

Mr. Paul D. Hinnenkamp  
Vice President - Operations  
Entergy Operations, Inc.  
River Bend Station  
P. O. Box 220  
St. Francisville, LA 70775

July 11, 2002

SUBJECT: RIVER BEND STATION, UNIT 1 - ISSUANCE OF AMENDMENT RE:  
RELOCATE REQUIREMENTS FOR MAIN STEAM ISOLATION VALVE  
ISOLATIONS ON CERTAIN AREA TEMPERATURES (TAC NO. MB4025)

Dear Mr. Hinnenkamp:

The Commission has issued the enclosed Amendment No. 124 to Facility Operating License No. NPF-47 for the River Bend Station, Unit 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated February 6, 2002, as supplemented by letter dated June 7, 2002.

The amendment relocates the requirements for Main Steam Isolation Valve isolations on certain area temperatures from TS Section 3.3.6.1, "Primary Containment and Drywell Isolation Instrumentation," to the Technical Requirements Manual.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA by Thomas Alexion for/

Michael Webb, Project Manager, Section 1  
Project Directorate IV  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-458

Enclosures: 1. Amendment No. 124 to NPF-47  
2. Safety Evaluation

cc w/encls: See next page

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L.Hurley, RIV  
D. Bujol, RIV

Accession No.: ML021920319

\*\*No legal objection

\*See previous concurrence

OFFICE	PDIV-1/PM	PDIV-1/LA	RORP	OGC**	PDIV-1/SC
NAME	TAlexion for MWebb	DJohnson	RDennig*	RWeisman	RGramm
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ENTERGY GULF STATES, INC. \*\*

AND

ENTERGY OPERATIONS, INC.

DOCKET NO. 50-458

RIVER BEND STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 124  
License No. NPF-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Gulf States, Inc.\* (the licensee) dated February 6, 2002, as supplemented by letter dated June 7, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

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\* Entergy Operations, Inc. is authorized to act as agent for Entergy Gulf States, Inc., and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

\*\*Entergy Gulf States, Inc., has merged with a wholly owned subsidiary of Entergy Corporation. Entergy Gulf States, Inc., was the surviving company in the merger.

- E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-47 is hereby amended to read as follows:
- (2) Technical Specifications and Environmental Protection Plan
- The Technical Specifications contained in Appendix A, as revised through Amendment No. 124 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. EOI shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.
3. The license amendment is effective as of its date of issuance and shall be implemented within 60 days from the date of issuance. The implementation of this amendment shall include the relocation of certain technical specification requirements to the River Bend Station Technical Requirements Manual as described in the Licensee's application dated February 6, 2002, as supplemented by letter dated June 7, 2002, and evaluated in the staff's Safety Evaluation attached to this amendment.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Robert A. Gramm, Chief, Section 1  
Project Directorate IV  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: July 11, 2002

ATTACHMENT TO LICENSE AMENDMENT NO. 124

FACILITY OPERATING LICENSE NO. NPF-47

DOCKET NO. 50-458

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by Amendment number and contain marginal lines indicating the areas of change.

Remove

Insert

3.3-53

3.3-53

3.3-54

3.3-54

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 124 TO FACILITY OPERATING LICENSE NO. NPF-47  
ENTERGY OPERATIONS, INC.  
RIVER BEND STATION, UNIT 1  
DOCKET NO. 50-458

1.0 INTRODUCTION

By application dated February 6, 2002, as supplemented by letter dated June 7, 2002, Entergy Operations, Inc. (Entergy or the licensee) requested changes to the Technical Specifications (TSs) for the River Bend Station, Unit 1 (RBS). The supplement dated June 7, 2002, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on March 19, 2002 (67 FR 12601).

The proposed changes would relocate the requirement for Main Steam Isolation Valve (MSIV) isolations on certain area temperatures from TS Section 3.3.6.1, "Primary Containment and Drywell Isolation Instrumentation." Specifically, the proposed changes would revise TS Table 3.3.6.1-1, "Primary Containment and Drywell Isolation Instrumentation," by relocating the following functions:

- 1.f. Main Steam Tunnel Area Temperature - High (El. 95ft)
- 1.g. Main Steam Tunnel Area Temperature - High (El. 114ft)
- 1.h. Main Steam Line Turbine Shield Wall Temperature - High
- 1.i. MSL [main steam line] Moisture Separator and Reheater Area Temperature - High

These functions are associated with temperature switches that are located in the turbine building and are used to detect a main steam line break of a magnitude of 25 gallons per minute equivalent steam. Entergy proposed to relocate the requirements associated with these functions to RBS Technical Requirements Manual (TRM). The isolation from function 1.e, Main Steam Tunnel Temperature - High, as well as all the other redundant and diverse isolation functions will remain unchanged.

2.0 REGULATORY EVALUATION

In Section 5.0, "Regulatory Analysis," of Attachment 1 to the February 6, 2002, submittal, the licensee determined that the proposed changes do not require any exemptions or relief from regulatory requirements, other than the TS, and do not affect conformance with the General Design Criteria differently than described in the Updated Safety Analysis Report (USAR). The

regulatory requirements for which the U.S. Nuclear Regulatory Commission (NRC or the Commission) staff based its acceptance are contained in 10 CFR 50.36.

The Commission's regulatory requirements related to the content of TS are set forth in 10 CFR 50.36. The Commission has provided guidance for the contents of TS in its "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" (Final Policy Statement), 58 FR 39132, July 22, 1993. The Final Policy Statement identified four criteria to be used in determining whether particular safety functions are required to be included in the TS. The criteria set forth in the Final Policy Statement have been codified and incorporated into 10 CFR 50.36 (60 FR 36953, July 19, 1995). The criteria are (1) installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary; (2) a process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; and (4) a structure, system, or component which operating experience or probabilistic risk assessment has shown to be significant to public health and safety. As a result, TS requirements which fall within or satisfy any of the criteria in 10 CFR 50.36 must be retained in the TS, while those TS requirements which do not fall within or satisfy these criteria may be relocated to licensee-controlled documents where 10 CFR 50.59 or other regulations provide adequate regulatory control.

### 3.0 TECHNICAL EVALUATION

#### TS Table 3.3.6.1-1, Functions 1.f, 1.g, 1.h, and 1.i

The requirements in TS Table 3.3.6.1-1, for functions 1.f, 1.g, 1.h, and 1.i temperature switches are proposed to be relocated to the TRM. These temperature switches, which are located in the turbine building, do not detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary (i.e., the piping upstream of and including the outboard MSIV). Although these turbine building area temperature switches are used to detect a main steam line break of a magnitude of 25 gallons per minute equivalent steam leak, for small breaks of this size, no credit is taken for the automatic isolation of the MSIVs by the switches. As such, these switches are not assumed to mitigate the consequences of a design basis accident (DBA) or transient, and are not an input assumption for any DBA analysis. In addition, the onsite and offsite dose consequences for a manual isolation have been calculated to remain orders of magnitudes below acceptance criteria for this type of event because virtually no water carryover (and resulting iodine) is expected to occur.

The RBS probabilistic risk analysis (PRA) did not explicitly model the action of the turbine building high temperature function to isolate the MSIVs. The impact of this function is implicitly included in the scram initiator values within the PRA (T2 transients, scrams with a loss of the power conversion system). Using the equipment out of service program, the licensee calculated the contribution to plant risk from T2 scram initiators as a core damage frequency of 7E-09 per year. The MSIV isolation transients are only one contributor to the overall T2 initiator frequency. Therefore, T2 transients, and thus MSIV isolation transients, are not significant

contributors to RBS plant risk. In addition, engineering judgment supports this conclusion since T2 transients do not impact the availability of offsite power sources and do not impact the ability of the plant normal service water and standby service water systems to serve as ultimate heat sinks.

Based on the forgoing discussion, functions 1.f, 1.g, 1.h, and 1.i of TS Table 3.3.6.1-1 do not meet any of the criteria in 10 CFR 50.36 and may be relocated from the TSs to the TRM. The TRM is incorporated by reference into the USAR and any changes to these requirements after they are relocated to the TRM will require an evaluation pursuant to 10 CFR 50.59. Thus, sufficient regulatory controls exist to ensure adequate control of the relocated requirements. Therefore, relocation of these TS requirements for the turbine building area temperature switches to the TRM is acceptable.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Louisiana State official was notified of the proposed issuance of the amendment. The State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (67 FR 12601, published March 19, 2002). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with issuance of the amendment.

#### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. Wrona

Date: July 11, 2002



River Bend Station

cc:

Winston & Strawn  
1400 L Street, N.W.  
Washington, DC 20005-3502

Manager - Licensing  
Entergy Operations, Inc.  
River Bend Station  
P. O. Box 220  
St. Francisville, LA 70775

Senior Resident Inspector  
P. O. Box 1050  
St. Francisville, LA 70775

President of West Feliciana  
Police Jury  
P. O. Box 1921  
St. Francisville, LA 70775

Regional Administrator, Region IV  
U.S. Nuclear Regulatory Commission  
611 Ryan Plaza Drive, Suite 1000  
Arlington, TX 76011

Ms. H. Anne Plettinger  
3456 Villa Rose Drive  
Baton Rouge, LA 70806

Mr. Michael E. Henry, Administrator  
and State Liaison Officer  
Department of Environmental Quality  
P. O. Box 82135  
Baton Rouge, LA 70884-2135

Wise, Carter, Child & Caraway  
P. O. Box 651  
Jackson, MS 39205

Executive Vice President and  
Chief Operating Officer  
Entergy Operations, Inc.  
P. O. Box 31995  
Jackson, MS 39286-1995

General Manager - Plant Operations  
Entergy Operations, Inc.  
River Bend Station  
P. O. Box 220  
St. Francisville, LA 70775

Director - Nuclear Safety  
Entergy Operations, Inc.  
River Bend Station  
P. O. Box 220  
St. Francisville, LA 70775

Vice President - Operations Support  
Entergy Operations, Inc.  
P. O. Box 31995  
Jackson, MS 39286-1995

Attorney General  
State of Louisiana  
P. O. Box 94095  
Baton Rouge, LA 70804-9095

Brian Almon  
Public Utility Commission  
William B. Travis Building  
P.O. Box 13326  
1701 North Congress Avenue  
Austin, Texas 78701-3326

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