



UNITED STATES
NUCLEAR REGULATORY COMMISSION
 WASHINGTON, D.C. 20555-0001

June 5, 1996

Mr. Lee Liu
 Chairman of the Board and
 Chief Executive Officer
 IES Utilities Inc.
 Post Office Box 351
 Cedar Rapids, IA 52406

**SUBJECT: AMENDMENT NO. 214 TO FACILITY OPERATING LICENSE NO. DPR-49 - DUANE
 ARNOLD ENERGY CENTER (TAC NO. M93034)**

Dear Mr. Liu:

The Commission has issued the enclosed Amendment No. 214 to Facility Operating License No. DPR-49 for the Duane Arnold Energy Center (DAEC). This amendment consists of changes to the Technical Specifications (TS) in response to your application dated July 21, 1995, as supplemented August 8, 1995, and December 15, 1995.

The amendment makes administrative changes to various sections of the DAEC TS. The amendment replaces the surveillance condition when an Emergency Service Water pump or loop is inoperable (the surveillance had required demonstration that the opposite train's Emergency Diesel Generator (EDG) was operable) with an OPERABILITY verification of the opposite train's EDG. The amendment modifies the TS to require a determination within 24 hours that the other EDG is not affected by a common cause failure. The determination may be made by evaluation or test. The EDG conditional surveillance will still be required each 72 hours. The amendment revises the format and language of TS Section 5.5 to clarify the requirements and state the capacity of the spent fuel pool and vault storage in order to remove ambiguities in the wording and to be more consistent with the Improved Standard TS guidance. The amendment also revises the list of Operations Committee responsibilities (Section 6.5.1.6) to eliminate Committee review of procedures implementing Security and Emergency Plans.

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A copy of the Safety Evaluation is also enclosed. Notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original signed by:

Glenn B. Kelly, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-331

- Enclosures: 1. Amendment No. 214 to License No. DPR-49
- 2. Safety Evaluation

cc w/encls: See next page

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*See previous concurrence.

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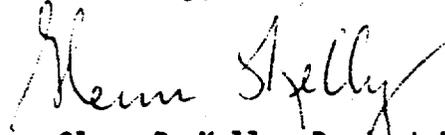
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Mr. Lee Liu

-2-

A copy of the Safety Evaluation is also enclosed. Notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,



Glenn B. Kelly, Project Manager
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Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-331

Enclosures: 1. Amendment No. 214 to
License No. DPR-49
2. Safety Evaluation

cc w/encls: See next page

Mr. Lee Liu
IES Utilities Inc.

Duane Arnold Energy Center

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

IES UTILITIES INC.
CENTRAL IOWA POWER COOPERATIVE
CORN BELT POWER COOPERATIVE
DOCKET NO. 50-331
DUANE ARNOLD ENERGY CENTER
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 214
License No. DPR-49

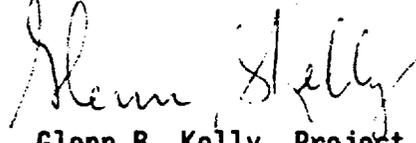
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by IES Utilities Inc. et al., dated July 21, 1995, as supplemented August 8, 1995, and December 15, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-49 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.214 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of the date of issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Glenn B. Kelly, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of issuance: June 5, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 214

FACILITY OPERATING LICENSE NO. DPR-49

DOCKET NO. 50-331

Replace the following pages of the license and Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by vertical lines.

<u>Remove</u>	<u>Insert</u>
4	4
3.5-10	3.5-10
3.5-23	3.5-23
3.8-4	3.8-4
3.8-6	3.8-6
5.5-1	5.5-1
----	5.5-1a
6.5-3	6.5-3
6.8-1	6.8-1
6.8-2	6.8-2

(3) Fire Protection

IES Utilities Inc. shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the Duane Arnold Energy Center and as approved in the SER dated June 1, 1978, and Supplement dated February 10, 1981, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

- (4) The licensee is authorized to operate the Duane Arnold Energy Center following installation of modified safe-ends on the eight primary recirculation system inlet lines which are described in the licensee letter dated July 31, 1978, and supplemented by letter dated December 8, 1978.

(5) Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans, including amendments made pursuant to the authority of 10 CFR 50.54(p). The approved plans, which contain Safeguards Information as described in 10 CFR 73.21, are collectively entitled:

"Duane Arnold Energy Center Security Plan" dated December 1, 1978, January 19, March 9 and March 21, 1978, as revised through revisions dated January 1984 (transmittal letter dated January 12, 1984), as revised by revision dated February 1984 (transmittal letter dated February 27, 1984), as revised by revision dated September 1984 (transmittal letter dated September 26, 1984); "Duane Arnold Energy Center Safeguards Contingency Plan," dated April 1980, as revised through revision dated January 1984 (transmittal letter dated January 12, 1984); "Duane Arnold Energy Center Guard Training and Qualification Plan" dated January 29, 1982, as revised April 1, 1982, as revised through revisions dated January 1984 (transmittal letter dated January 12, 1984), as revised by updated revisions (transmittal letter dated July 30, 1984), as revised by revision dated September 1984 (transmittal letter dated September 26, 1984) as revised by revision dated October 1984 (transmittal letter dated October 26, 1984).

LIMITING CONDITIONS FOR OPERATION

- G. Minimum Low Pressure Cooling and Diesel Generator Availability
1. During any period when one diesel generator is inoperable, continued reactor operation is permissible only during the succeeding seven days unless such diesel generator is sooner made OPERABLE, provided that the remaining diesel generator and all low pressure core and containment cooling subsystems supported by the OPERABLE diesel generator are OPERABLE. If this requirement cannot be met, an orderly SHUTDOWN shall be initiated and the reactor shall be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 2. Any combination of inoperable components in the core and containment cooling systems shall not defeat the capability of the remaining OPERABLE components to fulfill the cooling functions.
 3. When irradiated fuel is in the reactor vessel and the reactor is in the COLD SHUTDOWN Condition or REFUEL Mode:
 - a. If no work is being performed which has the potential for draining the reactor vessel, both core spray and RHR systems may be inoperable; or
 - b. If work is being performed which has the potential for draining the reactor vessel, at least two of any combination of core spray and/or RHR (LPCI or shutdown cooling mode) pumps shall be OPERABLE (including the capability to inject water into the reactor vessel with suction from the suppression pool) except as

SURVEILLANCE REQUIREMENTS

- G. Minimum Low Pressure Cooling and Diesel Generator Availability
1. With one diesel generator inoperable, determine that the OPERABLE diesel generator is not inoperable due to common cause failure within 24 hours and perform Surveillance Requirement 4.8.A.2.a.1.a within the first 72 hours and every 72 hours thereafter. In addition, all low pressure core cooling and containment cooling subsystems supported by the OPERABLE diesel shall be verified to be OPERABLE.

4.5 BASES

Core and Containment Cooling Systems Surveillance Frequencies

The testing interval for the core and containment cooling systems is based on industry practice, quantitative reliability analysis, judgement, and practicality. The core cooling systems have not been designed to be fully testable during operation. For example, in the case of the HPCI, automatic initiation during power operation would result in pumping cold water into the reactor water vessel, which is not desirable. Complete ADS testing during power operation causes an undesirable loss-of-coolant inventory. To increase the availability of the core and containment cooling systems, the components which make up the system, i.e., instrumentation, pumps, valves, etc., are tested frequently. The test intervals are based upon Section XI of the ASME Code. A simulated automatic actuation test once per year combined with frequent tests of the pumps and injection valves is deemed to be adequate testing of these systems.

When components and subsystems are out-of-service, overall core and containment cooling reliability is maintained by evaluating the operability of the remaining equipment. The degree of evaluation depends on the nature of the reason for the out-of-service equipment. For routine out-of-service periods caused by preventative maintenance, etc., the evaluation may consist of verifying the redundant equipment is not known to be inoperable and applicable surveillance intervals have been satisfied. However, if a failure due to a design deficiency caused the outage, then the evaluation of operability should be thorough enough to assure that a generic problem does not exist.

The Diesel Generators are critical to operation of all core and containment cooling systems. Therefore, it is imperative that they be maintained in a standby readiness condition. In the event that one Diesel Generator is made or found to be inoperable, the remaining Diesel Generator must be shown to not be susceptible to the same condition within 24 hours. This evaluation may be performed by analysis or inspection or by demonstration of OPERABILITY. The OPERABLE Diesel Generator must also be demonstrated to continue to be OPERABLE each 72 hours during the period that the other Diesel Generator is inoperable.

The RHR valve power bus is not instrumented. For this reason surveillance requirements require once per shift observation and verification of lights and instrumentation operability.

LIMITING CONDITIONS FOR OPERATION

chargers for the 24 Volt Systems, two of the three battery chargers for the 125 Volt Systems, and one of the two battery chargers for the 250 Volt System shall be OPERABLE.

2. Operation with Inoperable Components.
 - a. With normal battery room ventilation unavailable, portable ventilation equipment shall be provided.
 - b. With one of the two 125 Volt DC Systems inoperable, verify that Specification 3.5.G is met, and within 3 days either:
 - 1) Restore the inoperable 125 Volt DC System to OPERABLE status, or
 - 2) Be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 - c. With the 250 Volt DC System inoperable, the HPCI System and other affected primary containment isolation valves shall be considered inoperable and the requirements of Specifications 3.5.D and 3.7.B respectively shall be met.
 - d. With one of the 24 Volt DC Systems inoperable, the requirements associated with the affected instruments of Specifications 3.1 and 3.2 shall be met.

SURVEILLANCE REQUIREMENTS

- voltage shall be measured and recorded.
- b. Each three months the essential batteries' voltage of each cell to the nearest 0.01 Volt, specific gravity of each cell, and temperature of every fifth cell shall be measured and recorded.
 - c. Once each OPERATING CYCLE, the essential batteries shall be subjected to a Service Discharge Test (load profile). The specific gravity and voltage of each cell shall be determined after the discharge and recorded.
 - d. Once every five years, the essential batteries shall be subjected to a Performance Discharge Test (capacity). This test will be performed in lieu of the Service Test requirement of 4.8.B.1.c above.
2. Surveillance Requirements with Inoperable Components.
 - a. With the battery room ventilation unavailable, samples of the battery room atmosphere shall be taken daily for hydrogen concentration determination.

LIMITING CONDITIONS FOR OPERATION

- E. Emergency Service Water System
1. Except as required in Specification 3.8.E.2 below, both Emergency Service Water System loops shall be OPERABLE whenever irradiated fuel is in the reactor vessel and reactor coolant temperature is greater than 212°F.

 2. With one of the Emergency Service Water System pumps or loops inoperable, REACTOR POWER OPERATION must be limited to seven days unless OPERABILITY of that system is restored within this period. During such seven days all active components of the other Emergency Service Water System shall be OPERABLE, provided the requirements of Specification 3.5.G are met.

 3. If the requirements of Specification 3.8.E cannot be met, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

SURVEILLANCE REQUIREMENTS

- E. Emergency Service Water System
1. Emergency Service Water System surveillance shall be as follows:
 - a. Simulated automatic actuation test. once/
OPERATING
CYCLE

 - b. Pump and motor operated valve OPERABILITY. As specified in
the IST Program

 - c. Flow Rate Test

<p>Each Emergency Service Water pump shall deliver at least that flow determined from Figure 4.8.E-1 for the existing river water temperature.</p>	<p>After major pump maintenance and once per 3 months, except weekly during periods of time the river water temperature exceeds 80°F.</p>
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 2. With one Emergency Service Water System pump or loop inoperable, the OPERABLE pump and loop shall be verified to be OPERABLE. In addition, all low pressure core cooling and containment cooling subsystems and the diesel generator supported by the OPERABLE ESW loop shall be verified to be OPERABLE.

DAEC-1

5.5 SPENT AND NEW FUEL STORAGE

5.5.1 Criticality

- 5.5.1.1 The spent fuel storage racks are designed and shall be maintained with:
- Fuel assemblies having a maximum k_{∞} of 1.31 in the normal reactor core configuration at cold conditions and a maximum initial uniform average U-235 enrichment of 4.6 weight percent.
 - $k_{\text{eff}} \leq 0.95$ flooded with unborated water.
- 5.5.1.2 The new fuel storage racks are designed and shall be maintained with:
- Fuel assemblies having a maximum k_{∞} of 1.31 in the normal reactor core configuration at cold conditions.
 - $k_{\text{eff}} \leq 0.90$ dry and ≤ 0.95 flooded with unborated water.

5.5.2 Capacity

- 5.5.2.1 The spent fuel storage pool has been analyzed to allow storage of a maximum of 3152 fuel assemblies in a vertical orientation only.
- 5.5.2.2 The new fuel storage vault is equipped with racks for storage of up to 110 fuel assemblies in a vertical orientation only.

Bases

The basis for the k_{∞} limit is described in Reference 1 for the GE-designed new fuel storage racks. Compliance with this specification is demonstrated by comparing the beginning-of-life, uncontrolled k_{∞} values for the fuel type of interest to the 1.31 limit. For GE-supplied fuel, k_{∞} values can be found in Reference 2. The k_{∞} values found in Reference 2 represent the maximum, exposure-dependent lattice reactivity and can be conservatively applied to the new fuel limit.

Calculations have been performed (Reference 3) to determine the bounding reactivity limits for bundles of GE-designed fuel, when stored in the spent fuel storage racks of an approved design. These analyses were performed conservatively assuming uniform average initial enrichments in a parametric evaluation for fuel with enrichments up to 4.6 wt% U-235 initially. The bounding limit of an infinite multiplication factor of 1.31 for fuel of 4.6 wt% enrichment (or less) was evaluated at the maximum k_{∞} over burnup and includes a conservative allowance for possible differences between the rack design calculations and the fuel vendor calculations.

DAEC-1

References

- 1) General Electric Standard Application for Reactor Fuel, NEDE-24011-P-A.*
- 2) General Electric Fuel Bundle Designs, NEDE-31152-P.*
- 3) Licensing Report for Spent Fuel Storage Capacity Expansion, Duane Arnold Energy Center, Holtec Report HI-92889.

*Latest NRC-approved revision.

DAEC-1

- f. Review of all Reportable Events.
- g. Review of facility operations to detect potential safety hazards.
- h. Performance of special reviews, investigations or analysis and reports thereon as requested by the Chairman of the Safety Committee.
- i. Review of the Plant Security Plan.
- j. Review of the Emergency Plan.
- k. Review of every unplanned release of radioactivity to the environs for which a report to the NRC is required.
- l. Review of changes to the Offsite Dose Assessment Manual and changes to the Process Control Program.
- m. Review of the Fire Protection Program and implementing procedures.

6.5.1.7 Authority

The Operations Committee shall:

- a. Recommend to the Plant Superintendent-Nuclear written approval or disapproval of items considered under Specification 6.5.1.6 (a) through (d) above.

6.8 PLANT OPERATING PROCEDURES

6.8.1 Written procedures involving nuclear safety, including applicable check-off lists and instructions, covering areas listed below shall be prepared, and approved as specified in Subsection 6.8.2. All procedures shall be implemented and maintained.

1. Normal startup, operation, and shutdown of systems and components of the facility.
2. Refueling operation.
3. Actions to be taken to correct specific and foreseen potential malfunctions of systems or components, including responses to alarms, suspected primary system leaks, and abnormal reactivity changes.
4. Emergency and off-normal condition procedures.
5. Preventive and corrective maintenance operations which could have an effect on the nuclear safety of the facility.
6. Surveillance and testing requirements of equipment that could have an effect on the nuclear safety of the facility.
7. Deleted.

DAEC-1

8. Deleted
9. Operation of radioactive waste systems.
10. Fire Protection Program implementation.
11. A preventive maintenance and periodic visual examination program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient to as low as practical levels. This program shall also include provisions for performance of periodic systems leak tests of each system once per OPERATING CYCLE.
12. Program to ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions, including training of personnel, procedures for monitoring and provisions for maintenance of sampling and analysis equipment.
13. Administrative procedures for shift overtime for Operations personnel to be consistent with the Commission's June 15, 1982 policy statement.
14. OFFSITE DOSE ASSESSMENT MANUAL.
15. PROCESS CONTROL PROGRAM.
16. Quality Control Program for effluents.

6.8.2 Procedures described in 6.8.1 above, and changes thereto, shall be reviewed by the Operations Committee as indicated in Specification 6.5.1.6 and approved by the Plant Superintendent-Nuclear or designee prior to implementation, except as provided in 6.8.3 below.

6.8.3 Temporary minor changes to procedures described in 6.8.1 above which do not change the intent of the original procedure may be made with the concurrence of two members of the plant management staff, at least one of whom shall hold a senior operator license. Such changes shall be documented and promptly reviewed by the Operations Committee and by the Plant Superintendent-Nuclear or designee. Subsequent incorporation, if necessary, as a permanent change, shall be in accord with 6.8.2 above.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 214 TO FACILITY OPERATING LICENSE NO. DPR-49

IES UTILITIES INC.

CENTRAL IOWA POWER COOPERATIVE

CORN BELT POWER COOPERATIVE

DUANE ARNOLD ENERGY CENTER

DOCKET NO. 50-331

1.0 INTRODUCTION

By letter dated July 21, 1995, as supplemented August 8, 1995 and December 15, 1995, IES Utilities Inc. (licensee) submitted a request for revision of the Technical Specifications (TS) for the Duane Arnold Energy Center (DAEC). The request sought to make several administrative changes to various sections of the DAEC TS. The first change to the TS replaces the surveillance condition when an Emergency Service Water pump or loop is inoperable (the surveillance required demonstration that the opposite train's Emergency Diesel Generator (EDG) was operable) with an OPERABILITY verification of the opposite train's EDG. The amendment modifies the TS to require a determination within 24 hours that the other EDG is not affected by a common cause failure. The determination may be made by evaluation or test. The EDG conditional surveillance will still be required each 72 hours. The second change to the TS revises the format and language of TS Section 5.5 to clarify the requirements and state the capacity of the spent fuel pool and vault storage in order to remove ambiguities in the wording and to be more consistent with the Improved Standard TS guidance. The third change revises the list of Operations Committee responsibilities (Section 6.5.1.6) to eliminate Committee review of procedures implementing Security and Emergency Plans. The fourth change corrects wording in the operating license to be consistent with Amendment 47.

The additional information contained in the supplemental letters dated August 8, 1995, and December 15, 1995, was noticed in the Federal Register on February 2, 1996 (61 FR 3953).

2.0 EVALUATION

Surveillance Requirements - One change to the surveillance requirements replaces a surveillance (4.8.E.2) that had the potential to require unnecessary starts of Emergency Diesel Generators (EDGs). The existing TS surveillance for the Emergency Service Water (ESW) System requires that if an

ESW pump or loop becomes inoperable, then the opposite train's EDG must be physically tested to demonstrate that it is operable. The staff agrees with the licensee that this is an unnecessary and burdensome test unless there is some reason to believe that a common-mode/common-cause failure exists between the two ESW trains, or between the inoperable ESW train and the opposite EDG. The change modifies the TS surveillance to require that the EDG supported by the operable ESW loop be verified to be operable. The staff finds this change to be acceptable.

The second surveillance requirement modifies the requirements for testing an EDG when the other EDG is inoperable and brings the TS into close agreement with the Improved Technical Specifications (NUREG-1433, revision 1). The modification requires the licensee to determine within 24 hours that the OPERABLE EDG is not inoperable due to common-cause failure. If it is determined to be OPERABLE, within 72 hours (and every 72 hours thereafter) it must be manually started, brought up to synchronous speed, and verified to deliver rated voltage and frequency. This testing confirms the capability of the OPERABLE EDG. The staff finds this change to be acceptable.

Spent Fuel and New Fuel Storage - The changes to the spent fuel and new fuel storage TS eliminates ambiguities in the wording of the existing TS but retains the existing limits on spent fuel and new fuel storage. The limit on the number of spent fuel assemblies that can be stored in the spent fuel pool is based on Amendment 195 to the TS (February 22, 1994). The limit on the number of new fuel assemblies is based on the UFSAR and the physical capacity of the new fuel storage vault. The format of the changes is consistent with the Improved Standard TS. The staff finds these changes acceptable.

Operations Committee Responsibilities - In Generic Letter 93-07, the staff provided guidance for changes to the TS to remove the audit of the emergency plans, security plans, and their implementing procedures from the list of responsibilities of the company's nuclear audit and review group. Parts 50 and 73 of Title 10 of the Code of Federal Regulations (10 CFR) include provisions that are sufficient to address these requirements. The licensee's changes follow the intent of Generic Letter 93-07, which provided guidance on removal of TS audit responsibilities from the Operations Committee. The staff finds these changes acceptable.

Operating Licensee - In Operating License (OL) Section 2.C(4), there was an erroneous reference to a preceding paragraph in the OL. The error was made in Amendment 47 that supported recirculation piping safe end replacement. The modification corrects the section by adding references to letters submitted by the licensee in support of the safe end replacement. The staff finds these changes acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Iowa State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATIONS

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or changes a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluent that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued proposed findings that the amendment involves no significant hazards consideration and there has been no public comment on such findings (60 FR 49938 and 61 FR 3953)). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Glenn B. Kelly

Date: June 5, 1996