



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

January 9, 1976

Dockets Nos. 50-266  
and 50-301

Wisconsin Electric Power Company  
Wisconsin Michigan Power Company  
ATTN: Mr. Sol Burstein  
Executive Vice President  
231 West Michigan Street  
Milwaukee, Wisconsin 53201

Gentlemen:

The Commission has issued the enclosed Amendments Nos. 12 and 16 to Facility Operating Licenses Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Units Nos. 1 and 2. These amendments consist of changes to the Technical Specifications, Appendix A, and is in response to your requests dated February 7, 1975 and July 15, 1975.

These amendments incorporate into the Point Beach Nuclear Plant, Units Nos. 1 and 2 Technical Specifications changes to the reporting requirements. Changes to your proposal were necessary to meet our requirements. These have been discussed with your staff. The Technical Specifications are based on Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4.

We request that you use the formats presented in the Appendices to Regulatory Guide 1.16, Revision 4, for reporting operating information and that you report events of the type described under the section "Events of Potential Public Interest". Instructions for using these reporting formats are contained in Regulatory Guide 1.16 (a copy is enclosed for your use), and AEC report OOE-SS-001 titled "Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File" of which you were previously provided a copy. This report is modified by updated instructions dated December 8, 1975 which are enclosed. Copy requirements are summarized in Regulatory Guide 10.1, "Compilation of Reporting Requirements for Persons Subject to NRC Regulations", a copy of which is also enclosed. This guide will assist you in identifying reports that are required by the Commission's regulations set forth in Title 10 Code of Federal Regulations but are not contained in your Technical Specifications.

Please note that we have discontinued the use of separate identifying numbers for changes to technical specifications. Sequential amendment numbers will be continued as in the past.

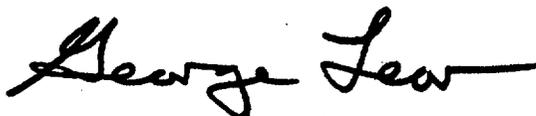
Mr. Sol Burstein

- 2 -

January 9, 1976

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,



George Lear, Chief  
Operating Reactors Branch #3  
Division of Reactor Licensing

Enclosures:

1. Amendment No. 12
2. Amendment No. 16
3. Regulatory Guide 1.16
4. Updated Instructions
5. Regulatory Guide 10.1
6. Safety Evaluation
7. Federal Register Notice

cc: w/enclosures

Mr. Bruce Churchill, Esquire  
Shaw, Pittman, Potts and Trowbridge  
Barr Building  
910 17th Street, N. W.  
Washington, D. C. 20006

Mr. Arthur M. Fish  
Document Department  
University of Wisconsin -  
Stevens Point Library  
Stevens Point, Wisconsin 54481

Mr. William F. Eich, Chairman  
Public Service Commission  
of Wisconsin  
Hill Farms State Office Building  
Madison, Wisconsin 53702



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY  
WISCONSIN MICHIGAN POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 12  
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Wisconsin Electric Power Company and Wisconsin Michigan Power Company (the licensees) dated February 7, 1975 and July 15, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
  - E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.

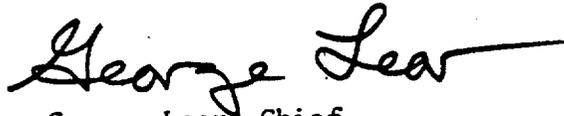
Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B. of Facility Operating License No. DPR-24 is hereby amended to read as follows:

"B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications, as revised".

3. This license amendment is effective 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief  
Operating Reactors Branch #3  
Division of Reactor Licensing

Attachment:  
Changes to the  
Technical Specifications

Date of Issuance: January 9, 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 12

FACILITY OPERATING LICENSE NO. DPR-24

DOCKET NO. 50-266

Replace pages 15.1-1 thru 15.1-5 with the attached revised pages.

Replace pages 15.6.1-1A thru 15.6.6-10 with the attached revised pages.

Replace pages 15.6.7-1 thru 15.6.7-3 with the attached renumbered pages 15.6.6-1 thru 15.6.6-3, respectively.

## TABLE OF CONTENTS

<u>Section</u>	<u>Title</u>	<u>Page</u>
15	TECHNICAL SPECIFICATIONS AND BASES	
15.1	Definitions	15.1-1
15.2 0	Safety Limits and Limiting Safety System Settings	15.2.1-1
15.2.1	Safety Limit, Reactor Core	15.2.1-1
15.2.2	Safety Limit, Reactor Coolant System Pressure	15.2.2-1
15.2.3	Limiting Safety System Settings, Protective Instrumentation	15.2.3-1
15.3	Limiting Conditions for Operation	15.3-1
15.3.1	Reactor Coolant System	15.3.1-1
15.3.2	Chemical and Volume Control System	15.3.2-1
15.3.3	Emergency Core Cooling System, Auxiliary Cooling Systems, Air Recirculation Fan Coolers, and Containment Spray	15.3.3-1
15.3.4	Steam and Power Conversion System	15.3.4-1
15.3.5	Instrumentation System	15.3.5-1
15.3.6	Containment System	15.3.6-1
15.3.7	Auxiliary Electrical Systems	15.3.7-1
15.3.8	Refueling	15.3.8-1
15.3.9	Effluent Releases	15.3.9-1
15.3.10	Control Rod and Power Distribution Limits	15.3.10-1
15.3.11	Movable In-Core Instrumentation	15.3.11-1
15.3.12	Control Room Emergency Filtration	15.3.12-1
15.4	Surveillance Requirements	15.4-1
15.4.1	Operational Safety Review	15.4.1-1
15.4.2	In-Service Inspection of Primary System Components	15.4.2-1
15.4.3	Primary System Testing Following Opening	15.4.3-1
15.4.4	Containment Tests	15.4.4-1
15.4.5	Emergency Core Cooling System and Containment Cooling System Tests	15.4.5-1
15.4.6	Emergency Power System Periodic Tests	15.4.6-1
15.4.7	Main Steam Stop Valves	15.4.7-1
15.4.8	Auxiliary Feedwater System	15.4.8-1
15.4.9	Reactivity Anomalies	15.4.9-1
15.4.10	Operational Environmental Monitoring	15.4.10-1
15.4.11	Control Room Emergency Filtration	15.4.11-1
15.5	Design Features	15.5.1-1
15.5.1	Site	15.5.1-1
15.5.2	Containment	15.5.2-1
15.5.3	Reactor	15.5.3-1
15.5.4	Fuel Storage	15.5.4-1
15.6	Administrative Controls	15.6.1-1
15.6.1	Organization, Review and Audit	15.6.1-1
15.6.2	Action to be Taken if a Safety Limit is Exceeded	15.6.2-1
15.6.3	Plant Operating Procedures	15.6.3-1
15.6.4	Plant Operating Records	15.6.4-1
15.6.5	Plant Reporting Requirements	15.6.5-1
15.6.6	Respiratory Protection for Airborne Radioactivity	15.6.6-1

15. TECHNICAL SPECIFICATIONS AND BASES

15.1 DEFINITIONS

The definitions for frequently used terms which are applied to the Point Beach Nuclear Power Plant, Units No. 1 and 2, are stated below.

a. DELETED

b. Quadrant Power Tilt

Quadrant to average power tilt is expressed in percent as defined by the following equation:

$$100 \times \left( \frac{\text{power in any core quadrant}}{\text{average for all quadrants}} - 1 \right)$$

c. Operable

A system or component is operable when it is capable of performing its intended function within the required range. The system or component shall be considered to have this capability when: (1) it satisfies the Limiting Conditions for Operation defined in Section 15.3, and (2) it has been tested periodically in accordance with Section 15.4 and has met its performance requirements.

d. Containment Integrity\*

Containment integrity is defined to exist when:

- 1) All non-automatic containment isolation valves and blind flanges are closed as required.
- 2) The equipment hatch is properly closed.
- 3) At least one door in each personnel air lock is properly closed.
- 4) All automatic containment isolation valves are operable or are secured closed.
- 5) The uncontrolled containment leakage satisfies Specification 15.4.4.

e. Protective Instrumentation Logic

1) Analog Channel

An analog channel is an arrangement of components and modules as required to generate a single protective action signal when required by a plant condition. An analog channel loses its identity where single action signals are combined.

---

\* Containment isolation valves are discussed in FFDSAR Section 5.2.

5. Qualifications with regard to educational background, experience and technical specialties of the key supervisory personnel listed below shall be maintained in accordance with the levels described in ANSI N18.1-1971 "Selection and Training of Personnel For Nuclear Power Plants".

- a. Manager - Nuclear Power Division (Resident Superintendent)
- b. Maintenance Superintendent
- c. Operations Superintendent
- d. Assistant to the Operations Superintendent
- e. Reactor Engineer
- f. Radiochemical Engineer
- g. Instrument and Control Engineer
- h. Health Physicist
- i. Assistant to the Manager

6. Retraining and replacement training of plant personnel shall be in accordance with Section 5.5 of ANSI N18.1 - 1971 "Selection and Training of Personnel for Nuclear Power Plants".

Organizational units for the review and audit of plant operations shall be comprised of both on-site and off-site personnel and shall have responsibility and authority as outlined below:

1. Duty and Call Superintendents

To assist and counsel the Shift Supervisor in the event of reportable occurrences, a Duty and Call Superintendent Group has been established. The Duty and Call Superintendent Group shall consist of any qualified person designated by the Manager - Nuclear Power Division. In the

event of a reportable occurrence, the Shift Supervisor shall communicate with at least one Duty and Call Superintendent before taking other than the immediate on-the-spot action required. One Duty and Call Superintendent will be assigned to be "on call" at all times. The Duty and Call Superintendent provides continuously available counsel, call out backup, and review to the Shift Supervisor.

2. Manager's Supervisory Staff

On-site review of plant operations will be conducted by the Manager's Supervisory Staff (hereinafter called Supervisory Staff). The Supervisory Staff is made up of the Manager - Nuclear Power Division, Assistant to the Manager - Nuclear Power Division, Operations' Superintendent, Maintenance Superintendent, Instrument and Control Engineer, Reactor Engineer, Radiochemical Engineer, and Health Physicist. The Staff will meet to determine or review action taken with respect to reportable occurrences, records, personnel exposures, and emergency procedures. Records of proceedings of staff meetings will be kept.

a. Frequency of meetings: At least monthly or as required by the Manager - Nuclear Power Division.

b. Quorum: Chairman plus four members.

c. Responsibilities:

- 1) Review existing and proposed normal, abnormal and emergency operating procedures. Review maintenance procedures and proposed changes to these procedures and other procedures or changes thereto as determined by the Manager to affect plant operational safety. (Re: Section 15.6.4 for area of review).
- 2) Review all proposed tests and experiments related to Safety and the results thereof when applicable.
- 3) Review all proposed changes to Technical Specifications.

- 4) Review all proposed changes or modifications to plant systems or equipment where changes would require a change in operating or emergency procedures.
- 5) Periodically review plant operations for industrial and nuclear safety hazards.
- 6) Investigate violations or suspected violations of Technical Specifications, such investigations to include reports, evaluations, and recommendations to prevent recurrence, to the Vice President - Nuclear Plant and to the Chairman of the Off-Site Review Committee.
- 7) Perform special reviews and investigations and prepare reports thereon as requested by the Chairman of the Off-Site Review Committee.
- 8) Investigate, review, and report on all reportable occurrences.
- 9) Cause to be conducted periodic drills on emergency procedures, including evacuation (partial or complete) of the site and check adequacy of communications with off-site support groups.

d. Authority

- 1) The Supervisory Staff shall serve as advisory to the Manager - Nuclear Power Division.
- 2) The Supervisory Staff shall recommend to the Manager approval or disapproval of proposals under items c (1) through (4) above.
  - a) In the event of disagreement between a majority of the Supervisory Staff and decisions by the Manager, the course of action will be determined by the Manager and the disagreement recorded in the Staff minutes. Records of the disagreement

will be included in the minutes sent for review to the Off-Site Review Committee and the Vice President - Nuclear Plant.

- 3) The Supervisory Staff shall make tentative recommendations as to whether or not proposals considered by the Staff involve unreviewed safety questions. These recommendations shall be subject to review and further recommendations by the Off-Site Review Committee. Minutes shall be kept of all meetings of the Staff and copies shall be sent to the Vice President - Nuclear Plant and to the Chairman of the Off-Site Review Committee.
- 4) The Supervisory Staff shall cause to be prepared and shall approve the contents of a report for each reportable occurrence. This report shall include an evaluation of the cause of the occurrence and recommendations for appropriate action to prevent or reduce the possibility of a recurrence. Copies of all such reports shall be submitted to the Vice President - Nuclear Plant and to the Chairman of the Off-Site Review Committee for review.

### 3. Off-Site Review Committee

#### a. Membership

The Off-Site Review Committee is made up of a minimum of five regular members appointed by the Vice-President - Nuclear Plant and one ex-officio member. Of the five or more regular members, at least two will be persons not directly employed by WEPCo or WMPCo. All members will be experienced in one or more aspects of the nuclear industry. The ex-officio member will be the Vice President - Nuclear Plant of Wisconsin Michigan Power Company. The five regular members are presently as follows:

Mr. C. W. Fay, Director, Quality Assurance and Technical  
Services Department, Wisconsin Electric Power Company (Chairman)

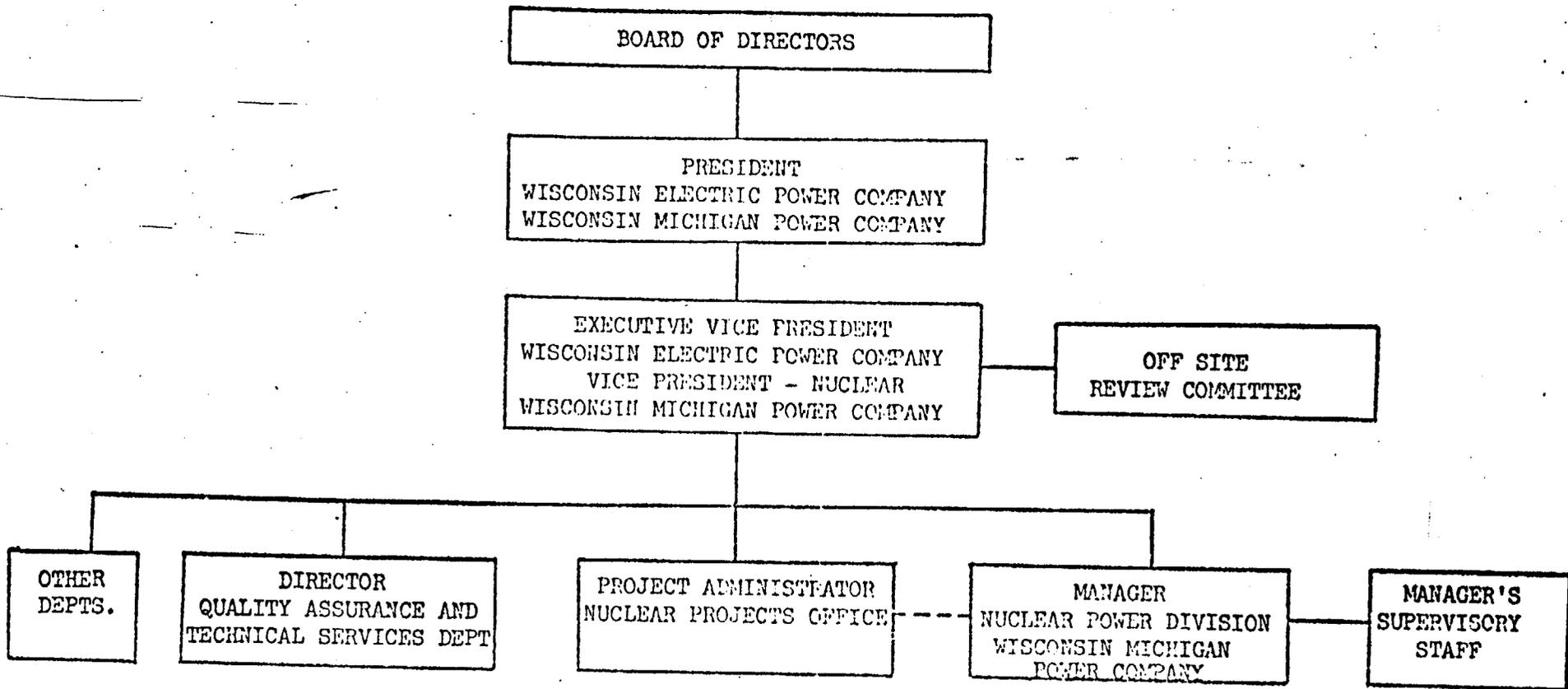
Mr. R. R. Balsbaugh, Assistant Manager - Power Plant Operations  
Wisconsin Electric Power Company

Dr. Ralph Grunewald, University of Wisconsin - Milwaukee -  
Dr. Grunewald is the Company's consultant for site and  
radiological monitoring and health physics.

Mr. G. F. Neils, Nuclear Plant Supervising Engineer,  
Power Production Department, Northern States Power Company

Mr. Lee Hausler, Administrator - Special Nuclear Projects,  
Consumers Power Company

- b. Frequency of Meetings: At least every 6 months or on the call of the Chairman.
- c. Quorum: Chairman plus three members.
- d. Responsibilities:
  - 1) Review proposed changes to the operating license, including Technical Specifications.
  - 2) Review minutes of meetings of the Supervisory Staff to determine if matters considered by the Staff involve unreviewed or unresolved safety questions.
  - 3) Review matters, including proposed changes or modifications to plant systems or equipment, referred to it by the Supervisory Staff or by the Manager.
  - 4) Conduct periodic audits of plant operations.
  - 5) Investigate violations of the Technical Specifications and report findings and recommendations to prevent recurrence to the Vice President - Nuclear Plant.
  - 6) Perform special reviews and investigations and submit reports thereon as requested by the Vice President Nuclear Plant.



NOTES: 1) The Project Administrator-Nuclear Proj Office Provides Services As Requested The Manager-Nuclear Power Division.

MANAGEMENT ORGANIZATION CHART

15.6.2 ACTION TO BE TAKEN IF A SAFETY LIMIT IS EXCEEDED

Specification

- A. If a safety limit is exceeded, the affected reactor shall be shut down and reactor operation shall not be resumed until approval is received from the NRC .
- B. An immediate report shall be made to the Vice President - Nuclear Plant and the Chairman of the Off-site Review Committee.
- C. The Vice President - Nuclear Plant shall report the circumstances to the NRC.
- D. A complete analysis of the circumstances leading to and resulting from the occurrence, together with recommendations to prevent a recurrence, shall be prepared by the Supervisory Staff. This report shall be submitted to the Vice President - Nuclear Plant and the Chairman of the Off-site Review Committee. Appropriate analyses or reports will be submitted to the NRC by the Vice President - Nuclear Plant.

15.6.3 PLANT OPERATING PROCEDURES

Specification

A. The plant will be operated and maintained in accordance with approved procedures. Detailed procedures with appropriate check-off lists and instructions shall be provided for the following conditions involving nuclear safety of the plant:

1. Normal startup, operation, and shutdown of the complete plant and for all systems and components.
2. Refueling operations.
3. Actions to be taken to respond to specific and foreseen potential malfunctions of systems or components, including responses to alarms and abnormal reactivity changes.
4. Emergency conditions involving potential or actual release of radioactivity.
5. Preventative or corrective maintenance operations.
6. Surveillance and testing.

- B. Radiological control procedures shall be written and made available to all station personnel and will state permissible radiation exposure levels. The radiation protection program will meet the requirements of 10CFR20, with the following exceptions:

Paragraph 20.203 - Caution signs, labels, and signals

In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c) (2), each radiation area in which the intensity of radiation is greater than 100 mrem/hr

shall be barricaded and conspicuously posted as a High Radiation Area, and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit. A person or persons permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.

In addition, each High Radiation Area outside the containment building in which the intensity of Radiation is greater than 1000 mrem/hr shall be provided with locked barricades to prevent unauthorized entry into these areas, and the keys to these locked barricades shall be maintained under the administrative control of the Shift Supervisor.

- C. All procedures described in A and B above, and changes thereto, shall be reviewed by the Supervisory Staff and approved by the Manager - Nuclear Power Division prior to implementation, except as provided below.

- D. Temporary changes to procedures which do not change the intent of the original procedure may be made provided such changes are approved by the Shift Supervisor and one of the Duty and Call Superintendents. Such changes shall be documented and subsequently reviewed by the Supervisory Staff, and shall be subject to the approval by the Manager - Nuclear Power Division.
- E. Semiannual drills, including site boundary control center equipment and communications, and annual drills including communications with off-site support groups, shall be conducted to verify the functionality of the Emergency Plans.

#### 15.6.4 PLANT OPERATING RECORDS

##### Specification

Records and logs relative to the following items shall be retained for six (6) years unless a longer period is required by applicable regulations.

- A. Records of normal plant operation, including power levels and periods of operation at each power level.
- B. Records of principal maintenance activities, including inspection, repair, substitution or replacement of principal items of equipment pertaining to nuclear safety.
- C. Records of reportable occurrences.
- D. Records of periodic checks, inspections and calibrations performed to verify that surveillance requirements are being met.
- E.\* Records of new and spent fuel inventory and assembly histories.
- F.\* Records of changes made to the plant and to plant drawings as described in the FFDSAR.
- G.\* Records of plant radiation and contamination surveys.
- H.\* Records of off-site environmental monitoring surveys.
- I.\* Records of radiation exposure of all plant personnel, including all contractor personnel and visitors who enter radiation control areas in the plant.
- J.\* Records of radioactivity levels in liquid and gaseous wastes released to the environment and dilution of these wastes.

15.6.4-1

K. Records of any special reactor tests or experiments.

L. Records of changes made in the Operating Procedures.

\* Starred items will be permanently retained.

Amendment No. 12

Dated: JAN 9 1975

15.6.4-2

## 15.6.5 Plant Reporting Requirement

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following program for reporting of operating information shall be followed. Reports should be addressed to the Director, Directorate of Regulatory Operations, Region III unless otherwise noted.

### 15.6.5.1 Routine Reports

#### A. Startup Report

1. A summary report of plant startup and power escalation testing which addresses each of the tests identified in the FFDSAR and includes a general description of the measured values obtained during the test program and a comparison of these values with design predictions and specifications must be submitted under the following conditions:
  - a. Receipt of an operating license
  - b. Amendment to the license involving a planned increase in power level.
  - c. Installation of fuel that has a different design or has been manufactured by a different fuel supplier.
  - d. Modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.Any corrective actions that were required to obtain satisfactory operation shall also be described.
2. This report shall be submitted within the earliest time frame of the following:

- a. 90 days following completion of the startup tests.
- b. 90 days following resumption or commencement of commercial power operation.
- c. 9 months following initial criticality.

**B. Annual Operating Report**

1. A report covering the operation of the units during the previous calendar year shall be submitted prior to March 1 of each year.
2. The report shall include:
  - a. A narrative summary of operating experience during the report period relating to safe operation of the facility, including safety-related maintenance.
  - b. A description of each outage or forced reduction in power of over 20 percent of design power level where the reduction extends for greater than four hours. This description shall include the proximate cause, system or major component involved, reference to any pertaining abnormal occurrences, corrective action and maintenance undertaken, operating time lost, and a report of any release of radioactivity or radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

15.6.5-2

- c. A tabulation on an annual basis of the number of station, utility, and other personnel receiving exposures greater than 100 mrem/year and their associated man-rem exposure according to work and job functions. The dose assignments to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20 percent of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.
- d. Indications of failed fuel resulting from irradiated fuel examinations, including eddy current tests, ultrasonic tests, or visual examinations completed during the report period.

C. Monthly Operating Reports

1. Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis.
2. Completed reports shall be sent to the Director of Regulatory Operations, U.S. Nuclear Regulatory Commission, Washington, D.C., 20555 with a copy to RO Region III to arrive no later than the tenth of each month following the calendar month covered by the report.
3. The report format for this information is given in figure 15.6.5.1, 15.6.5-2, and 15.6.5-3.

## 15.6.5.2 Reportable Occurrences

### A.. Prompt Notification with Written Followup

The types of events listed in items 1 through 9 below shall be reported as expeditiously as possible within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director, Regulatory Operations, Region III, or his designate, no later than the first working day following the event. A written followup report must be submitted within two weeks. This written followup report shall include a completed copy of the licensee event report form, and may include additional narrative material to provide complete explanation of the circumstances surrounding the event.

1. Failure of the reactor protection system or other systems subject to limiting safety-system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety-system setting in the technical specifications or failure to complete the required protective function.
2. Operation of the unit or affected systems when any parameter or operation subject to a limiting condition for operation is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.
3. Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

4. Reactivity anomalies, involving disagreement with the predicted value of reactivity balance under steady state conditions during power operation, greater than or equal to  $1\% \Delta k/k$ ; a calculated reactivity balance indicating a shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if sub-critical, an unplanned reactivity insertion of more than  $0.5\% \Delta k/k$  or occurrence of any unplanned criticality.
5. Failure or malfunction of one or more components which prevents, or could prevent by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the FFDSAR.
6. Personnel error or procedural inadequacy which prevents, or could prevent by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the FFDSAR.
7. Conditions arising from natural or manmade events that, as a direct result of the event, require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.
8. Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.

9. Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than that assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

B. Thirty-Day Written Reports

The types of events listed in item 1 through 4 below have lesser immediate importance. These events shall be the subject of written reports to the Director, Regulatory Operations, Region III within 30 days of the occurrence of the event. The written report shall include, as a minimum, a completed copy of the licensee event report form, and may be supplemented, as needed to provide complete explanation of the circumstances surrounding the event.

1. Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.
2. Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

3. Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.
4. Abnormal degradation of systems other than those specified in 15.6.5.2.A.3 above designed to contain radioactive material resulting from the fission process.

#### 15.6.5.2. UNIQUE REPORTING REQUIREMENTS

The following written reports shall be submitted to the Director Division of Reactor Licensing, USAEC:

- A. Each integrated leak test shall be the subject of a summary technical report, including results of the local leak rate tests and isolation valve leak rate tests since the last report. The report shall include analysis and interpretations of the results which demonstrate compliance with specified leak rate limits.
- B. The results of the initial containment structural test shall be reported within three months following completion of the test.
- C. To provide the Commission with added verification of the safety and reliability of pre-pressurized Zircaloy-clad nuclear fuel, a limited program of non-destructive fuel inspections will be conducted at Point Beach Unit No. 1. The program shall consist of a visual inspection (e.g., underwater TV, periscope, or other) of the two lead burnup assemblies during the second and third refueling shutdowns. Any condition observed by this inspection which would lead to unacceptable fuel performance may be the object of an expanded effort. The visual inspection program and,

if indicated, the expanded program will be conducted in addition to that being performed in the Saxton and Cabrera reactors. If another domestic plant which contains pre-pressurized fuel of a similar design reaches the second and third refueling shutdowns first, and if a limited inspection program is or has been performed there, then the program may not have to be performed at Point Beach. However, such action requires approval of the Nuclear Regulatory Commission. The results of these inspections will be reported to the Nuclear Regulatory Commission.

- D. At the end of the first 5 years of the in-service inspection period, a review of the inspection program will be conducted. This review will evaluate the results obtained to date in view of possible modifications to the inspection program. These modifications may increase or decrease surveillance requirements as experience dictates. The results of these evaluations will be reported to the Atomic Energy Commission.
- E. Submission of a report within 60 days after January 1 and after July 1 each year for the six-month period or fraction thereof, ending June 30 and December 31 containing:
  - 1. Information relative to the quantities of liquid, gaseous and solid radioactive effluents released from the facility, and effluent volumes used in maintaining the releases within the limits of 10 CFR 20 shall be provided (summarized on a monthly basis) as follows:

a. Liquid Releases

- (1) Total radioactivity (in curies) released, other than tritium, and average concentration at outfall of discharge canal.
- (2) Total tritium (in curies) discharged, and average concentration at outfall of discharge canal.
- (3) Total volume (in gallons) of liquid waste released into circulating water discharge.
- (4) Total volume (in gallons) of dilution water used.
- (5) The maximum concentration released (averaged over the period of a single release).
- (6) Estimated total radioactivity (in curies) released, by nuclide (other than tritium), based on representative isotopic analyses performed.
- (7) Percent of applicable limits released, based on nuclide identification performed.

b. Gaseous Releases

- (1) Total radioactivity (in curies) released of:
  - (a) Noble gases.
  - (b) Halogens.
  - (c) Particulates with half-lives greater than eight days.
  - (d) Tritium oxide.

- (2) Maximum release rate (for any one-hour period).
- (3) Estimated total radioactivity (in curies) released by nuclide (other than tritium, based on representative isotopic analyses performed).
- (4) Percent of applicable limit released, based on nuclide identification performed.

c. Solid Waste

- (1) The total amount of solid waste packaged (in cubic feet).
- (2) Estimated total radioactivity (in curies) involved.
- (3) The dates of shipment and disposition (if shipped off-site).

2. New and Spent Fuel Receipts and Shipments

- a. Number and type of new fuel assemblies received during the reporting period, if any.
- b. Number of spent fuel assemblies shipped off site during the reporting period, if any.

3. Environmental Monitoring

- a. A narrative summary of pertinent environmental monitoring activities performed during the reporting period, including:
  - (1) The number and types of samples taken and the measurements made on the samples; e.g., gross beta gamma scan, etc.
  - (2) Any changes made in sample types or locations during the reporting period, and criteria for these changes.
- b. A summary of survey results during the reporting period.

UNIT \_\_\_\_\_  
 DATE \_\_\_\_\_  
 COMPLETED BY \_\_\_\_\_  
 DOCKET NO. \_\_\_\_\_

**OPERATING STATUS**

1. REPORTING PERIOD \_\_\_\_\_ THROUGH \_\_\_\_\_  
 HOURS IN REPORTING PERIOD: \_\_\_\_\_
2. CURRENTLY AUTHORIZED POWER LEVEL (MWh) \_\_\_\_\_ MAX. DEPENDABLE CAPACITY (MWe-NET) \_\_\_\_\_
3. LOWEST POWER LEVEL TO WHICH SPECIFICALLY RESTRICTED (IF ANY) (MWe-NET): \_\_\_\_\_
4. REASONS FOR RESTRICTION (IF ANY): \_\_\_\_\_

	THIS REPORTING PERIOD	YR TO DATE	CUMULATIVE TO DATE
5. HOURS REACTOR WAS CRITICAL	_____	_____	_____
6. REACTOR RESERVE SHUTDOWN HOURS	_____	_____	_____
7. HOURS GENERATOR ON LINE	_____	_____	_____
8. UNIT RESERVE SHUTDOWN HOURS	_____	_____	_____
9. GROSS THERMAL ENERGY GENERATED (MWH)	_____	_____	_____
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	_____	_____	_____
11. NET ELECTRICAL ENERGY GENERATED (MWH)	_____	_____	_____
12. REACTOR AVAILABILITY FACTOR (1)	_____	_____	_____
13. UNIT AVAILABILITY FACTOR (2)	_____	_____	_____
14. UNIT CAPACITY FACTOR (3)	_____	_____	_____
15. UNIT FORCED OUTAGE RATE (4)	_____	_____	_____
16. SHUTDOWNS SCHEDULED TO BEGIN IN NEXT 6 MONTHS (STATE TYPE, DATE, AND DURATION OF EACH):	_____		

17. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP \_\_\_\_\_
18. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION) REPORT THE FOLLOWING

	DATE LAST FORECAST	DATE ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICAL POWER GENERATION	_____	_____
COMMERCIAL OPERATION	_____	_____

- (1) REACTOR AVAILABILITY FACTOR =  $\frac{\text{HOURS REACTOR WAS CRITICAL}}{\text{HOURS IN REPORTING PERIOD}} \times 100$
- (2) UNIT AVAILABILITY FACTOR =  $\frac{\text{HOURS GENERATOR ON LINE}}{\text{HOURS IN REPORTING PERIOD}} \times 100$
- (3) UNIT CAPACITY FACTOR =  $\frac{\text{NET ELECTRICAL POWER GENERATED}}{\text{MAX. DEPENDABLE CAPACITY (MWe-NET) X HOURS IN REPORTING PERIOD}}$
- (4) UNIT FORCED OUTAGE RATE =  $\frac{\text{FORCED OUTAGE HOURS}}{\text{HOURS GENERATOR ON LINE + FORCED OUTAGE HOURS}} \times 100$

Figure 15.6.5.1-1

UNIT SHUTDOWNS

DOCKET NO. \_\_\_\_\_

UNIT NAME \_\_\_\_\_

DATE \_\_\_\_\_

COMPLETED BY \_\_\_\_\_

REPORT MONTH \_\_\_\_\_

NO.	DATE	TYPE F-FORCED S-SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTIONS/COMMENTS
<p>(1) REASON</p> <ul style="list-style-type: none"> <li>A EQUIPMENT FAILURE (EXPLAIN)</li> <li>B MAINT. OR TEST</li> <li>C REFUELING</li> <li>D REGULATORY RESTRICTION</li> <li>E OPERATOR TRAINING AND LICENSE EXAMINATION</li> <li>F ADMINISTRATIVE</li> <li>G OPERATIONAL ERROR (EXPLAIN)</li> <li>H OTHER (EXPLAIN)</li> </ul> <p>(2) METHOD</p> <ul style="list-style-type: none"> <li>1 MANUAL</li> <li>2 MANUAL SCRAM</li> <li>3 AUTOMATIC SCRAM</li> </ul>						

SUMMARY:

Amendment No. 12

Figure 15.6.5.1-2

Dated: JAN 9 1975

DOCKET NO. \_\_\_\_\_

UNIT \_\_\_\_\_

DATE \_\_\_\_\_

COMPLETED BY \_\_\_\_\_

### AVERAGE DAILY UNIT POWER LEVEL

MONTH \_\_\_\_\_

DAY	AVERAGE DAILY POWER LEVEL (MWe-net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-net)
1	_____	17	_____
2	_____	18	_____
3	_____	19	_____
4	_____	20	_____
5	_____	21	_____
6	_____	22	_____
7	_____	23	_____
8	_____	24	_____
9	_____	25	_____
10	_____	26	_____
11	_____	27	_____
12	_____	28	_____
13	_____	29	_____
14	_____	30	_____
15	_____	31	_____
16	_____		

#### DAILY UNIT POWER LEVEL FORM INSTRUCTIONS

On this form, list the average daily unit power level in MWe-net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that by using maximum dependable capacity for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

## 15.6.6 RESPIRATORY PROTECTION FOR AIRBORNE RADIOACTIVITY

### Applicability

This section applies to protection of plant personnel from exposure to excessive airborne radioactivity in accordance with provisions of 10CFR20, Section 20.103.

### Objective

To document the requirements for a respiratory protection program pursuant to 10CFR20.103, subparagraph (c)(3).

### Specification

- A. Personnel shall use appropriate respiratory equipment when required by airborne radioactivity concentrations in working areas, to insure against inhalation, absorption or ingestion of radioactive material in excess of that permitted under 10CFR20, Section 20.103.<sup>(1)</sup>
- B. Credit for respirator protection factors shall be taken as outlined in Table 15.6.7-1.
- C. Persons using respirators shall be relieved at reasonable intervals such that the use of respiratory protection equipment will not endanger personnel or discourage observance of proper work and safety procedures.
- D. A respiratory protection program, described in the FFDSAR, Section 11.2.3, shall be maintained to insure that the objectives of this specification are met.
- E. This section shall be superseded by adoption of proposed changes to 10CFR20, Section 103 which would make this specification unnecessary.

### References

- (1) PSAR Section 11.2

Table 15.6.6-1

Protection Factors for Respirators

<u>Description</u>	<u>(1) Modes</u>	<u>(2) Protection Factors</u>	
		<u>(3) Particulates, Vapors and Gases Except Tritium Oxide</u>	<u>Tritium Oxide</u>
<b>I. <u>Air-Purifying Respirators</u></b>			
Facepiece, Half Mask		10	1
Facepiece, Full		100	1
<b>II. <u>Atmosphere-Supplying Respirators</u></b>			
<b>A. <u>Air Line Respirator</u></b>			
Facepiece, Full	CF	1000	2
Facepiece, Full	PD	1000	2
Hood	CF	1000	2
Suit	CF	(4)	(4)
<b>B. <u>Self-Contained Breathing Apparatus</u></b>			
	PD	1000	2
<b>III. <u>Combination Respirator</u></b>			
Any combination of air purifying and atmosphere supplying respirator.		Protection factor for type and mode of operation as listed above	

- (1) CF: Continuous Flow  
 PD: Pressure Demand (i.e., Always Positive Pressure)

- (2) (a) For purposes of this authorization, the protection factor is a measure of the degree of protection afforded by a respirator, defined as the ratio of the concentration of airborne radioactive material outside the respiratory protective equipment to that inside the equipment (usually inside the facepiece) under conditions of use. It is applied to the airborne concentration to determine the concentration inhaled by the wearer, according to the following formula:

$$\text{Concentration Inhaled} = \frac{\text{Airborne Concentration}}{\text{Protection Factor}}$$

(b) The protection factors apply:

- (i) Only for individually fitted respirators worn by trained individuals and used and maintained under supervision in a well-planned respiratory protection program.
  - (ii) For air-purifying respirators only when high efficiency particulate filters and/or adsorbents appropriate to the hazard are used.
  - (iii) For atmosphere-supplying respirators only when supplied with adequate respirable air.
- (3) Excluding radioactive contaminants that present an absorption or submersive hazard.
- (4) Appropriate protection factors must be determined taking account of the permeability of the suit to the contaminant under conditions of use. No protection factor greater than 1000 shall be used except as authorized by the Commission.

NOTE 1: Protection factors for respirators as may be approved in the future by the U. S. Bureau of Mines according to approval schedules for respirators to protect against airborne radionuclides may be used in lieu of the protection factors listed in this table. Where additional respiratory hazards other than radioactive ones are present, especially those immediately dangerous to life, the selection and use of respirators shall also be governed by the approvals of the U. S. Bureau of Mines in accordance with their applicable schedules.

NOTE 2: Radioactive contaminants for which the concentration values in Appendix E, Table 1, of 10CFR, Part 20, are based on internal dose due to inhalation may, in addition, present external exposure hazards at higher concentrations.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY  
WISCONSIN MICHIGAN POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 16  
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Wisconsin Electric Power Company and Wisconsin Michigan Power Company (the licensees) dated February 7, 1975 and July 15, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
  - E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.

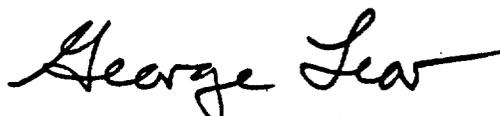
Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B. of Facility Operating License No. DPR-27 is hereby amended to read as follows:

"B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications, as revised".

3. This license amendment is effective 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief  
Operating Reactors Branch #3  
Division of Reactor Licensing

Attachment:  
Changes to the  
Technical Specifications

Date of Issuance: January 9, 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 16

FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NO. 50-301

Replace pages 15.1-1 thru 15.1-5 with the attached revised pages.

Replace pages 15.6.1-1A thru 15.6.6-10 with the attached revised pages.

Replace pages 15.6.7-1 thru 15.6.7-3 with the attached renumbered pages 15.6.6-1 thru 15.6.6-3, respectively.

TABLE OF CONTENTS

<u>Section</u>	<u>Title</u>	<u>Page</u>
15	TECHNICAL SPECIFICATIONS AND BASES	
15.1	Definitions	15.1-1
15.2.0	Safety Limits and Limiting Safety System Settings	15.2.1-1
15.2.1	Safety Limit, Reactor Core	15.2.1-1
15.2.2	Safety Limit, Reactor Coolant System Pressure	15.2.2-1
15.2.3	Limiting Safety System Settings, Protective Instrumentation	15.2.3-1
15.3	Limiting Conditions for Operation	15.3-1
15.3.1	Reactor Coolant System	15.3.1-1
15.3.2	Chemical and Volume Control System	15.3.2-1
15.3.3	Emergency Core Cooling System, Auxiliary Cooling Systems, Air Recirculation Fan Coolers, and Containment Spray	15.3.3-1
15.3.4	Steam and Power Conversion System	15.3.4-1
15.3.5	Instrumentation System	15.3.5-1
15.3.6	Containment System	15.3.6-1
15.3.7	Auxiliary Electrical Systems	15.3.7-1
15.3.8	Refueling	15.3.8-1
15.3.9	Effluent Releases	15.3.9-1
15.3.10	Control Rod and Power Distribution Limits	15.3.10-1
15.3.11	Movable In-Core Instrumentation	15.3.11-1
15.3.12	Control Room Emergency Filtration	15.3.12-1
15.4	Surveillance Requirements	15.4-1
15.4.1	Operational Safety Review	15.4.1-1
15.4.2	In-Service Inspection of Primary System Components	15.4.2-1
15.4.3	Primary System Testing Following Opening	15.4.3-1
15.4.4	Containment Tests	15.4.4-1
15.4.5	Emergency Core Cooling System and Containment Cooling System Tests	15.4.5-1
15.4.6	Emergency Power System Periodic Tests	15.4.6-1
15.4.7	Main Steam Stop Valves	15.4.7-1
15.4.8	Auxiliary Feedwater System	15.4.8-1
15.4.9	Reactivity Anomalies	15.4.9-1
15.4.10	Operational Environmental Monitoring	15.4.10-1
15.4.11	Control Room Emergency Filtration	15.4.11-1
15.5	Design Features	15.5.1-1
15.5.1	Site	15.5.1-1
15.5.2	Containment	15.5.2-1
15.5.3	Reactor	15.5.3-1
15.5.4	Fuel Storage	15.5.4-1
15.6	Administrative Controls	15.6.1-1
15.6.1	Organization, Review and Audit	15.6.1-1
15.6.2	Action to be Taken if a Safety Limit is Exceeded	15.6.2-1
15.6.3	Plant Operating Procedures	15.6.3-1
15.6.4	Plant Operating Records	15.6.4-1
15.6.5	Plant Reporting Requirements	15.6.5-1
15.6.6	Respiratory Protection for Airborne Radioactivity	15.6.6-1

15. TECHNICAL SPECIFICATIONS AND BASES

15.1 DEFINITIONS

The definitions for frequently used terms which are applied to the Point Beach Nuclear Power Plant, Units No. 1 and 2, are stated below.

a. DELETED

b. Quadrant Power Tilt

Quadrant to average power tilt is expressed in percent as defined by the following equation:

$$100 \times \left( \frac{\text{power in any core quadrant}}{\text{average for all quadrants}} - 1 \right)$$

c. Operable

A system or component is operable when it is capable of performing its intended function within the required range. The system or component shall be considered to have this capability when: (1) it satisfies the Limiting Conditions for Operation defined in Section 15.3, and (2) it has been tested periodically in accordance with Section 15.4 and has met its performance requirements.

d. Containment Integrity\*

Containment integrity is defined to exist when:

- 1) All non-automatic containment isolation valves and blind flanges are closed as required.
- 2) The equipment hatch is properly closed.
- 3) At least one door in each personnel air lock is properly closed.
- 4) All automatic containment isolation valves are operable or are secured closed.
- 5) The uncontrolled containment leakage satisfies Specification 15.4.4.

e. Protective Instrumentation Logic

1) Analog Channel

An analog channel is an arrangement of components and modules as required to generate a single protective action signal when required by a plant condition. An analog channel loses its identity where single action signals are combined.

---

\* Containment isolation valves are discussed in FFDSAR Section 5.2.

5. Qualifications with regard to educational background, experience and technical specialties of the key supervisory personnel listed below shall be maintained in accordance with the levels described in ANSI N18.1-1971 "Selection and Training of Personnel For Nuclear Power Plants".
  - a. Manager - Nuclear Power Division (Resident Superintendent)
  - b. Maintenance Superintendent
  - c. Operations Superintendent
  - d. Assistant to the Operations Superintendent
  - e. Reactor Engineer
  - f. Radiochemical Engineer
  - g. Instrument and Control Engineer
  - h. Health Physicist
  - i. Assistant to the Manager
6. Retraining and replacement training of plant personnel shall be in accordance with Section 5.5 of ANSI N18.1 - 1971 "Selection and Training of Personnel for Nuclear Power Plants".

Organizational units for the review and audit of plant operations shall be comprised of both on-site and off-site personnel and shall have responsibility and authority as outlined below:

1. Duty and Call Superintendents

To assist and counsel the Shift Supervisor in the event of reportable occurrences, a Duty and Call Superintendent Group has been established. The Duty and Call Superintendent Group shall consist of any qualified person designated by the Manager - Nuclear Power Division. In the

event of a reportable occurrence, the Shift Supervisor shall communicate with at least one Duty and Call Superintendent before taking other than the immediate on-the-spot action required. One Duty and Call Superintendent will be assigned to be "on call" at all times. The Duty and Call Superintendent provides continuously available counsel, call out backup, and review to the Shift Supervisor.

2. Manager's Supervisory Staff

On-site review of plant operations will be conducted by the Manager's Supervisory Staff (hereinafter called Supervisory Staff). The Supervisory Staff is made up of the Manager - Nuclear Power Division, Assistant to the Manager - Nuclear Power Division, Operations Superintendent, Maintenance Superintendent, Instrument and Control Engineer, Reactor Engineer, Radiochemical Engineer, and Health Physicist. The Staff will meet to determine or review action taken with respect to reportable occurrences, records, personnel exposures, and emergency procedures. Records of proceedings of staff meetings will be kept.

- a. Frequency of meetings: At least monthly or as required by the Manager - Nuclear Power Division.
- b. Quorum: Chairman plus four members.
- c. Responsibilities:
  - 1) Review existing and proposed normal, abnormal and emergency operating procedures. Review maintenance procedures and proposed changes to these procedures and other procedures or changes thereto as determined by the Manager to affect plant operational safety. (Re: Section 15.6.4 for area of review).
  - 2) Review all proposed tests and experiments related to Safety and the results thereof when applicable.
  - 3) Review all proposed changes to Technical Specifications.

- 4) Review all proposed changes or modifications to plant systems or equipment where changes would require a change in operating or emergency procedures.
- 5) Periodically review plant operations for industrial and nuclear safety hazards.
- 6) Investigate violations or suspected violations of Technical Specifications, such investigations to include reports, evaluations, and recommendations to prevent recurrence, to the Vice President - Nuclear Plant and to the Chairman of the Off-Site Review Committee.
- 7) Perform special reviews and investigations and prepare reports thereon as requested by the Chairman of the Off-Site Review Committee.
- 8) Investigate, review, and report on all reportable occurrences.
- 9) Cause to be conducted periodic drills on emergency procedures, including evacuation (partial or complete) of the site and check adequacy of communications with off-site support groups.

d. Authority

- 1) The Supervisory Staff shall serve as advisory to the Manager - Nuclear Power Division.
- 2) The Supervisory Staff shall recommend to the Manager approval or disapproval of proposals under items c (1) through (4) above.
  - a) In the event of disagreement between a majority of the Supervisory Staff and decisions by the Manager, the course of action will be determined by the Manager and the disagreement recorded in the Staff minutes. Records of the disagreement

will be included in the minutes sent for review to the Off-Site Review Committee and the Vice President - Nuclear Plant.

- 3) The Supervisory Staff shall make tentative recommendations as to whether or not proposals considered by the Staff involve unreviewed safety questions. These recommendations shall be subject to review and further recommendations by the Off-Site Review Committee. Minutes shall be kept of all meetings of the Staff and copies shall be sent to the Vice President - Nuclear Plant and to the Chairman of the Off-Site Review Committee.
- 4) The Supervisory Staff shall cause to be prepared and shall approve the contents of a report for each reportable occurrence. This report shall include an evaluation of the cause of the occurrence and recommendations for appropriate action to prevent or reduce the possibility of a recurrence. Copies of all such reports shall be submitted to the Vice President - Nuclear Plant and to the Chairman of the Off-Site Review Committee for review.

### 3. Off-Site Review Committee

#### a. Membership

The Off-Site Review Committee is made up of a minimum of five regular members appointed by the Vice-President - Nuclear Plant and one ex-officio member. Of the five or more regular members, at least two will be persons not directly employed by WEPCo or WMPCo. All members will be experienced in one or more aspects of the nuclear industry. The ex-officio member will be the Vice President - Nuclear Plant of Wisconsin Michigan Power Company. The five regular members are presently as follows:

Mr. C. W. Fay, Director, Quality Assurance and Technical  
Services Department, Wisconsin Electric Power Company (Chairman)

Mr. R. R. Balsbaugh, Assistant Manager - Power Plant Operations  
Wisconsin Electric Power Company

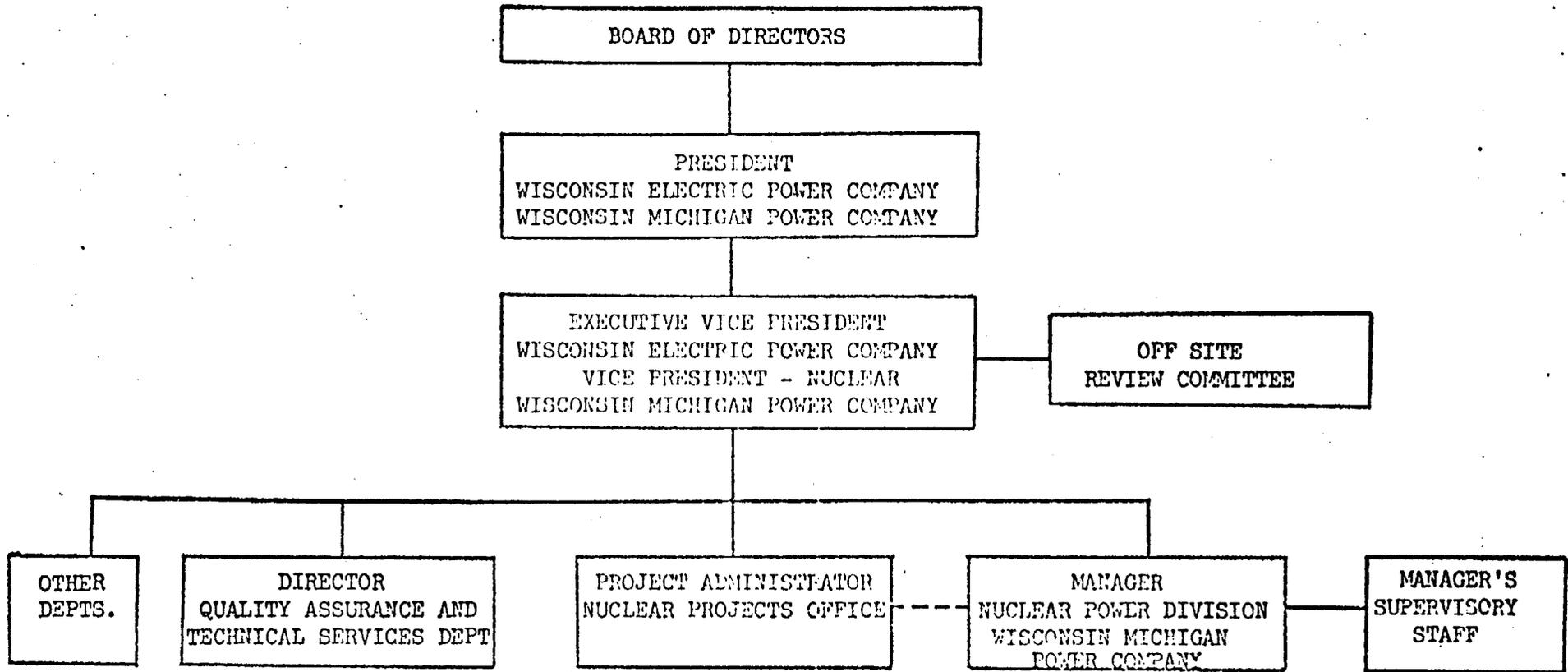
Dr. Ralph Grunewald, University of Wisconsin - Milwaukee -

Dr. Grunewald is the Company's consultant for site and  
radiological monitoring and health physics.

Mr. G. M. Neils, Nuclear Plant Supervising Engineer,  
Power Production Department, Northern States Power Company

Mr. Lee Hausler, Administrator - Special Nuclear Projects,  
Consumers Power Company

- b. Frequency of Meetings: At least every 6 months or on the call of the Chairman.
- c. Quorum: Chairman plus three members.
- d. Responsibilities:
  - 1) Review proposed changes to the operating license, including Technical Specifications.
  - 2) Review minutes of meetings of the Supervisory Staff to determine if matters considered by the Staff involve unreviewed or unresolved safety questions.
  - 3) Review matters, including proposed changes or modifications to plant systems or equipment, referred to it by the Supervisory Staff or by the Manager.
  - 4) Conduct periodic audits of plant operations.
  - 5) Investigate violations of the Technical Specifications and report findings and recommendations to prevent recurrence to the Vice President - Nuclear Plant.
  - 6) Perform special reviews and investigations and submit reports thereon as requested by the Vice President Nuclear Plant.



NOTES: 1) The Project Administrator-Nuclear Project Office Provides Services As Requested By The Manager-Nuclear Power Division.

MANAGEMENT ORGANIZATION CHART

15.6.2 ACTION TO BE TAKEN IF A SAFETY LIMIT IS EXCEEDED

Specification

- A. If a safety limit is exceeded, the affected reactor shall be shut down and reactor operation shall not be resumed until approval is received from the NRC .
- B. An immediate report shall be made to the Vice President - Nuclear Plant and the Chairman of the Off-site Review Committee.
- C. The Vice President - Nuclear Plant shall report the circumstances to the NRC.
- D. A complete analysis of the circumstances leading to and resulting from the occurrence, together with recommendations to prevent a recurrence, shall be prepared by the Supervisory Staff. This report shall be submitted to the Vice President - Nuclear Plant and the Chairman of the Off-site Review Committee. Appropriate analyses or reports will be submitted to the NRC by the Vice President - Nuclear Plant.

15.6.3 PLANT OPERATING PROCEDURES

Specification

A. The plant will be operated and maintained in accordance with approved procedures. Detailed procedures with appropriate check-off lists and instructions shall be provided for the following conditions involving nuclear safety of the plant:

1. Normal startup, operation, and shutdown of the complete plant and for all systems and components.
2. Refueling operations.
3. Actions to be taken to respond to specific and foreseen potential malfunctions of systems or components, including responses to alarms and abnormal reactivity changes.
4. Emergency conditions involving potential or actual release of radioactivity.
5. Preventative or corrective maintenance operations.
6. Surveillance and testing.

- B. Radiological control procedures shall be written and made available to all station personnel and will state permissible radiation exposure levels. The radiation protection program will meet the requirements of 10CFR20, with the following exceptions:

Paragraph 20.203 - Caution signs, labels, and signals

In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c) (2), each radiation area in which the intensity of radiation is greater than 100 mrem/hr

shall be barricaded and conspicuously posted as a High Radiation Area, and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit. A person or persons permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.

In addition, each High Radiation Area outside the containment building in which the intensity of Radiation is greater than 1000 mrem/hr shall be provided with locked barricades to prevent unauthorized entry into these areas, and the keys to these locked barricades shall be maintained under the administrative control of the Shift Supervisor.

- C. All procedures described in A and B above, and changes thereto, shall be reviewed by the Supervisory Staff and approved by the Manager - Nuclear Power Division prior to implementation, except as provided below.

D. Temporary changes to procedures which do not change the intent of the original procedure may be made provided such changes are approved by the Shift Supervisor and one of the Duty and Call Superintendents. Such changes shall be documented and subsequently reviewed by the Supervisory Staff, and shall be subject to the approval by the Manager - Nuclear Power Division.

E. Semiannual drills, including site boundary control center equipment and communications, and annual drills including communications with off-site support groups, shall be conducted to verify the functionality of the Emergency Plans.

#### 15.6.4 PLANT OPERATING RECORDS

##### Specification

Records and logs relative to the following items shall be retained for six (6) years unless a longer period is required by applicable regulations.

- A. Records of normal plant operation, including power levels and periods of operation at each power level.
- B. Records of principal maintenance activities, including inspection, repair, substitution or replacement of principal items of equipment pertaining to nuclear safety.
- C. Records of reportable occurrences.
- D. Records of periodic checks, inspections and calibrations performed to verify that surveillance requirements are being met.
- E.\* Records of new and spent fuel inventory and assembly histories.
- F.\* Records of changes made to the plant and to plant drawings as described in the FFDSAR.
- G.\* Records of plant radiation and contamination surveys.
- H.\* Records of off-site environmental monitoring surveys.
- I.\* Records of radiation exposure of all plant personnel, including all contractor personnel and visitors who enter radiation control areas in the plant.
- J.\* Records of radioactivity levels in liquid and gaseous wastes released to the environment and dilution of these wastes.

15.6.4-1

K. Records of any special reactor tests or experiments.

L. Records of changes made in the Operating Procedures.

\* Starred items will be permanently retained.

## 15.6.5 Plant Reporting Requirement

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following program for reporting of operating information shall be followed. Reports should be addressed to the Director, Directorate of Regulatory Operations, Region III unless otherwise noted.

### 15.6.5.1 Routine Reports

#### A. Startup Report

1. A summary report of plant startup and power escalation testing which addresses each of the tests identified in the FFDSAR and includes a general description of the measured values obtained during the test program and a comparison of these values with design predictions and specifications must be submitted under the following conditions:
  - a. Receipt of an operating license
  - b. Amendment to the license involving a planned increase in power level.
  - c. Installation of fuel that has a different design or has been manufactured by a different fuel supplier.
  - d. Modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.Any corrective actions that were required to obtain satisfactory operation shall also be described.
2. This report shall be submitted within the earliest time frame of the following:

- a. 90 days following completion of the startup tests.
- b. 90 days following resumption or commencement of commercial power operation.
- c. 9 months following initial criticality.

**B. Annual Operating Report**

1. A report covering the operation of the units during the previous calendar year shall be submitted prior to March 1 of each year.
2. The report shall include:
  - a. A narrative summary of operating experience during the report period relating to safe operation of the facility, including safety-related maintenance.
  - b. A description of each outage or forced reduction in power of over 20 percent of design power level where the reduction extends for greater than four hours. This description shall include the proximate cause, system or major component involved, reference to any pertaining abnormal occurrences, corrective action and maintenance undertaken, operating time lost, and a report of any release of radioactivity or radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

- c. A tabulation on an annual basis of the number of station, utility, and other personnel receiving exposures greater than 100 mrem/year and their associated man-rem exposure according to work and job functions. The dose assignments to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20 percent of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.
- d. Indications of failed fuel resulting from irradiated fuel examinations, including eddy current tests, ultrasonic tests, or visual examinations completed during the report period.

C. Monthly Operating Reports

1. Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis.
2. Completed reports shall be sent to the Director of Regulatory Operations, U.S. Nuclear Regulatory Commission, Washington, D.C., 20555 with a copy to RO Region III to arrive no later than the tenth of each month following the calendar month covered by the report.
3. The report format for this information is given in figure 15.6.5.1, 15.6.5-2, and 15.6.5-3.

## 15.6.5.2 Reportable Occurrences

### A.. Prompt Notification with Written Followup

The types of events listed in items 1 through 9 below shall be reported as expeditiously as possible within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director, Regulatory Operations, Region III, or his designate, no later than the first working day following the event. A written followup report must be submitted within two weeks. This written followup report shall include a completed copy of the licensee event report form, and may include additional narrative material to provide complete explanation of the circumstances surrounding the event.

1. Failure of the reactor protection system or other systems subject to limiting safety-system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety-system setting in the technical specifications or failure to complete the required protective function.
2. Operation of the unit or affected systems when any parameter or operation subject to a limiting condition for operation is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.
3. Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

4. Reactivity anomalies, involving disagreement with the predicted value of reactivity balance under steady state conditions during power operation, greater than or equal to 1%  $k/k$ ; a calculated reactivity balance indicating a shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if sub-critical, an unplanned reactivity insertion of more than 0.5%  $\Delta k/k$  or occurrence of any unplanned criticality.
5. Failure or malfunction of one or more components which prevents, or could prevent by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the FFDSAR.
6. Personnel error or procedural inadequacy which prevents, or could prevent by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the FFDSAR.
7. Conditions arising from natural or manmade events that, as a direct result of the event, require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.
8. Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.

9. Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than that assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

B. Thirty-Day Written Reports

The types of events listed in item 1 through 4 below have lesser immediate importance. These events shall be the subject of written reports to the Director, Regulatory Operations, Region III within 30 days of the occurrence of the event. The written report shall include, as a minimum, a completed copy of the licensee event report form, and may be supplemented, as needed to provide complete explanation of the circumstances surrounding the event.

1. Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.
2. Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

3. Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.
4. Abnormal degradation of systems other than those specified in 15.6.5.2.A.3 above designed to contain radioactive material resulting from the fission process.

#### 15.6.5.2. UNIQUE REPORTING REQUIREMENTS

The following written reports shall be submitted to the Director Division of Reactor Licensing, USAEC:

- A. Each integrated leak test shall be the subject of a summary technical report, including results of the local leak rate tests and isolation valve leak rate tests since the last report. The report shall include analysis and interpretations of the results which demonstrate compliance with specified leak rate limits.
- B. The results of the initial containment structural test shall be reported within three months following completion of the test.
- C. To provide the Commission with added verification of the safety and reliability of pre-pressurized Zircaloy-clad nuclear fuel, a limited program of non-destructive fuel inspections will be conducted at Point Beach Unit No. 1. The program shall consist of a visual inspection (e.g., underwater TV, periscope, or other) of the two lead burnup assemblies during the second and third refueling shutdowns. Any condition observed by this inspection which would lead to unacceptable fuel performance may be the object of an expanded effort. The visual inspection program and,

if indicated, the expanded program will be conducted in addition to that being performed in the Saxton and Cabrera reactors. If another domestic plant which contains pre-pressurized fuel of a similar design reaches the second and third refueling shutdowns first, and if a limited inspection program is or has been performed there, then the program may not have to be performed at Point Beach. However, such action requires approval of the Nuclear Regulatory Commission. The results of these inspections will be reported to the Nuclear Regulatory Commission.

- D. At the end of the first 5 years of the in-service inspection period, a review of the inspection program will be conducted. This review will evaluate the results obtained to date in view of possible modifications to the inspection program. These modifications may increase or decrease surveillance requirements as experience dictates. The results of these evaluations will be reported to the Atomic Energy Commission.
- E. Submission of a report within 60 days after January 1 and after July 1 each year for the six-month period or fraction thereof, ending June 30 and December 31 containing:
  - 1. Information relative to the quantities of liquid, gaseous and solid radioactive effluents released from the facility, and effluent volumes used in maintaining the releases within the limits of 10 CFR 20 shall be provided (summarized on a monthly basis) as follows:

**a. Liquid Releases**

- (1) Total radioactivity (in curies) released, other than tritium, and average concentration at outfall of discharge canal.
- (2) Total tritium (in curies) discharged, and average concentration at outfall of discharge canal.
- (3) Total volume (in gallons) of liquid waste released into circulating water discharge.
- (4) Total volume (in gallons) of dilution water used.
- (5) The maximum concentration released (averaged over the period of a single release).
- (6) Estimated total radioactivity (in curies) released, by nuclide (other than tritium), based on representative isotopic analyses performed.
- (7) Percent of applicable limits released, based on nuclide identification performed.

**b. Gaseous Releases**

- (1) Total radioactivity (in curies) released of:
  - (a) Noble gases.
  - (b) Halogens.
  - (c) Particulates with half-lives greater than eight days.
  - (d) Tritium oxide.

- (2) Maximum release rate (for any one-hour period).
- (3) Estimated total radioactivity (in curies) released by nuclide (other than tritium, based on representative isotopic analyses performed).
- (4) Percent of applicable limit released, based on nuclide identification performed.

c. Solid Waste

- (1) The total amount of solid waste packaged (in cubic feet).
- (2) Estimated total radioactivity (in curies) involved.
- (3) The dates of shipment and disposition (if shipped off-site).

2. New and Spent Fuel Receipts and Shipments

- a. Number and type of new fuel assemblies received during the reporting period, if any.
- b. Number of spent fuel assemblies shipped off site during the reporting period, if any.

3. Environmental Monitoring

- a. A narrative summary of pertinent environmental monitoring activities performed during the reporting period, including:
  - (1) The number and types of samples taken and the measurements made on the samples; e.g., gross beta gamma scan, etc.
  - (2) Any changes made in sample types or locations during the reporting period, and criteria for these changes.
- b. A summary of survey results during the reporting period.

UNIT \_\_\_\_\_

DATE \_\_\_\_\_

COMPLETED BY \_\_\_\_\_

DOCKET NO. \_\_\_\_\_

**OPERATING STATUS**

1. REPORTING PERIOD \_\_\_\_\_ THROUGH \_\_\_\_\_  
HOURS IN REPORTING PERIOD: \_\_\_\_\_
2. CURRENTLY AUTHORIZED POWER LEVEL (MWh) \_\_\_\_\_ MAX. DEPENDABLE CAPACITY (MWe-NET) \_\_\_\_\_
3. LOWEST POWER LEVEL TO WHICH SPECIFICALLY RESTRICTED (IF ANY) (MWe-NET): \_\_\_\_\_
4. REASONS FOR RESTRICTION (IF ANY): \_\_\_\_\_

	THIS REPORTING PERIOD	YR TO DATE	CUMULATIVE TO DATE
5. HOURS REACTOR WAS CRITICAL	_____	_____	_____
6. REACTOR RESERVE SHUTDOWN HOURS	_____	_____	_____
7. HOURS GENERATOR ON LINE	_____	_____	_____
8. UNIT RESERVE SHUTDOWN HOURS	_____	_____	_____
9. GROSS THERMAL ENERGY GENERATED (MWH)	_____	_____	_____
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	_____	_____	_____
11. NET ELECTRICAL ENERGY GENERATED (MWH)	_____	_____	_____
12. REACTOR AVAILABILITY FACTOR (1)	_____	_____	_____
13. UNIT AVAILABILITY FACTOR (2)	_____	_____	_____
14. UNIT CAPACITY FACTOR (3)	_____	_____	_____
15. UNIT FORCED OUTAGE RATE (4)	_____	_____	_____
16. SHUTDOWNS SCHEDULED TO BEGIN IN NEXT 6 MONTHS (STATE TYPE, DATE, AND DURATION OF EACH).	_____		

17. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: \_\_\_\_\_
18. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION) REPORT THE FOLLOWING

	DATE LAST FORECAST	DATE ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICAL POWER GENERATION	_____	_____
COMMERCIAL OPERATION	_____	_____

- (1) REACTOR AVAILABILITY FACTOR =  $\frac{\text{HOURS REACTOR WAS CRITICAL}}{\text{HOURS IN REPORTING PERIOD}} \times 100$
- (2) UNIT AVAILABILITY FACTOR =  $\frac{\text{HOURS GENERATOR ON LINE}}{\text{HOURS IN REPORTING PERIOD}} \times 100$
- (3) UNIT CAPACITY FACTOR =  $\frac{\text{NET ELECTRICAL POWER GENERATED}}{\text{MAX. DEPENDABLE CAPACITY (MWe-NET)} \times \text{HOURS IN REPORTING PERIOD}}$
- (4) UNIT FORCED OUTAGE RATE =  $\frac{\text{FORCED OUTAGE HOURS}}{\text{HOURS GENERATOR ON LINE} + \text{FORCED OUTAGE HOURS}} \times 100$

Figure 15.6.5.1-1

# UNIT SHUTDOWNS

DOCKET NO. \_\_\_\_\_

UNIT NAME \_\_\_\_\_

DATE \_\_\_\_\_

COMPLETED BY \_\_\_\_\_

REPORT MONTH \_\_\_\_\_

NO.	DATE	TYPE F-FORCED S-SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTIONS/COMMENTS
						<div style="font-size: small;"> <p>(1) REASON</p> <p>A EQUIPMENT FAILURE (EXPLAIN)</p> <p>B MAINT. OR TEST</p> <p>C REFUELING</p> <p>D REGULATORY RESTRICTION</p> <p>E OPERATOR TRAINING AND LICENSE EXAMINATION</p> <p>F ADMINISTRATIVE</p> <p>G OPERATIONAL ERROR (EXPLAIN)</p> <p>H OTHER (EXPLAIN)</p> </div> <div style="font-size: small; margin-left: 200px;"> <p>(2) METHOD</p> <p>1 MANUAL</p> <p>2 MANUAL SCRAM</p> <p>3 AUTOMATIC SCRAM</p> </div>

SUMMARY:

DOCKET NO. \_\_\_\_\_

UNIT \_\_\_\_\_

DATE \_\_\_\_\_

COMPLETED BY \_\_\_\_\_

### AVERAGE DAILY UNIT POWER LEVEL

MONTH \_\_\_\_\_

DAY	AVERAGE DAILY POWER LEVEL (MWe-net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-net)
1	_____	17	_____
2	_____	18	_____
3	_____	19	_____
4	_____	20	_____
5	_____	21	_____
6	_____	22	_____
7	_____	23	_____
8	_____	24	_____
9	_____	25	_____
10	_____	26	_____
11	_____	27	_____
12	_____	28	_____
13	_____	29	_____
14	_____	30	_____
15	_____	31	_____
16	_____		

#### DAILY UNIT POWER LEVEL FORM INSTRUCTIONS

On this form, list the average daily unit power level in MWe-net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that by using maximum dependable capacity for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

## 15.6.6 RESPIRATORY PROTECTION FOR AIRBORNE RADIOACTIVITY

### Applicability

This section applies to protection of plant personnel from exposure to excessive airborne radioactivity in accordance with provisions of 10CFR20, Section 20.103.

### Objective

To document the requirements for a respiratory protection program pursuant to 10CFR20.103, subparagraph (c)(3).

### Specification

- A. Personnel shall use appropriate respiratory equipment when required by airborne radioactivity concentrations in working areas, to insure against inhalation, absorption or ingestion of radioactive material in excess of that permitted under 10CFR20, Section 20.103. <sup>(1)</sup>
- B. Credit for respirator protection factors shall be taken as outlined in Table 15.6.7-1.
- C. Persons using respirators shall be relieved at reasonable intervals such that the use of respiratory protection equipment will not endanger personnel or discourage observance of proper work and safety procedures.
- D. A respiratory protection program, described in the FFDSAR, Section 11.2.3, shall be maintained to insure that the objectives of this specification are met.
- E. This section shall be superseded by adoption of proposed changes to 10CFR20, Section 103 which would make this specification unnecessary.

### References

- (1) PSAR Section 11.2

Protection Factors for Respirators

<u>Description</u>	<u>(1) Modes</u>	<u>(2) Protection Factors</u>	
		<u>(3) Particulates, Vapors and Gases Except Tritium Oxide</u>	<u>Tritium Oxide</u>
<b>I. <u>Air-Purifying Respirators</u></b>			
Facepiece, Half Mask		10	1
Facepiece, Full		100	1
<b>II. <u>Atmosphere-Supplying Respirators</u></b>			
<b>A. <u>Air Line Respirator</u></b>			
Facepiece, Full	CF	1000	2
Facepiece, Full	PD	1000	2
Hood	CF	1000	2
Suit	CF	(4)	(4)
<b>B. <u>Self-Contained Breathing Apparatus</u></b>			
	PD	1000	2
<b>III. <u>Combination Respirator</u></b>			
Any combination of air purifying and atmosphere supplying respirator.		Protection factor for type and mode of operation as listed above	

- (1) CF: Continuous Flow  
 PD: Pressure Demand (i.e., Always Positive Pressure)

- (2) (a) For purposes of this authorization, the protection factor is a measure of the degree of protection afforded by a respirator, defined as the ratio of the concentration of airborne radioactive material outside the respiratory protective equipment to that inside the equipment (usually inside the facepiece) under conditions of use. It is applied to the airborne concentration to determine the concentration inhaled by the wearer, according to the following formula:

$$\text{Concentration Inhaled} = \frac{\text{Airborne Concentration}}{\text{Protection Factor}}$$

(b) The protection factors apply:

- (1) Only for individually fitted respirators worn by trained individuals and used and maintained under supervision in a well-planned respiratory protection program.
  - (ii) For air-purifying respirators only when high efficiency particulate filters and/or adsorbents appropriate to the hazard are used.
  - (iii) For atmosphere-supplying respirators only when supplied with adequate respirable air.
- (3) Excluding radioactive contaminants that present an absorption or submersic hazard.
  - (4) Appropriate protection factors must be determined taking account of the permeability of the suit to the contaminant under conditions of use. No protection factor greater than 1000 shall be used except as authorized by the Commission.

NOTE 1: Protection factors for respirators as may be approved in the future by the U. S. Bureau of Mines according to approval schedules for respirators to protect against airborne radionuclides may be used in lieu of the protection factors listed in this table. Where additional respiratory hazards other than radioactive ones are present, especially those immediately dangerous to life, the selection and use of respirators shall also be governed by the approvals of the U. S. Bureau of Mines in accordance with their applicable schedules.

NOTE 2: Radioactive contaminants for which the concentration values in Appendix B, Table 1, of 10CFR, Part 20, are based on internal dose due to inhalation may, in addition, present external exposure hazards at higher concentrations.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENTS NOS. 12 AND 16 TO FACILITY LICENSES NOS. DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY  
WISCONSIN MICHIGAN POWER COMPANY

POINT BEACH UNITS 1 AND 2

DOCKETS NOS. 50-266 AND 50-301

Introduction

By letters dated February 7, 1975 and July 15, 1975, Wisconsin Electric Power Company (WEPCO) proposed changes to the Technical Specifications appended to Facility Operating Licenses Nos. DPR-24 and DPR-27, for Point Beach Units 1 and 2. The proposed amendments involve changes to the reporting requirements and some portions of the administrative controls section of the Technical Specifications.

Discussion

The proposed changes would be administrative in nature and would affect the conduct of operation. The proposed changes are intended to provide uniform license requirements. Areas covered by the proposed uniform specifications include reporting requirements, and a change of the definition of abnormal occurrence.

In Section 208 of the Energy Reorganization Act of 1974 "abnormal occurrences" is defined as an unscheduled incident or event which the Commission determines is significant from the standpoint of public health or safety. The term "abnormal occurrence" is reserved for usage by NRC. Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4, enumerates required reports consistent with Section 208. The proposed change to required reports identifies the reports required of all licensees not already identified by the regulations and those unique to this facility. The proposal would formalize present reporting and would delete any reports no longer needed for assessment of safety related activities.

Evaluation

The new guidance for reporting operating information does not identify any event as an "abnormal occurrence". The proposed reporting require-

ments also delete reporting of information no longer required and duplication of reported information. The standardization of required reports and desired format for the information will permit more rapid recognition of potential problems.

During our review of the proposed changes, we found that certain modifications to the proposal were necessary to have conformance with the desired regulatory position. These changes were discussed with, and concurred in by the licensee, and have been incorporated into the proposed changes.

We have concluded that the proposed changes, as modified, improve the licensee's program for the reporting of the operating information needed by the Commission to assess safety related activities and is acceptable. The modified reporting program is consistent with the guidance provided by Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4.

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

#### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: January 9, 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-266 AND 50-301

WISCONSIN ELECTRIC POWER COMPANY  
WISCONSIN MICHIGAN POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 12 and 16 to Facility Operating Licenses Nos. DPR-24 and DPR-27 issued to Wisconsin Electric Power Company and Wisconsin Michigan Power Company, which revised Technical Specifications for operation of the Point Beach Nuclear Plant Units Nos. 1 and 2, located in the town of Two Creeks, Manitowoc County, Wisconsin.

The amendment modifies the reporting requirements of the Technical Specifications for the Point Beach Nuclear Plant, Units 1 and 2.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

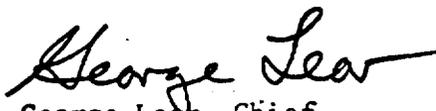
The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated February 7, 1975 and July 15, 1975, (2) Amendments Nos. 12 and 16 to Licenses Nos. DPR-24 and DPR-27, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Document Department, University of Wisconsin - Stevens Point Library, Stevens Point, Wisconsin 54481.

A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 9th day of January, 1976.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief  
Operating Reactors Branch #3  
Division of Reactor Licensing