable.
3. If the requirements of 15.3.13.1 and 15.3.13.2 cannot be met, an orderly shutdown shall be initiated and the reactor put in the Cold Shutdown condition within 36 hours.
4. If a snubber is determined to be inoperable while the reactor is in the Cold Shutdown or Refueling Shutdown condition, the snubber shall be made operable prior to reactor startup.
5. Snubbers may be added to safety related systems without prior License Amendment to Table 15.3.13-1 provided that a revision to the Table is included with a subsequent License Amendment request.

Basis

Snubbers are designed to prevent unrestrained pipe motion under dynamic loads as might occur during an earthquake as severe transient, while allowing normal thermal motion during plant startup and shutdown. The consequence of an inoperable snubber is an increase in the probability of structural damage to piping as a result of seismic or other events initiating dynamic loads. It is therefore required that all snubbers required to protect the primary coolant system, and other safety related systems or components, be operable during reactor operation.

Because the snubber protection is required only during relatively low probability events, a period of 72 hours is allowed for repairs or replacement. In case a shutdown is required, the allowance of 36 hours to reach a Cold Shutdown condition will permit an orderly shutdown consistent with standard operating procedures. Since plant power operation should not commence with knowingly defective safety related equipment, Specification 15.3.13.4 prohibits reactor startup with inoperable snubbers.

TABLE 15.3.13-1
SAFETY RELATED SHOCK SUPPRESSORS (SNUBBERS)

Snubber ID Number	Location	Elevation	Snubber in High Radiation Area During Shutdown*	Snubber Especially Difficult to Remove	Snubber Inaccessible During Normal Operation	Snubber Accessible During Normal Operation
	Unit 1					
37149A-1	"A" SG Top	100				X
37149A-2	"A" SG Top	100				X
37149A-6	"A" SG Side	66				X (2)
37149A-7	"A" SG Side	66				X (2)
37149A-8	"A" SG Side	66				X (2)
37149B-4	"B" SG Top	100				X
37149B-3	"B" SG Top	100				X
37149B-10	"B" SG Side	66				X (2)
37149B-7	"B" SG Side	66				X (2)
37149B-5	"B" SG Side	66				X (2)
3145	"A" Main Feed Line at 66'	61				X
3147	Containment Spray Header Above 66'	120				X
3150	Containment Spray Header Above 66'	120				X
3221	Pressurizer Relief Line	80				X
3222	Pressurizer Relief Line	80				X
3146	Power Operated Header-Pressurizer Cubicle	78				X
7762	Relief Valve Header-Pressurizer Cubicle	80				X
3142	SIS Line- Regen. HX Cubicle	34	X			X
3149	SIS Line at 21' Elevation	40				X (2)
3143	Reactor Vessel Keyway	3	X (1)		X	
3144	Reactor Vessel Keyway	3	X (1)		X	
	Unit 2					
37149A-17	"A" SG Side	66				X (2)
37149B-18	"A" SG Side	66				X (2)
37149A-19	"A" SG Side	66				X (2)
37149A-12	"A" SG Top	100				X
37149A-11	"A" SG Top	100				X
37149B-20	"B" SG Side	66				X (2)
37149B-15	"B" SG Side	66				X (2)
37149B-16	"B" SG Side	66				X (2)

*Modifications to this Table due to changes in high radiation areas shall be submitted to the NRC as part of the next license amendment.

TABLE 15.3.13-1 (CONTINUED)

Snubber ID Number	Location	Elevation	Snubber in High Radiation Area During Shutdown*	Snubber Especially Difficult to Remove	Snubber Inaccessible During Normal Operation	Snubber Accessible During Normal Operation
37149B-13	"B" SG Top	100				X
37149B-14	"B" SG Top	100				X
3377	Aux Feed Line to "A" SG	72				X
3367	SIS Line at 46'	50				X
3365	SIS Line at 46'	34				X
3370	At Overhead to Keyway at 26'	36				X
3378	Downstream of Valve PCV 434	80				X
3377	Downstream of Valve PCV 435	80				X
7763	Downstream of Pressurizer Safety Valve	80				X
3751	Line to Power Operated Relief Valves	77				X
3368	Valve 541 in "A" Loop Cubicle	41	X		X	
4019	Reactor Vessel Keyway	3	X (1)		X	
4020	Reactor Vessel Keyway	3	X (1)		X	
3366	Regen HX Cubicle at 26'	34	X			X (2)

(1) High radiation during shutdown with flux thimble withdrawn.

(2) Accessible during normal operation for visual inspection only.

*Modifications to this Table due to changes in high radiation areas shall be submitted to the NRC as part of the next license amendment.

15.4.13 Shock Suppressors (Snubbers)

Applicability

Applies to the periodic inspection and testing requirements of safety related snubbers.

Objective

To verify the operability of the snubbers.

Specifications

The following surveillance requirements apply to those snubbers listed in Table 15.3.13-1.

1. All snubbers shall be visually inspected to verify operability in accordance with the following schedule:

Number of Snubbers Found Inoperable During Inspection or During Inspection Interval	Next Required Inspection Interval
0	18 months ± 25%
1	12 months ± 25%
2	6 months ± 25%
3, 4	124 days ± 25%
5, 6, 7	62 days ± 25%
≥8	31 days ± 25%

The required inspection interval shall not be lengthened more than one step at a time.

2. During each refueling shutdown, two representative snubbers shall be functionally tested for operability. For each snubber found to be inoperable, an additional 10% of that type snubber shall be tested until no more failures are found or all units have been tested. Snubbers of rated capacity greater than 50,000 lb need not be functionally tested.
3. The initial inspection shall be performed within 6 months from the date of issuance of these specifications. For the purpose of entering the schedule in Specification 15.4.13.1, it shall be assumed that the facility had been on a 6 month inspection interval.

Basis

All safety related snubbers are visually inspected for overall integrity and operability. The inspection will include verification of proper orientation, adequate hydraulic fluid level and proper attachment of snubber to piping and structures. To further increase the assurance of snubber reliability, functional tests are performed once each refueling cycle on a representative sample of snubbers. These tests may include stroking of the unit to verify proper piston movement, lock-up and blood; however, on those units where such tests cannot be performed at the plant other types of functional tests may be performed using recommendations of the manufacturer whenever possible. Observed failures on these samples shall require testing of additional units.

The inspection frequency is based upon maintaining a constant level of snubber protection. Thus the required inspection interval varies inversely with the observed snubber failures. The number of inoperable snubbers found during a required inspection determines the time interval for the next required inspection. Inspections performed before that interval has elapsed may be used as a new reference point to determine the next inspection. However, the results of such early inspections performed before the original required time interval has elapsed (nominal time less 25%) may not be used to lengthen the required inspection interval. Any inspection whose results require a shorter inspection interval will override the previous schedule.

The snubbers selected for the testing and inspection mentioned in specification 15.4.13.2 above are chosen on a rotating basis such that over a period of eleven refueling cycles all accessible snubbers will have been tested.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY
WISCONSIN MICHIGAN POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 22
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Electric Power Company and Wisconsin Michigan Power Company (the licensees) dated February 18, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the
Technical Specifications

Date of Issuance: June 18, 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 22

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NO. 50-301

Replace page 15.3.13-1, 15.3.13-2, Table 15.3.13-1, Table 15.3.13-1 (Continued), page 15.4.13-1 and page 15.4.13-2 with the attached revised pages.

15.3.13 SHOCK SUPPRESSORS (SNUBBERS)

Applicability

Applies to the operability of safety related snubbers.

Objective

To define those limiting conditions for operation for snubbers required to protect the primary coolant system and safety related systems.

Specification

1. During all modes of operation except Cold Shutdown and Refueling Shutdown all safety related snubbers as listed in Table 15.3.13-1 shall be operable except as noted in items 15.3.13.2 through 15.3.13.4 below.
2. If a snubber is determined to be inoperable, reactor operation may be continued no longer than 72 hours after the time the snubber was determined to be inoperable.
3. If the requirements of 15.3.13.1 and 15.3.13.2 cannot be met, an orderly shutdown shall be initiated and the reactor put in the Cold Shutdown condition within 36 hours.
4. If a snubber is determined to be inoperable while the reactor is in the Cold Shutdown or Refueling Shutdown condition, the snubber shall be made operable prior to reactor startup.
5. Snubbers may be added to safety related systems without prior License Amendment to Table 15.3.13-1 provided that a revision to the Table is included with a subsequent License Amendment request.

Basis

Snubbers are designed to prevent unrestrained pipe motion under dynamic loads as might occur during an earthquake as severe transient, while allowing normal thermal motion during plant startup and shutdown. The consequence of an inoperable snubber is an increase in the probability of structural damage to piping as a result of seismic or other events initiating dynamic loads. It is therefore required that all snubbers required to protect the primary coolant system, and other safety related systems or components, be operable during reactor operation.

Because the snubber protection is required only during relatively low probability events, a period of 72 hours is allowed for repairs or replacement. In case a shutdown is required, the allowance of 36 hours to reach a Cold Shutdown condition will permit an orderly shutdown consistent with standard operating procedures. Since plant power operation should not commence with knowingly defective safety related equipment, Specification 15.3.13.4 prohibits reactor startup with inoperable snubbers.

TABLE 15.3.13-1
SAFETY RELATED SHOCK SUPPRESSORS (SNUBBERS)

Snubber ID Number	Location	Elevation	Snubber in High Radiation Area During Shutdown*	Snubber Especially Difficult to Remove	Snubber Inaccessible During Normal Operation	Snubber Accessible During Normal Operation
	Unit 1					
37149A-1	"A" SG Top	100				X
37149A-2	"A" SG Top	100				X
37149A-6	"A" SG Side	66				X (2)
37149A-7	"A" SG Side	66				X (2)
37149A-8	"A" SG Side	66				X (2)
37149B-4	"B" SG Top	100				X
37149B-3	"B" SG Top	100				X
37149B-10	"B" SG Side	66				X (2)
37149B-7	"B" SG Side	66				X (2)
37149B-5	"B" SG Side	66				X (2)
3145	"A" Main Feed Line at 66'	61				X
3147	Containment Spray Header Above 66'	120				X
3150	Containment Spray Header Above 66'	120				X
3221	Pressurizer Relief Line	80				X
3222	Pressurizer Relief Line	80				X
3146	Power Operated Header-Pressurizer Cubicle	78				X
7762	Relief Valve Header-Pressurizer Cubicle	80				X
3142	SIS Line- Regen. HX Cubicle	34	X			X (2)
3149	SIS Line at 21' Elevation	40				X
3143	Reactor Vessel Keyway	3	X (1)		X	
3144	Reactor Vessel Keyway	3	X (1)		X	
	Unit 2					
37149A-17	"A" SG Side	66				X (2)
37149B-18	"A" SG Side	66				X (2)
37149A-19	"A" SG Side	66				X (2)
37149A-12	"A" SG Top	100				X
37149A-11	"A" SG Top	100				X
37149B-20	"B" SG Side	66				X (2)
37149B-15	"B" SG Side	66				X (2)
37149B-16	"B" SG Side	66				X (2)

*Modifications to this Table due to changes in high radiation areas shall be submitted to the NRC as part of the next license amendment.

TABLE 15.3.13-1 (CONTINUED)

Snubber ID Number	Location	Elevation	Snubber in High Radiation Area During Shutdown*	Snubber Especially Difficult to Remove	Snubber Inaccessible During Normal Operation	Snubber Accessible During Normal Operation
37149B-13	"B" SG Top	100				X
37149B-14	"B" SG Top	100				X
3377	Aux Feed Line to "A" SG	72				X
3367	SIS Line at 46'	50				X
3365	SIS Line at 46'	34				X
3370	At Overhead to Keyway at 26'	36				X
3378	Downstream of Valve PCV 434	80				X
3377	Downstream of Valve PCV 435	80				X
7763	Downstream of Pressurizer Safety Valve	80				X
3751	Line to Power Operated Relief Valves	77				X
3368	Valve 541 in "A" Loop Cubicle	41	X		X	
4019	Reactor Vessel Keyway	3	X (1)		X	
4020	Reactor Vessel Keyway	3	X (1)		X	
3366	Regen HX Cubicle at 26'	34	X			X (2)

(1) High radiation during shutdown with flux thimble withdrawn.

(2) Accessible during normal operation for visual inspection only.

*Modifications to this Table due to changes in high radiation areas shall be submitted to the NRC as part of the next license amendment.

15.4.13 Shock Suppressors (Snubbers)

Applicability

Applies to the periodic inspection and testing requirements of safety related snubbers.

Objective

To verify the operability of the snubbers.

Specifications

The following surveillance requirements apply to those snubbers listed in Table 15.3.13-1.

1. All snubbers shall be visually inspected to verify operability in accordance with the following schedule:

Number of Snubbers Found Inoperable During Inspection or During Inspection Interval	Next Required Inspection Interval
0	18 months ± 25%
1	12 months ± 25%
2	6 months ± 25%
3, 4	124 days ± 25%
5, 6, 7	62 days ± 25%
≥8	31 days ± 25%

The required inspection interval shall not be lengthened more than one step at a time.

2. During each refueling shutdown, two representative snubbers shall be functionally tested for operability. For each snubber found to be inoperable, an additional 10% of that type snubber shall be tested until no more failures are found or all units have been tested. Snubbers of rated capacity greater than 50,000 lb need not be functionally tested.
3. The initial inspection shall be performed within 6 months from the date of issuance of these specifications. For the purpose of entering the schedule in Specification 15.4.13.1, it shall be assumed that the facility had been on a 6 month inspection interval.

Basis

All safety related snubbers are visually inspected for overall integrity and operability. The inspection will include verification of proper orientation, adequate hydraulic fluid level and proper attachment of snubber to piping and structures. To further increase the assurance of snubber reliability, functional tests are performed once each refueling cycle on a representative sample of snubbers. These tests may include stroking of the unit to verify proper piston movement, lock-up and blood; however, on those units where such tests cannot be performed at the plant other types of functional tests may be performed using recommendations of the manufacturer whenever possible. Observed failures on these samples shall require testing of additional units.

The inspection frequency is based upon maintaining a constant level of snubber protection. Thus the required inspection interval varies inversely with the observed snubber failures. The number of inoperable snubbers found during a required inspection determines the time interval for the next required inspection. Inspections performed before that interval has elapsed may be used as a new reference point to determine the next inspection. However, the results of such early inspections performed before the original required time interval has elapsed (nominal time less 25%) may not be used to lengthen the required inspection interval. Any inspection whose results require a shorter inspection interval will override the previous schedule.

The snubbers selected for the testing and inspection mentioned in specification 15.4.13.2 above are chosen on a rotating basis such that over a period of eleven refueling cycles all accessible snubbers will have been tested.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENTS NOS. 17 AND 22 TO LICENSES DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY
WISCONSIN MICHIGAN POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

DOCKETS NOS. 50-266 AND 50-301

Introduction

By letter dated February 18, 1976, Wisconsin Electric Power Company (WEPCO) proposed changes to the Technical Specifications appended to Facility Licenses DPR-24 and DPR-27 for Point Beach Units Nos. 1 and 2. The proposed changes were submitted in response to our request dated December 24, 1975. They would add new limiting conditions for operation and surveillance requirements for safety related shock suppressors (snubbers) to the Technical Specifications.

Discussion

Shock suppressors (snubbers) are designed to prevent unrestrained pipe motion under dynamic loads as might occur during an earthquake or severe transient, while allowing normal thermal movement during startup and shutdown. The consequence of an inoperable snubber would be an increase in the probability of structural damage to piping resulting from a seismic or other postulated event which initiates dynamic loads. It is, therefore, necessary that snubbers installed to protect safety system piping be operable during reactor operation and be inspected at appropriate intervals to assure their operability.

Consequently, in the summer of 1975 we requested all light water cooled reactor facility licensees to propose changes to their Technical Specifications that would provide assurance of satisfactory snubber performance and reliability. We provided licensees with model Technical Specifications for guidance. After receiving proposals and comments from licensees, and further consideration by ourselves, we revised the model Technical Specifications to provide some relaxation and clarification of the requirements. Then in December, 1975 we requested licensees to amend their proposals to incorporate the revised model Technical Specifications. WEPCO responded to our request on February 18, 1976. Our evaluation of their February 18 proposal follows.

Evaluation

The proposed changes would add new limiting conditions for operation and surveillance requirements for safety related snubbers to the Technical Specifications. The specifications would require that safety related snubbers be operable during reactor operation and prior to startup. Because snubber protection is required only during relatively low probability events, a period of 72 hours is allowed for repair or replacement of defective units before the reactor must be shut down.

In addition, a surveillance program would be specified to provide assurance that the snubbers remain operable. The inspection frequency is based upon maintaining a constant level of snubber protection. Thus the required inspection interval, as modified by the staff and concurred in by the licensee, varies inversely with the observed snubber failures. The longest inspection interval allowed in the Technical Specifications after a record of no snubber failures has been established is nominally 18 months. Experience at operating facilities has shown that the specified surveillance program should provide an acceptable level of snubber performance.

Based on our review of the proposed additions to the Technical Specifications, as modified by the staff and concurred in by the licensee, we have concluded that they would increase the probability of successful snubber performance, and would thereby increase reactor safety; and thus, are acceptable.

Furthermore, we have determined that this amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental statement, negative declaration, or environmental appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: June 18, 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-266 AND 50-301

WISCONSIN ELECTRIC POWER COMPANY
WISCONSIN MICHIGAN POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 17 and 22 to Facility Operating Licenses Nos. DPR-24 and DPR-27 issued to Wisconsin Electric Power Company and Wisconsin Michigan Power Company, which revised Technical Specifications for operation of the Point Beach Nuclear Plant Units Nos. 1 and 2, located in the town of Two Creeks, Manitowoc County, Wisconsin. The amendments are effective as of the date of issuance

The amendments consist of changes in the Technical Specifications that will add new shock suppressor (snubber) limiting conditions for operation and surveillance requirements.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated February 18, 1976, (2) Amendment No. 17 to License No. DPR-24, (3) Amendment No. 22 to License No. DPR-27, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Document Department, University of Wisconsin - Stevens Point Library, Stevens Point, Wisconsin.

A copy of items (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 18th day of June 1976.

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors