



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION II  
SAM NUNN ATLANTA FEDERAL CENTER  
61 FORSYTH STREET SW SUITE 23T85  
ATLANTA, GEORGIA 30303-8931**

**July 9, 2002**

South Carolina Electric & Gas Company  
ATTN: Mr. Stephen A. Byrne  
Senior Vice President, Nuclear Operations  
Virgil C. Summer Nuclear Station  
P. O. Box 88  
Jenkinsville, SC 29065

**SUBJECT: VIRGIL C. SUMMER NUCLEAR STATION - NOTIFICATION OF SAFETY  
SYSTEM DESIGN AND PERFORMANCE CAPABILITY INSPECTION  
(NRC INSPECTION REPORT NO. 50-395/02-07)**

Dear Mr. Byrne:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a safety system design and performance capability inspection at your Virgil C. Summer Nuclear Station during September 23-27, 2002 and October 7-11, 2002. A team of five inspectors will perform the inspection. The inspection team will be led by Mr. F. Jape, a Senior Reactor Inspector from the NRC Region II Office. The inspection will be conducted in accordance with baseline Inspection Procedure 71111.21, Safety System Design and Performance Capability.

The inspection objective will be to evaluate the capability of the plant and installed equipment to mitigate a small break loss of coolant accident. We will evaluate the capability of the high pressure injection system and support systems, as well as other related systems, to perform the functions required to mitigate a small break loss of coolant accident event.

During a telephone conversation on July 3, 2002, Mr. F. Jape of my staff, and Mr. Melvin N. Browne of your staff, confirmed arrangements for an information gathering site visit and the two-week onsite inspection. The schedule is as follows:

- Information gathering visit: Week of July 29, 2002, and August 2, 2002,
- Onsite inspection: September 23-27, 2002 and October 7-11, 2002

The purpose of the information gathering visit is to obtain information and documentation outlined in the Enclosure needed to support the inspection. Mr. Walter Rogers, a Region II Senior Reactor Analyst, may accompany Mr. F. Jape during the information gathering visit to review PRA data and identify risk significant components which will be examined during the inspection. Please contact Mr. F. Jape prior to preparing copies of the materials listed in the Enclosure. The inspectors will try to minimize your administrative burden by specifically identifying only those documents required for inspection preparation.

During the information gathering visit, the team leader will also discuss the following inspection support administrative details: office space; specific documents requested to be made available to the team in their office space; arrangements for site access; and the availability of knowledgeable plant engineering and licensing organization personnel to serve as points of contact during the inspection.

Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact me at (404) 562-4669, or Mr. F. Jape at (404) 562-4541.

Sincerely,

**/original signed by J. Moorman for: RA/**

D. Charles Payne, Acting Chief  
Engineering Branch 1  
Division of Reactor Safety

Docket No. 50-395  
License No. NPF-12

Enclosure: Information Request for the Safety System Design and  
Performance Capability Inspection

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(cc w/encl cont'd - See page 3)

SCE&G

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NAME	JAPE	ROGERS	LANDIS				
DATE	<b>7/9/2002</b>	<b>7/9/2002</b>	<b>7/9/2002</b>	7/ /2002	7/ /2002	7/ /2002	7/ /2002
E-MAIL COPY?	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO

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**INFORMATION REQUEST FOR THE SAFETY SYSTEM DESIGN AND  
PERFORMANCE CAPABILITY INSPECTION**

**SMALL BREAK LOCA EVENT**

Note: Electronic media is preferred if readily available (i.e., on computer disc).

1. Site specific administrative procedures related to standard operation, abnormal operation, and emergency operation of the charging pump system, including support systems, and other related systems during a small break loss of coolant accident (LOCA). Other related systems include, but may not be limited to the emergency power source, other portions of the emergency core cooling system ECCS (e.g. the accumulator flooding system, residual heat removal pumps, and containment building sumps), the service water system, and the instrument air system.
2. Design criteria (i.e., design basis documents) for the ECCS components and other related systems.
3. Charging pumps Technical Specification requirements and a list of associated surveillance test/calibration procedures for the system and related systems.
4. Charging pumps, and other related systems, piping and instrumentation drawings, one-line diagrams; electrical schematics, and wiring and logic diagrams.
5. A list of engineering calculations (Electrical, Instrumentation and Controls and Mechanical/Nuclear) applicable to the ECCS components, and other related systems.
6. A list of plant modifications to the ECCS, and other related systems, implemented since 1992.
7. List of current open temporary modifications and operator work arounds involving operation of the ECCS components and the other related systems.
8. List of Problem Investigation Process Reports (PIPs) initiated since 1992 affecting the ECCS components, and other related systems.
9. Summary of corrective maintenance activities, including the maintenance rule event log, performed on ECCS components and other related systems in the past 12 months.
10. An index of drawings for the ECCS, and other related systems.
1. Self-assessment performed on ECCS and other related systems in the last 24 months.
12. System description and operator training modules for the ECCS and other related systems.
13. Strategy for handling a small break LOCA.

ENCLOSURE

14. List of Operating Experience Program evaluations of industry, vendor, or NRC generic issues related to the ECCS components for the past 3 years.
15. List of valves in the ECCS required to change position for a small break LOCA.
16. List of instrument setpoint changes affecting the ECCS and related systems initiated since 1992. Include the number and title, date, brief description, and corresponding calculation number.
17. PRA Fault Tree Data for the ECCS.
18. PRA/Risk Achievement Worth (RAW) listing for the ECCS, and related support systems, evaluated for failure of the ECCS components.
19. PRA Event Tree for the small break LOCA initiating event.
20. A list of PRA system dependencies and success criteria for ECCS, and its support systems.