



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

Docket No. 50-331

DISTRIBUTION:

November 23, 1992

Mr. Lee Liu
Chairman of the Board and
Chief Executive Officer
Iowa Electric Light and Power Company
Post Office Box 351
Cedar Rapids, Iowa 52406

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Dear Mr. Liu:

SUBJECT: AMENDMENT NO.190 TO FACILITY OPERATING LICENSE NO. DPR-49
(TAC NO. M80092)

The Commission has issued the enclosed Amendment No.190 to Facility Operating License No. DPR-49 for the Duane Arnold Energy Center. The license is amended by changing paragraph 2.C.(3), "Fire Protection," and by changing the Technical Specifications in response to your application dated December 13, 1990.

The amendment revises the Technical Specifications by: (1) Adopting the standard license condition for fire protection, (2) removing requirements for fire protection systems, (3) removing fire brigade staffing requirements, (4) adding administrative controls that are consistent with those for other programs implemented by a license condition.

The review of your amendment request was performed by the staff's contractor, Science Applications International Corporation (SAIC). The Technical Evaluation Report attached to the staff's Safety Evaluation was prepared by them, and reviewed and concurred in by the staff.

A copy of the related Safety Evaluation is also enclosed. Notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely, **original signed by**

Robert M. Pulsifer, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.190 to License No. DPR-49
2. Safety Evaluation with attached TER SAIC-92/6678

cc w/enclosures: See next page

LA:PD3-3:DRPW

PKreutzer

11/15/92

PM:PD3-3:DRPW

CShiraki/cs

11/15/92

D:PD3-3

JHannon

11/19/92

11/23

OGC-WF1

S. HOM

11/16/92

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P PDR

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Sincerely,

A handwritten signature in black ink, appearing to read "Robert M. Pulsifer", is written over the typed name.

Robert M. Pulsifer, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

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License No. DPR-49
2. Safety Evaluation with
attached TER SAIC-92/6678

cc w/enclosures: See next page

Mr. Lee Liu
Iowa Electric Light and Power Company

Duane Arnold Energy Center

cc:

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Kathleen H. Shea, Esquire
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Chairman, Linn County
Board of Supervisors
Cedar Rapids, Iowa 52406

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Des Moines, Iowa 50319



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

IOWA ELECTRIC LIGHT AND POWER COMPANY
CENTRAL IOWA POWER COOPERATIVE
CORN BELT POWER COOPERATIVE

DOCKET NO. 50-331

DUANE ARNOLD ENERGY CENTER

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 190
License No. DPR-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Iowa Electric Light and Power Company, et al., dated December 13, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Facility Operating License No. DPR-49 is amended by changing paragraphs 2.C.(2) and 2.C.(3) to read as follows and by changing the Technical Specifications as indicated in the attachment to this license amendment.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 190, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

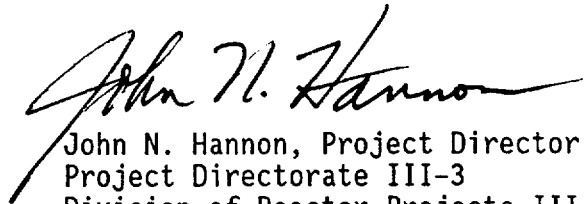
(3) Fire Protection

Iowa Light and Power Company shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the Duane Arnold Energy Center and as approved in the SER dated June 1, 1978 and Supplement dated February 10, 1981 subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

3. The license amendment is effective as of the date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John N. Hannon, Project Director
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of issuance: November 23, 1992

ATTACHMENT TO LICENSE AMENDMENT NO.190

FACILITY OPERATING LICENSE NO. DPR-49

DOCKET NO. 50-331

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

LIST OF AFFECTED PAGES

REMOVE

INSERT

iiia

iiia

vi

vi

3.13-1 through 3.13-12

3.13-1

6.2-2

6.2-2

6.5-3

6.5-3

6.11-5

6.11-5

DAEC-1

<u>LIMITING CONDITIONS FOR OPERATION</u>		<u>SURVEILLANCE REQUIREMENTS</u>	<u>PAGE NO.</u>
3.13	Deleted		
3.14	Radioactive Effluents	4.14	3.14-1
	A. Liquid Holdup Tanks	A	3.14-1
	B. Liquid Holdup Tank Instrumentation	B	3.14-2

DAEC-1

TECHNICAL SPECIFICATIONS
LIST OF TABLES (Continued)

<u>Table Number</u>	<u>Title</u>	<u>Page</u>
4.7-1	Summary Table of New Activated Carbon Physical Properties	3.7-50
4.10-1	Summary Table of New Activated Carbon Physical Properties	3.10-7
3.13-1	Deleted	
3.13-2	Deleted	
6.2-1	Minimum Shift Crew Personnel and License Requirements	6.2-3
6.11-1	Reporting Summary - Routine Reports	6.11-6

LIMITING CONDITIONS FOR OPERATION

3.13 FIRE PROTECTION SYSTEMS

SECTION 3.13 - Pages 3.13-1
through 3.13-12 have been deleted

SURVEILLANCE REQUIREMENTS

4.13 FIRE PROTECTION SYSTEMS

SECTION 4.13 - Pages 3.13-1 through
3.13-12 have been deleted

6.2.2 PLANT STAFF ORGANIZATION

The following manning requirements shall be met:

1. All CORE ALTERATIONS shall be directly supervised by either a Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
2. At all times when there is fuel in the reactor:
 - a. A senior reactor operator shall be on the plant site.
 - b. A reactor operator shall be in the control room.
 - c. Two reactor operators shall be in the control room during startup, scheduled shutdown, and during recovery from trips caused by transients or emergencies.
 - d. Minimum operating shift crew compositions shall conform to those shown in Table 6.2-1.
 - e. At least one member of each operating shift crew shall be qualified to implement radiation protection procedures.

DAEC-1

- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Manager-Nuclear Division and to the Chairman of the Safety Committee.
- f. Review of all Reportable Events.
- g. Review of facility operations to detect potential safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Safety Committee.
- i. Review of the Plant Security Plan and implementing procedures.
- j. Review of the Emergency Plan and implementing procedures.
- k. Review of every unplanned release of radioactivity to the environs for which a report to the NRC is required.
- l. Review of changes to the Offsite Dose Assessment Manual and changes to the Process Control Program.
- | m. Review of the Fire Protection Program and implementing procedures.

6.5.1.7 Authority

The Operations Committee shall:

- a. Recommend to the Plant Superintendent-Nuclear written approval or disapproval of items considered under Specification 6.5.1.6 (a) through (d) above.

DAEC-1

- e. deleted
- f. deleted
- g. deleted
- h. Radioactive Liquid or Gaseous Effluent - calculated dose exceeding specified limit (ODAM Sections 6.1.3, 6.2.3 and 6.2.4).
- i. Off-Gas System inoperable (ODAM Section 6.2.5).
- j. Measured levels of radioactivity in an environmental sampling medium determined to exceed the reporting level values of ODA Table 6.3-3 when averaged over any calendar quarter sampling period (ODAM Section 7.3.2.2).
- k. Annual dose to a MEMBER OF THE PUBLIC determined to exceed 40 CFR Part 190 dose limit (ODAM Section 6.3.1.1).
- l. Radioactive liquid waste released without treatment when activity concentration exceeds 0.01 $\mu\text{Ci/ml}$ (ODAM Section 6.1.4.1).
- m. Explosive Gas Monitoring Instrumentation Inoperable (Specification 3.2.I.1).
- n. Liquid Holdup Tank Instrumentation Inoperable (Specification 3.14.B.1).



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WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 190 TO FACILITY OPERATING LICENSE NO. DPR-49

IOWA ELECTRIC LIGHT AND POWER COMPANY
CENTRAL IOWA POWER COOPERATIVE
CORN BELT POWER COOPERATIVE

DUANE ARNOLD ENERGY CENTER

DOCKET NO. 50-331

1.0 INTRODUCTION

By letter dated December 13, 1990, the licensee requested an amendment to its operating license to remove certain fire protection related items from the Duane Arnold Technical Specifications and relocate them to the Fire Protection Program in the Updated Final Safety Analysis Report in accordance with guidance provided in Generic Letters 86-10 and 88-12. The review of the licensee's request was performed by a contractor, Science Applications International Corporation (SAIC), and is documented in the attached Technical Evaluation Report (TER) SAIC-92/6678.

2.0 EVALUATION

SAIC reviewed the licensee's request to ensure that all appropriate fire protection TS were captured in the Updated Final Safety Analysis Report (UFSAR) and Fire Protection Program prior to permitting their removal from the TS. SAIC determined that the licensee's request to relocate fire protection items from the DAEC Technical Specifications (TS) to its Fire Protection Program and the Updated Final Safety Analysis Report (UFSAR) is consistent with the guidance provided in Generic Letters 86-10 and 88-12.

SAIC did point out that the licensee's Technical Specifications (TS) do not address alternate shutdown systems and components in accordance with the guidelines of Generic Letter 81-12. Instead, the licensee has administrative procedures to maintain and control this equipment. This method of ensuring the availability of the equipment is satisfactory to the staff.

Based upon its review and a site visit on March 24, 1992, SAIC prepared a TER that states that the licensee's request to relocate fire protection items from the TS to the Fire Protection Program and the UFSAR is consistent with the guidance provided in Generic Letters 86-10 and 88-12.

The staff has reviewed SAIC's TER of the licensee's submittal which requests the relocation of fire protection items from the TS to the Fire Protection Program and the UFSAR and concurs in its evaluation. Based on the staff's evaluation of SAIC's TER, the staff concludes that the requested changes are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Iowa State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATIONS

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or a change to a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding (56 FR 33958). Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: Science Applications International Corporation
A. Singh

Date: November 23, 1992

Attachment: Technical Evaluation Report SAIC-92/6678