

March 29, 1993

Docket No. 50-331

DISTRIBUTION

Mr. Lee Liu  
Chairman of the Board and  
Chief Executive Officer  
Iowa Electric Light and Power Company  
Post Office Box 351  
Cedar Rapids, Iowa 52406

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PKreutzer	Region RIII, DRP
RPulsifer	PD3-3 Gray

Dear Mr. Liu:

SUBJECT: AMENDMENT NO.192 TO FACILITY OPERATING LICENSE NO. DPR-49  
(TAC NO. M85662)

The Commission has issued the enclosed Amendment No. 192 to Facility Operating License No. DPR-49 for the Duane Arnold Energy Center. This amendment consists of changes to the Technical Specifications in response to your application dated January 29, 1993.

The amendment revises the Technical Specifications by changing the surveillance interval for the Source Range Monitor functional test.

A copy of the related Safety Evaluation is also enclosed. Notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original Signed By:

Robert M. Pulsifer, Project Manager  
Project Directorate III-3  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 192 to  
License No. DPR-49
2. Safety Evaluation

cc w/enclosures:  
See next page

OFFICE	PD3-3:LA:DRPW	PD3-3:PM:DRPW	PD3-3:PD:DRPW	OGC	SRXB
NAME	PKreutzer <i>mit</i>	RPulsifer <i>bj</i>	JHannon <i>SBH</i>	<i>CPW</i>	REJES
DATE	3/19/93 <i>WBS</i>	3/19/93	3/19/93	3/17/93	3/14/93

3/29/93 OFFICIAL RECORD

DOCUMENT NAME: G:\DUANEARN\DUA85662.AMD

CP-1

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PDR ADDOCK 05000331  
P PDR

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

March 29, 1993

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Chairman of the Board and  
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Sincerely,

A handwritten signature in dark ink, appearing to read "Robert M. Pulsifer", is written over the typed name.

Robert M. Pulsifer, Project Manager  
Project Directorate III-3  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

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1. Amendment No. 192 to  
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See next page

Mr. Lee Liu  
Iowa Electric Light and Power Company

Duane Arnold Energy Center

cc:

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Chairman, Linn County  
Board of Supervisors  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

IOWA ELECTRIC LIGHT AND POWER COMPANY  
CENTRAL IOWA POWER COOPERATIVE  
CORN BELT POWER COOPERATIVE

DOCKET NO. 50-331

DUANE ARNOLD ENERGY CENTER

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 192  
License No. DPR-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Iowa Electric Light and Power Company, et al., dated January 29, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-49 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 192, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of the date of issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert M. Pulsifer, Project Manager  
Project Directorate III-3  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of issuance: March 29, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 192

FACILITY OPERATING LICENSE NO. DPR-49

DOCKET NO. 50-331

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised areas are indicated by marginal lines.

Remove

3.9-5

Insert

3.9-5

LIMITING CONDITIONS FOR OPERATIONB. Core Monitoring

1. During CORE ALTERATIONS two SRMs shall be OPERABLE - one in the core quadrant where fuel or control rods are being moved, and one in an adjacent quadrant, except as specified in Specifications 3.9.B.2 and 3.9.B.3. For an SRM to be considered OPERABLE, the following conditions shall be satisfied:
  - a. The SRMs shall be inserted to the normal operating level. (Use of special movable, dunking type detectors during CORE ALTERATIONS in place of normal detectors is permissible as long as the detector is connected to the normal SRM circuit.)
  - b. The SRMs shall have a minimum of 3 cps with all rods fully inserted in the core.
2. Prior to spiral unloading, the SRMs shall be proven OPERABLE as stated in Specifications 3.9.B.1; however, during spiral unloading the count rate may drop below 3 cps when the last bundles remaining in the core that surround the SRMs are being unloaded.
3. Prior to spiral reloading, two to four fuel assemblies which have previously accumulated exposure in the reactor shall be loaded next to each of the 4 SRMs to obtain the required 3 cps. Until these assemblies have been loaded, the 3 cps requirement is not necessary.

SURVEILLANCE REQUIREMENTSB. Core Monitoring

1. The SRMs shall be functionally tested prior to CORE ALTERATIONS and weekly thereafter as long as CORE ALTERATIONS are underway.
2. The SRMs shall be checked for neutron response prior to spiral unloading and daily as long as fuel is in the core.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 192 TO FACILITY OPERATING LICENSE NO. DPR-49

IOWA ELECTRIC LIGHT AND POWER COMPANY  
CENTRAL IOWA POWER COOPERATIVE  
CORN BELT POWER COOPERATIVE

DUANE ARNOLD ENERGY CENTER

DOCKET NO. 50-331

1.0 INTRODUCTION

Amendment No. 186 issued August 25, 1992, to the Duane Arnold Energy Center (DAEC) Technical Specifications (TS) revised Section 3/4.9 Core Alterations. That amendment inadvertently made the Source Range Monitor (SRM) functional test interval daily. This amendment changes the functional test interval for the SRMs to weekly.

2.0 EVALUATION

Surveillance Requirement 4.9.B.1 was revised in Amendment No. 186 to improve the clarity of the TS. The SRM response check and functional test were addressed in the same Surveillance Requirement. The response check did have a daily requirement while the functional test did not have an interval specified. When the response check was separated from the functional test requirement in Amendment No. 186, the daily requirement was inadvertently carried to the functional test.

A weekly interval is consistent with DAEC practice in accordance with doing their Surveillance Testing Program while accomplishing core alterations. This weekly interval is also consistent with the DAEC TS Bases as well as NUREG-1433, Standard Technical Specifications.

The staff has reviewed the licensee's submittal which requests a revision to the TS to change the SRM functional test interval from daily to weekly. Based on its review, the staff concludes that the requested change is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Iowa State official was notified of the proposed issuance of the amendment. The State official had no comments.



#### 4.0 ENVIRONMENTAL CONSIDERATIONS

This amendment changes a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (58 FR 12264). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### 5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Robert Pulsifer

Date: March 29, 1993