

July 30, 2002

Mr. Michael R. Kansler
Senior Vice President and
Chief Operating Officer
Entergy Nuclear Operations, Inc.
440 Hamilton Avenue
White Plains, NY 10601

SUBJECT: INDIAN POINT NUCLEAR GENERATING UNIT NO. 2 - AMENDMENT RE:
REACTOR VESSEL MATERIAL SURVEILLANCE PROGRAM
(TAC NO. MB3917)

Dear Mr. Kansler:

The Commission has issued the enclosed Amendment No. 231 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated January 8, 2002.

The amendment revised TS 3.1.B, "Heatup and Cooldown," to delete the requirements governing the reactor vessel surveillance program, including the reactor vessel specimen withdrawal schedule. In addition, the amendment corrected TS 4.2, "Inservice Inspection and Testing;" TS 5.2.C, "Design Features - Containment;" and TS 6.4, "Administrative Controls - Training." TS Sections 6.1, "Responsibility" and 6.2, "Organization," were changed to reflect the organizational changes resulting from the transfer of the operating license to ENO on September 6, 2001.

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's next regular biweekly *Federal Register* notice.

Sincerely,

/RA/

Patrick D. Milano, Sr. Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-247

Enclosures: 1. Amendment No. 231 to DPR-26
2. Safety Evaluation

cc w/encls: See next page

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Package No.: ML022120616

Accession Number: ML021910684

TSs: ML022120205

OFFICE	PDI-1/PM	PDI-1/LA	EMCB/SC	IOHS/SC	OGC	PDI-1/SC
NAME	PMilano	TCClark for SLittle	SCoffin	DTrimble	SBrock	RLaufer
DATE	7/15/02	7/15/02	7/15/02	7/18/02	7/22/02	7/30/02

Official Record Copy

DATED: July 30, 2002

AMENDMENT NO. 231 TO FACILITY OPERATING LICENSE NO. DPR-26 INDIAN POINT
UNIT 2

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ENTERGY NUCLEAR INDIAN POINT 2, LLC

ENTERGY NUCLEAR OPERATIONS, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 231
License No. DPR-26

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Entergy Nuclear Operations, Inc. (the licensee) dated January 8, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 231, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days. In implementing this amendment, the licensee shall revise Chapter 4, Section 4.5.2, of the Updated Final Safety Analysis Report to add a description of the reactor vessel surveillance program and the list of the surveillance program capsules, including the withdrawal schedule, as described in the licensee's application dated January 8, 2002.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Richard J. Laufer, Chief, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: July 30, 2002

ATTACHMENT TO LICENSE AMENDMENT NO. 231

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages

3.1.B-1
3.1.B-2
3.1.B-7
4.2-1
5.2-2
6-1
6-3

Insert Pages

3.1.B-1
3.1.B-2
3.1.B-7
4.2-1
5.2-2
6-1
6-3

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 231 TO FACILITY OPERATING LICENSE NO. DPR-26

ENTERGY NUCLEAR OPERATIONS, INC.

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

DOCKET NO. 50-247

1.0 INTRODUCTION

By letter dated January 8, 2002, Entergy Nuclear Operations, Inc. (ENO or the licensee) submitted a request for changes to the Indian Point Nuclear Generating Unit No. 2 Technical Specifications (TSs). The requested changes would revise TS 3.1.B, "Heatup and Cooldown," to delete the requirements governing the reactor vessel surveillance program, including the reactor vessel specimen withdrawal schedule. In addition, the requested changes would correct errors in TS 4.2, "Inservice Inspection and Testing;" TS 5.2.C, "Design Features - Containment;" and TS 6.4, "Administrative Controls - Training." TS Sections 6.1, "Responsibility," and 6.2, "Organization," would be changed to reflect the organizational changes resulting from the transfer of the operating license to ENO on September 6, 2001.

2.0 REGULATORY EVALUATION

The U.S. Nuclear Regulatory Commission (NRC) staff finds that, in its January 8, 2002, application, the licensee identified the applicable regulatory requirements. The regulatory requirements for which the staff based its acceptance are as follows:

1. Appendix G, "Fracture Toughness Requirements," to Part 50 of Title 10 of the *Code of Federal Regulations* (10 CFR Part 50) establishes the fundamental regulatory requirements with respect to the fracture toughness of ferritic materials of pressure-retaining components in the reactor coolant pressure boundary.
2. Appendix H, "Reactor Vessel Material Surveillance Program Requirements," to Part 50 of Title 10 of the *Code of Federal Regulations* (10 CFR Part 50) establishes the surveillance program to monitor changes in fracture toughness properties of ferritic materials used in the reactor vessel beltline region resulting from exposure to neutron irradiation and the thermal environment.
3. In Generic Letter (GL) 91-01, "Removal of the Schedule for the Withdrawal of Reactor Vessel Material Specimens from Technical Specifications," dated January 4, 1991, the NRC staff advised licensees that they may request a license amendment to remove the withdrawal schedule for reactor vessel material specimens from TSs. GL 91-01 included guidance that "if the Bases Section of this TS includes a reference to the TS

table that provides the schedule for material specimen withdrawal that is being removed from the TS, this section should be updated to reflect the removal of this TS table. However, to obtain a readily available copy of the NRC-approved version of the specimen withdrawal schedule, licensees should provide a commitment to include this schedule in the next revision of the updated safety analysis report.”

3.0 TECHNICAL EVALUATION

The NRC staff has reviewed the licensee’s regulatory and technical analyses in support of its proposed license amendment which are described in the licensee’s January 8, 2002, application. The detailed evaluation below will support the conclusions that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission’s regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

3.1 Reactor Vessel Specimen Withdrawal Schedule

Section III.B.3 of Appendix H to 10 CFR Part 50 states that a proposed withdrawal schedule, along with technical justification, must be submitted to the NRC for approval prior to implementation. The results of the evaluation of the capsule withdrawal specimens are used to determine the effects of neutron irradiation on the reactor pressure vessel. The results must be accounted for in the licensee’s analytical method used to determine the reactor vessel pressure-temperature limits as required by Appendix G to 10 CFR Part 50. As stated in GL 91-01, the control of changes to the surveillance removal schedule by way of a license amendment to modify the TS duplicates the requirements of Section II.B.3 of Appendix H. Thus, the removal of the schedule does not result in a loss of regulatory control because the changes to this schedule must meet the requirements of Appendix H.

The NRC staff evaluated the licensee’s submittal against the guidance provided in GL 91-01. Specifically, the staff determined that the proposed TS changes met the following criteria:

- (a) the requirement that the results of the examinations shall be used to update the TS figures for the pressure-temperature operating limits is retained;
- (b) the TS Basis section provides a detailed description of the basis for the limiting condition for operation and associated surveillance requirements and there is no reference to the table of the surveillance capsule withdrawal schedule; and
- (c) the licensee committed to maintain the NRC-approved version of the specimen withdrawal schedule in Updated Final Safety Analysis Report (UFSAR).

The staff also found that the licensee adequately removed the references to the surveillance capsule withdrawal schedule table from the TS Basis and included a commitment to maintain the NRC-approved version of the reactor vessel material specimen removal schedule in the UFSAR Section 4.5.2, “Reactor Vessel Surveillance Program.” Therefore, the staff found that the licensee adequately followed the guidance of GL 91-01. The NRC staff concludes that the proposed TS changes meet the criteria of GL 91-01, and are acceptable.

3.2 Other TS Changes

3.2.1 TS 4.2.1

The current TS 4.2.1, "Inservice Testing," states that inservice testing of pumps and valves shall be performed in accordance with Section XI of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code as required by 10 CFR Section 50.55a(g). However, 10 CFR 50.55a(g) provides inservice inspection requirements, not inservice testing requirements. The correct reference should be 10 CFR 50.55a(f), "Inservice Testing Requirements." Therefore, the proposed change to correct the reference in TS 4.2.1 is acceptable.

3.2.2 TS 5.2.C.1

The current TS 5.2.C, "Containment Systems," states, in part, that during the initial period of spray operation, sodium hydroxide would be added to the spray water to increase the removal of iodine from the containment atmosphere. In its January 8, 2002, application, the licensee noted that the requirement to use sodium hydroxide as an additive in the post-accident containment spray was deleted from TS Sections 3.3 and 4.5 by Amendment No. 191, dated April 23, 1997. However, TS 5.2.C.1 was not revised by the amendment. Since the deletion of sodium hydroxide was previously approved by the NRC, the staff finds the deletion from TS 5.2.C.1 administrative in nature and, therefore, acceptable.

3.2.3 TSs 6.1.1 and 6.2.1.c

The licensee proposed changing the designation of the Vice President Nuclear Power to "corporate officer with direct responsibility for the plant," to reflect the current organization of ENO. In addition, the licensee stated that the proposed wording would be identical to the comparable requirement in the TSs for Indian Point Unit 3.

The NRC staff finds that the corporate management organization and functional responsibilities as they relate to IP2 are described in the UFSAR Section 12.1, "Organization and Responsibility," and in the quality assurance program description. Since the functional responsibility of the corporate officer is not being revised, the staff finds the proposed change removes the specificity in the position titles but does not reduce the regulatory commitments for these positions. Thus, the changes are acceptable.

3.2.4 TS 6.4.1

The licensee proposed deleting the requirement that the retraining and replacement training program for the facility staff meet the requirements of Appendix A to 10 CFR Part 55, "Operators' Licenses." The licensee stated that 10 CFR Part 55 no longer includes an Appendix A and that operator retraining and replacement operator training programs are now specified by 10 CFR 55.59 and 10 CFR 55.31(a)(4), respectively.

The NRC staff finds that the deletion of the reference to Appendix A to 10 CFR Part 55 corrects the inaccurate reference to the regulatory requirements. In addition, the licensee must still comply with the applicable requirements in 10 CFR Part 55. Therefore, the NRC finds the proposed change acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (67 FR 10011). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: M. Khanna

Date: July 30, 2002