

March 12, 1998

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Mr. Lew W. Myers
 Vice President - Nuclear, Perry
 Centerior Service Company
 P.O. Box 97, A200
 Perry, OH 44081

SUBJECT: AMENDMENT NO. 92 TO FACILITY OPERATING LICENSE NO. NPF-58 -
 PERRY NUCLEAR POWER PLANT, UNIT 1 (TAC NO. MA0322)

Dear Mr. Myers:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 92 to Facility Operating License No. NPF-58 for the Perry Nuclear Power Plant, Unit 1. This amendment revises the Technical Specifications in response to your application dated December 23, 1997 (PY-CEI/NRR-2245L).

This amendment revises Technical Specification 3.8.1, "A.C. Sources - Operating," consistent with the recommendations in NRC Generic Letter 94-01, "Removal of Accelerated Testing and Special Reporting Requirements for Emergency Diesel Generators."

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original signed by:
 Douglas V. Pickett, Senior Project Manager
 Project Directorate III-3
 Division of Reactor Projects III/IV
 Office of Nuclear Reactor Regulation

Docket No. 50-440

Enclosures: 1. Amendment No. 92 to
 License No. NPF-58
 2. Safety Evaluation
 cc w/encls: See next page

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DATE	02/18/98		02/18/98		02/23/98		02/23/98		02/ /98	

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NAME	DPickett		EBarnhill		JCalvo					
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 12, 1998

Mr. Lew W. Myers
Vice President - Nuclear, Perry
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P.O. Box 97, A200
Perry, OH 44081

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Sincerely,

Handwritten signature of Douglas V. Pickett in cursive.

Douglas V. Pickett, Senior Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-440

Enclosures: 1. Amendment No. 92 to
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2. Safety Evaluation

cc w/encls: See next page

L. Myers
Centerior Service Company

Perry Nuclear Power Plant, Units 1 and 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CLEVELAND ELECTRIC ILLUMINATING COMPANY, ET AL.

DOCKET NO. 50-440

PERRY NUCLEAR POWER PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 92
License No. NPF-58

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Cleveland Electric Illuminating Company (CEICO), Centerior Service Company, Duquesne Light Company, Ohio Edison Company, OES Nuclear, Inc., Pennsylvania Power Company, and Toledo Edison Company (the licensees) dated December 23, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-58 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 92 are hereby incorporated into this license. The Cleveland Electric Illuminating Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented not later than 90 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Douglas V. Pickett, Senior Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: March 12, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 92

FACILITY OPERATING LICENSE NO. NPF-58

DOCKET NO. 50-440

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

Insert

3.8-5

3.8-5

3.8-6

3.8-6

3.8-16

3.8-16

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.8.1.1 Verify correct breaker alignment and indicated power availability for each required offsite circuit.	7 days
SR 3.8.1.2 -----NOTES----- 1. Performance of SR 3.8.1.7 satisfies this SR. 2. All DG starts may be preceded by an engine prelube period and followed by a warmup period prior to loading. 3. A modified DG start involving idling and gradual acceleration to synchronous speed may be used for this SR as recommended by the manufacturer. ----- Verify each DG starts from standby conditions and achieves: Voltage \geq 3900 V and \leq 4400 V and frequency \geq 58.8 Hz and \leq 61.2 Hz	31 days

(continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.8.1.3 -----NOTES-----</p> <ol style="list-style-type: none"> 1. DG loadings may include gradual loading as recommended by the manufacturer. 2. Momentary transients outside the load range do not invalidate this test. 3. This Surveillance shall be conducted on only one DG at a time. 4. This SR shall be preceded by, and immediately follow, without shutdown, a successful performance of SR 3.8.1.2 or SR 3.8.1.7. <p>-----</p> <p>Verify each DG operates for ≥ 60 minutes at a load ≥ 5600 kW and ≤ 7000 kW for Division 1 and 2 DGs, and ≥ 2600 kW for Division 3 DG.</p>	<p>31 days</p>
<p>SR 3.8.1.4 Verify each day tank contains ≥ 316 gal of fuel oil for Divisions 1 and 2 and ≥ 279 gal for Division 3.</p>	<p>31 days</p>
<p>SR 3.8.1.5 Check for and remove accumulated water from each day tank.</p>	<p>31 days</p>
<p>SR 3.8.1.6 Verify the fuel oil transfer system operates to automatically transfer fuel oil from the storage tank to the day tank.</p>	<p>31 days</p>

(continued)

NOT USED



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 92 TO FACILITY OPERATING LICENSE NO. NPF-58
CLEVELAND ELECTRIC ILLUMINATING COMPANY, ET AL.
PERRY NUCLEAR POWER PLANT, UNIT 1

DOCKET NO. 50-440

1.0 INTRODUCTION

In response to the resolution of Generic Safety Issue (GSI) B-56, "Diesel Generator Reliability," the NRC issued Generic Letter (GL) 94-01, "Removal of Accelerated Testing and Reporting Requirements for Emergency Diesel Generators," dated May 31, 1994. In this GL, the staff stated that licensees may request removal of the technical specification requirements for emergency diesel generator (EDG) accelerated testing and special reporting requirements. The GL stated that a maintenance program for monitoring and maintaining EDG performance in accordance with the provisions of the maintenance rule (10 CFR 50.65) and consistent with the guidance of Regulatory Guide 1.160 would provide a basis for the staff to approve a license amendment request to remove the accelerated testing and special reporting requirements from the technical specifications.

By letter dated December 23, 1997, the licensee for the Perry Nuclear Power Plant, Unit 1 (PNPP), proposed revisions to Technical Specification 3.8.1, "A.C. Sources - Operating," consistent with the recommendations of GL 94-01. The proposed changes would delete the accelerated testing requirements for the EDGs. During the conversion to the Improved Technical Specifications, the special reporting requirements were relocated from the technical specifications to the PNPP Operational Requirements Manual (ORM). Therefore, upon approval of this license amendment request, the licensee will remove the special reporting requirements from the ORM.

2.0 EVALUATION

The licensing basis assumes a high degree of reliability for the EDGs to start and perform their intended design function following a loss of offsite power. Technical specification surveillance requirements have historically been relied upon to demonstrate that this established reliability goal is being continually maintained. Typically, the EDGs would be tested monthly in order to demonstrate their reliability. Whenever an excessive number of monthly failures occurred, the technical specifications would require that the frequency of EDG testing be increased (i.e., accelerated). This practice would demonstrate that corrective actions were effective and provide timely test data to establish a numerical confidence level associated with EDG reliability. Concurrent with the increased frequency of testing, technical specifications would

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also require licensees to prepare and submit special test reports documenting the causes and corrective actions being taken.

GSI B-56 focused on staff concerns that accelerated testing of the EDGs was detrimental and led to long term degradation of EDG reliability. As part of the technical resolution of GSI B-56, the staff concluded that the proposed maintenance rule (i.e., 10 CFR 50.65) provided an acceptable method of ensuring EDG reliability. The elements of a program for the EDGs would include the performance of a detailed root cause analysis of individual EDG failures, effective corrective actions taken in response to individual EDG failures, and implementation of EDG preventive maintenance consistent with the maintenance rule.

When GL 94-01 was issued on May 31, 1994, the intent was that licensees could voluntarily implement the provisions of the maintenance rule for EDGs in advance of the mandatory implementation date of July 10, 1996. Implementation would follow the guidance provided in NUMARC 93-01, "Industry Guideline for Monitoring the Effectiveness of Maintenance at Nuclear Power Plants," dated May 1993, which was endorsed in Regulatory Guide 1.160, "Monitoring the Effectiveness of Maintenance at Nuclear Power Plants," dated January 1995. In exchange for these voluntary actions, both the accelerated testing requirements and special test reports could be removed from the technical specifications.

As stated above, 10 CFR 50.65 required that all licensees implement the provisions of the maintenance rule no later than July 10, 1996. In their letter dated December 23, 1997, the licensee stated that the PNPP was in compliance with the provisions of 10 CFR 50.65 and that they had implemented a maintenance program consistent with the guidance of Regulatory Guide 1.160. In Inspection Report 50-440/96014, dated January 29, 1997, the staff concluded that the licensee had developed and implemented a maintenance program that met 10 CFR 50.65.

In summary, the licensee's letter of December 23, 1997, requested removal of the accelerated testing requirements of EDGs in accordance with the recommendations of GL 94-01. The staff has determined that the licensee has implemented the provisions of the maintenance rule as described in 10 CFR 50.65. The staff concludes that the licensee has developed an appropriate program to maintain the reliability goals of the EDGs. Therefore, the staff finds the licensee's request to remove the technical specification requirements for accelerating testing of EDGs to be acceptable. As previously discussed, the special reporting requirements have previously been relocated from the technical specifications to the ORM. The staff finds that removal of these special reporting requirements, as permitted in the GL, is also acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Ohio State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or changes a

surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluent that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding (63 FR 4326). Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Douglas Pickett

Date: March 12, 1998