

September 28, 1990

Docket No. 50-331

DISTRIBUTION:

Mr. Lee Liu  
Chairman of the Board and  
Chief Executive Officer  
Iowa Electric Light and Power Company  
Post Office Box 351  
Cedar Rapids, Iowa 52406

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PDIII-3 Gray	GHill(4)
Wanda Jones	JCalvo
ACRS(10)	GPA/PA
ARM/LFMB	

Dear Mr. Liu:

SUBJECT: AMENDMENT NO. 170 TO FACILITY OPERATING LICENSE NO. DPR-49  
(TAC NO. 67893)

The Commission has issued the enclosed Amendment No. 170 to Facility Operating License No. DPR-49 for the Duane Arnold Energy Center (DAEC). This amendment consists of changes to the Technical Specifications (TS) in response to your application dated March 31, 1988.

The amendment revises the DAEC TSs by changing the iodine reporting requirements from a special report to an annual report, consistent with NRC Generic Letter 85-19, and by making other administrative changes to correct, clarify, and update several items. Other changes requested in your original submittal of March 31, 1988 concerning plant staff qualifications were addressed as a separate action in Amendment No. 166, issued July 6, 1990.

A copy of the related Safety Evaluation is also enclosed. Notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

original signed by

James R. Hall, Project Manager  
Project Directorate III-3  
Division of Reactor Projects - III,  
IV, V & Special Projects  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 170 to License No. DPR-49
2. Safety Evaluation

cc w/enclosures:  
See next page

Office: LA/PDIII-3  
Surname: PKreutzer  
Date: 8/14/90

PM/PDIII-3  
RHall/tg  
9/17/90

PD/PDIII-3  
JHannon  
9/14/90

OGC-WF1  
9/20/90

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Office: LA/PDIII-3  
Surname: PKreutzer  
Date: 8/15/90

PM/PDIII-3  
RHall/tg yeh  
9/17/90

PD/PDIII-3  
JHannon  
9/14/90

OGC-WFI  
9/20/90

Mr. Lee Liu  
Iowa Electric Light and Power Company

Duane Arnold Energy Center

cc:

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Kathleen H. Shea, Esquire  
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Washington, D.C. 20036

Chairman, Linn County  
Board of Supervisors  
Cedar Rapids, Iowa 52406

Iowa Electric Light and Power Company  
ATTN: R. Hanner  
Post Office Box 351  
Cedar Rapids, Iowa 52406

U.S. Nuclear Regulatory Commission  
Resident Inspector's Office  
Rural Route #1  
Palo, Iowa 52324

Regional Administrator, Region III  
U.S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Mr. John A. Eure  
Assistant to the Division Director  
for Environmental Health  
Iowa Department of Public Health  
Lucas State Office Building  
Des Moines, Iowa 50319



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

IOWA ELECTRIC LIGHT AND POWER COMPANY  
CENTRAL IOWA POWER COOPERATIVE  
CORN BELT POWER COOPERATIVE

DOCKET NO. 50-331

DUANE ARNOLD ENERGY CENTER

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 170  
License No. DPR-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Iowa Electric Light and Power Company, et al., dated March 31, 1988 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-49 is hereby amended to read as follows:

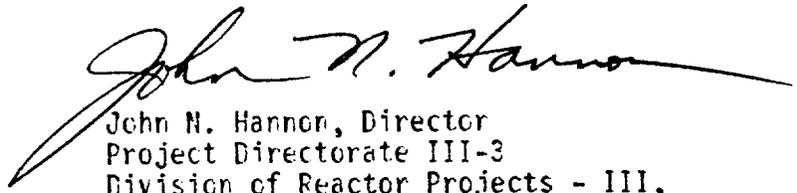
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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 170, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of the date of issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John N. Hannon, Director  
Project Directorate III-3  
Division of Reactor Projects - III,  
IV, V and Special Projects  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 28, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 170

FACILITY OPERATING LICENSE NO. DPR-49

DOCKET NO. 50-331

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

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6.11-8  
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6.11-12\*  
6.11-13\*  
6.11-15\*  
6.13-1\*

\*These pages have been deleted and may be removed

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## DAEC-1

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## TECHNICAL SPECIFICATIONS

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6.11-3a	Semiannual Radioactive Material Release Report Liquid Effluents	6.11-10
6.11-3b	Semiannual Radioactive Material Release Report Gaseous Effluents	6.11-11

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
<p>2. From and after the date that one of the RHR Service Water subsystem pumps is made or found to be inoperable for any reason, reactor operation must be limited to thirty days unless OPERABILITY of that pump is restored within this period. During such thirty days all other active components of the RHR Service Water subsystem are OPERABLE.</p>	<p>2. When it is determined that one RHR Service Water pump is inoperable, the remaining active components of that subsystem and the other subsystem shall be demonstrated to be OPERABLE immediately and daily thereafter.</p>
<p>3. From and after the date that one RHR Service Water pump in each subsystem is made or found to be inoperable for any reason, reactor operation is limited to seven days unless OPERABILITY of at least one pump is restored within this period. During such seven days all active components of both RHR Service Water subsystems and their associated diesel generators required for operation of such components (if no external source of power were available), shall be OPERABLE.</p>	<p>3. When one RHR Service Water pump in each subsystem becomes inoperable, the remaining active components of both subsystems shall be demonstrated to be OPERABLE immediately and daily thereafter.</p>
<p>4. From and after the date that one RHR Service Water subsystem is made or found to be inoperable for any reason, reactor operation is limited to seven days unless OPERABILITY of one pump is restored within this period. During such seven days all active components of the other RHR Service Water subsystem, and its associated diesel-generator required for operation of such components (if no external source of power were available), shall be OPERABLE.</p>	<p>4. When one RHR Service Water subsystem becomes inoperable, the OPERABLE subsystem shall be demonstrated to be OPERABLE immediately and daily thereafter.</p>
<p>5. If the requirements of 3.5.C cannot be met, an orderly SHUTDOWN of the reactor shall be initiated and the reactor shall be in the COLD SHUTDOWN Condition within 24 hours.</p>	

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
<p>3. The reactor vessel head bolting studs shall not be under tension unless the temperature of the vessel head flange and the head is greater than 74°F.</p> <p>4. The pump in an idle recirculation loop shall not be started unless the temperatures of the coolant within the idle and operating recirculation loops are within 50°F of each other.</p> <p>5. The reactor recirculation pumps shall not be started unless the coolant temperatures between the dome and the bottom head drain are within 145°F.</p>	<p>Test specimens of the reactor vessel base, weld and heat affected zone metal were installed in the reactor vessel adjacent to the vessel wall at the core midplane level at the start of operation. The next surveillance capsule shall be withdrawn at 15 effective full power years and tested in accordance with 10CFR50, Appendix H.</p> <p>3. When the reactor vessel head bolting studs are tensioned and the reactor is in a Cold Condition, the reactor vessel shell temperature immediately below the head flang shall be permanently recorded.</p> <p>4. Prior to and during startup of an idle recirculation loop, the temperature of the reactor coolant in the operating and idle loops shall be permanently logged.</p> <p>5. Prior to starting a recirculation pump, the reactor coolant temperatures in the dome and in the bottom head drain shall be compared and permanently logged.</p>

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
<p>2.a The reactor coolant water shall not exceed the following limits with steaming rates less than 100,000 pounds per hour, during SHUTDOWN or when in the REFUELING MODE:</p>	<p>h. Whenever the I-131 dose equivalent as determined in (e) above exceeds 0.6 <math>\mu\text{Ci/gm}</math> (50% of the equilibrium value) notify the USNRC, as specified in 6.11.1.h.</p> <p>2. A sample of reactor coolant shall be analyzed:</p> <p>a. At least every 4 hours during startup and at steaming rates below 100,000 pounds per hour for chloride ion content if the conductivity is above 0.5 <math>\mu\text{mho/cm}</math> or if it increases at a rate of 0.2 <math>\mu\text{mho/cm/hr}</math> or more. The minimum frequency will be once per day.</p>
<p>Conductivity      5 <math>\mu\text{mho/cm}</math></p>	
<p>Chloride ion      0.1 ppm</p>	
<p>At all times when the conductivity exceeds 5 micromhos/cm, the pH shall not be less than 4.6, except that short-term spikes of up to two hours duration each are permissible in the pH range 4.0 to 4.5 and of up to four hours duration each, in the range 4.5 to 4.6. The total time in which the conductivity exceeds 5 micromhos/cm shall not exceed 720 hours.</p>	

## LIMITING CONDITION FOR OPERATION

## SURVEILLANCE REQUIREMENT

- | LIMITING CONDITION FOR OPERATION  | SURVEILLANCE REQUIREMENT   |
|---|--|
| <p>2.a. From and after the date that the safety valve function of one relief valve is made or found to be inoperable, continued reactor operation is permissible only during the succeeding thirty days unless such valve function is sooner made OPERABLE.</p> | <p>2. At least one of the relief valves shall be disassembled and inspected once per operating cycle.</p>  |
| <p>b. From and after the date that the safety valve function of two relief valves is made or found to be inoperable, continued reactor operation is permissible only during the succeeding seven days unless such valve function is sooner made OPERABLE.</p>   |  |
| <p>3. If Specification 3.6.D.1 is not met, an orderly shutdown shall be initiated and the reactor coolant pressure shall be reduced to atmospheric within 24 hours.</p>   | <p>3. With the reactor pressure <math>\geq</math> 100 psig and turbine bypass flow to the main condenser, each relief valve shall be manually opened and verified open by turbine bypass valve position decrease and pressure switches and thermocouple readings downstream of the relief valve to indicate steam flow from the valve once per operating cycle.*</p> |
|   | <p>4. The relief valve setpoints for the Low-Low Set function shall be as specified in Section 2.2.1.c. Instrumentation and system logic shall be functionally tested, calibrated, and checked as required in Table 4.2-B.</p>   |

\*Intent Change Only (definition of operating cycle).

LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENT

E. Jet Pumps

1. Whenever the reactor is in the RUN mode, all jet pumps shall be OPERABLE. If the requirements of 4.6.E.1.a or .b are not met, perform the surveillance requirements of 4.6.E.2 within 24 hours. If one or more jet pumps do not meet the requirements of 4.6.E.2 and
- a. the recirculation pump speed is less than 60% of rated, continue to monitor the jet pump(s) performance per 4.6.E.2 daily until the evaluation can be performed at pump speed greater than 60%.

E. Jet Pumps

1. Jet pump operability shall be verified daily, following startup of a recirc pump and after any unexplained changes in either core flow, jet pump loop flow, recirculation loop flow, or core plate differential pressure ( $\Delta P$ ), by recording the jet pump diffuser to lower plenum  $\Delta P$ 's, recirculation pump flows, recirculation pump speeds, and jet pump loop flows and verifying that:
  - a. The recirculation pump flow to pump speed ratio does not vary from the normal expected operating range by more than 5%, and

RT<sub>NDT</sub> of 40°F is less limiting than the non-beltline regions which generally experience higher stresses at nozzles and discontinuities. The limiting RT<sub>NDT</sub> of 58°F for the Standby Liquid Control Nozzle (N10) is the highest RT<sub>NDT</sub> of any component in the non-beltline region.

The closure flange region, with RT<sub>NDT</sub> = 14°F, has a bolt preload and minimum operating temperature of 74°F. This exceeds original requirements of the ASME Code (Winter 1967 Addendum) and provides extra margin relative to current ASME Code requirements.

Neutron flux wires and samples of vessel material are installed in the reactor vessel adjacent to the vessel wall at the core midplane level. The first capsule was removed after fuel cycle 7, at 6 effective full power years. The neutron flux wires tested were used to determine the end-of-life fluence at the 1/4 T depth in the vessel wall of  $3.6 \times 10^{18}$  n/cm<sup>2</sup>. Irradiated and unirradiated Charpy specimens were tested, showing that the plate shift in NDTT was 31% higher than predicted by Regulatory Guide 1.99, Revision 1. The weld showed no measurable shift. The results of the surveillance testing, including a correction factor of 1.31 for the plate materials, have been incorporated into the curves on Figure 3.6.1.

As described in paragraph 4.2.5 of the Safety Analysis report, detailed stress analyses have been made on the reactor vessel for both steady state and transient conditions with respect to material fatigue. The results of these transients are compared to allowable stress limits. Requiring the coolant temperature in an idle recirculation loop to be within 50°F of the operating loop temperature before a recirculation pump is started assures that the changes in coolant temperature at the reactor vessel nozzles and bottom head region are acceptable.

TABLE 3.14-1  
(Continued)

TABLE NOTATION

- ACTION 18 With no channel OPERABLE, effluent may be released provided that prior to initiating a release:
1. At least two samples are analyzed in accordance with Specification 4.14.B.1, and
  2. A technically qualified member of the Facility Staff verifies the release rate calculations and discharge valving determined by another technically qualified Facility Staff member.
- Otherwise, suspend release of radioactive effluents via this pathway.
- ACTION 20 With no channel OPERABLE, effluent releases via the affected pathway may continue provided the effluent is sampled and analyzed for either gross beta/gamma or gamma isotopic radioactivity at least once per 8 hours during actual release. The analysis shall be capable of detecting  $10E-7$   $\mu\text{Ci/ml}$  gross beta/gamma or  $5 \times 10E-7$   $\mu\text{Ci/ml}$  for the principal gamma emitters and  $1 \times 10E-6$   $\mu\text{Ci/ml}$  for I-131.
- ACTION 21 With no channel OPERABLE, effluent releases via this pathway may continue provided the flow rate is estimated with pump curves at least once per batch during actual releases.
- ACTION 22 With no channel OPERABLE, suspend release of radioactive effluents via this pathway.

## g. (Continued)

- (7) Results of participation in the Interlaboratory Comparison Program.
- (8) Deviation from environmental sampling schedule.
- (9) A report of all analyses in which the LLD, required by Table 3.16-2, was not achieved.
- (10) A report of any changes in sample locations.

h. I-131 Dose Equivalent Exceeding 50% of Equilibrium Value

A report of the results of specific activity analysis in which the primary coolant exceeded the limits of Specification 4.6.B.1.h shall be submitted on an annual basis within 60 days after January 1. The following information shall be included:

- (1) Reactor power history starting 48 hours prior to the first sample in which the limit was exceeded;
- (2) Results of the last isotopic analysis for radioiodine performed prior to exceeding the limit, results of analysis while limit was exceeded and results of one analysis after the radioiodine activity was reduced to less than limit. Each result should include date and time of sampling and the radioiodine concentrations;
- (3) Cleanup system operating status starting 48 hours prior to the first sample in which the limit was exceeded;
- (4) Graph of the I-131 concentration and one other radioiodine isotope concentration in microcuries per gram as a function of time for the duration of the specific activity above the steady-state level; and
- (5) The time duration when the specific activity of the primary coolant exceeded the radioiodine limit.

## 6.11.2 Deleted

## 6.11.2 CORE OPERATING LIMITS REPORT

- a. Core cycle-dependent limits shall be established prior to each reload cycle, or prior to any remaining part of a reload cycle, for the following:
- 1) Maximum Average Planar Linear Heat Generation Rate (MAPLHGR) - Specification 3.12.A.
  - 2) Linear Heat Generation Rate (LHGR) - Specification 3.12.B.
  - 3) Minimum Critical Power Ratio (MCPR) - Specification 3.12.C.
  - 4)  $MAPFAC_f$  and  $MAPFAC_p$  Factors which multiply the MAPLHGR limits - Specification 3.6.F.2.a.

These limits shall be documented in the CORE OPERATING LIMITS REPORT.

- b. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC in General Electric Standard Application for Reactor Fuel, NEDE-24011-P-A, (GESTAR II).\*
- c. The core operating limits shall be determined such that all applicable limits (e.g. fuel thermal-mechanical limits, core thermal hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.
- d. The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements shall be provided upon issuance, for each reload cycle, to the NRC Document Control Desk with copies to the Regional Administrator and Resident Inspector.

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\* Approved revision number at time reload fuel analyses are performed.

6.11.3 UNIQUE REPORTING REQUIREMENTS

Special reports shall be submitted to the Director of Inspection and Enforcement Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification.

- a. Reactor vessel base, weld and heat affected zone metal test specimens (Specification 4.6.A.2).
- b. deleted
- c. Inservice inspection (Specification 4.6.G).
- d. Reactor Containment Integrated Leakage Rate Test (Specification 4.7.A.2.f).
- e. deleted
- f. Fire Protection Systems (Specifications 3.13.A.3, 3.13.B.2, 3.13.B.3, 3.13.C.3, and 3.13.D.3).
- g. deleted

TABLE 6.11-1  
REPORTING SUMMARY - ROUTINE REPORTS

<u>Requirement</u>	<u>Report</u>	<u>Timing of Submittal</u>
TS <sup>1</sup>	Annual Safety/ Relief Valve Challenge	Within 60 days after January 1.
TS	Annual Exposure	Within 60 days after January 1.
§20.407	Personnel Exposure and Monitoring	Within first quarter of each calendar year.
§20.408	Personnel Exposure on Termination of Employment or Work	Within 30 days after the exposure of the individual has been determined or 90 days after date of termination of employment or work assignment, whichever is earlier.
§40.64(a)	Transfer of Source Material	Promptly upon transfer.
§40.64(a)	Receipt of Source Material	Within 10 days after material is received.
§40.64(b)	Source Material Inventory	Within 30 days after September 30 of each year.

TABLE 6.11-1 (cont)

## REPORTING SUMMARY - ROUTINE REPORTS

<u>Requirement</u>	<u>Report</u>	<u>Timing of Submittal</u>
§50.59(b)	Changes, Tests, and Experiments	Within 60 days after January 1.
§70.53	Special Nuclear Material Status	Within 30 days after March 31 and September 30 of each year.
§70.54	Transfer of Special Nuclear Material	Promptly upon transfer
§70.54	Receipt of Special Nuclear Material	Within 10 days after material is received
Appendix G to 10 CFR Part 50	Fracture Toughness	On an individual-case basis at least 3 years prior to the date when the predicted fracture toughness levels will no longer satisfy section V.B. of Appendix G to 10 CFR Part 50.
Appendix H to 10 CFR Part 50	Reactor Vessel Material Surveillance	Completion of tests after each capsule withdrawal.
Appendix I to 10 CFR Part 50	Semiannual Radioactive Material Release Report	Within 60 days after January 1 and July 1.
Appendix I to 10 CFR Part 50	Annual Radiological Environmental Report	On or before May 1.
Appendix J to 10 CFR Part 50	Reactor Containment Building Integrated Leak Rate Test	Approximately 3 months following conduct of test.

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 1Technical Specifications



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 170 TO FACILITY OPERATING LICENSE NO. DPR-49

IOWA ELECTRIC LIGHT AND POWER COMPANY  
CENTRAL IOWA POWER COOPERATIVE  
CORN BELT POWER COOPERATIVE

DUANE ARNOLD ENERGY CENTER

DOCKET NO. 50-331

1.0 INTRODUCTION

By letter dated March 31, 1988, Iowa Electric Light and Power Company (the licensee) proposed changes to the Duane Arnold Energy Center (DAEC) Technical Specifications (TSs). The primary changes consist of revisions to the reporting requirements for iodine concentrations in reactor coolant, in accordance with the guidance of NRC Generic Letter 85-19. Several additional changes are proposed to correct omissions or deletions of items approved in previous amendments, to clarify wording, or to reflect current surveillance intervals. The proposed changes to update the plant staff qualifications, specified in Section 6.0, Administrative Controls, were addressed in separate correspondence and were approved in Amendment No. 166, issued July 6, 1990.

2.0 EVALUATION

The licensee has proposed to revise the reporting requirements identified in Surveillance Requirement 4.6.B.1.h of the DAEC TSs. Currently, when the iodine-131 dose equivalent exceeds 50% of the equilibrium value, a written report to the NRC Region III office is required within 10 days. Revised TS 4.6.B.1.h refers to a new specification, 6.11.1.h, which describes the revised reporting requirements. These revised requirements state that a report of the results of specific activity analysis in which the primary coolant exceeded the limits of TS 4.6.B.1.h shall be submitted (to the NRC) on an annual basis within 60 days after January 1. The contents of the report are further identified in proposed TS 6.11.1.h. This change is consistent with the guidance provided in NRC Generic Letter (GL) 85-19, "Reporting Requirements on Primary Coolant Iodine Spikes," dated September 27, 1985. The GL stated that the NRC staff had determined that the reporting requirements for iodine spiking could be reduced from a short-term report to an item which is to be included in an annual report. The content of the annual report as specified in the licensee's proposed TS 6.11.1.h is identical to the information contained in the GL. Therefore, the staff concludes that the proposed TSs 4.6.B.1.h and 6.11.1.h and the deletion of the special report requirement of TS 6.11.3.b are acceptable.

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The licensee further requested changes to TSs 6.3.1 and 6.4.1 and Figure 6.2-1 to update the standards for plant staff qualifications. These requests were superseded by letters dated November 3, 1989 and February 26, 1990; the revisions were approved by the staff in Amendment No. 166, dated July 6, 1990.

Additional changes requested by the licensee are administrative or editorial in nature and are described below. The Table of Contents, pages iv-vi, is revised to add Sections 5.6, 6.14 and 6.15, and Tables 3.14-1, 4.14-1, 4.14-2, 3.15-1, 4.15-1, 4.15-2, 3.16-1, 3.16-2, 3.16-3, 6.11-3a and 6.11-3b. The page number for Table 6.11-1 is also revised. These items were added to or revised the DAEC TS in Amendments 109, 128 and 135 (dated January 14, 1985, January 4, 1986 and August 26, 1986, respectively); however the Table of Contents was not revised to reflect the changes. These changes are editorial and are, therefore, acceptable.

Surveillance Requirements 4.5.B.2 and 3 are revised to specify the "active" components of the Residual Heat Removal (RHR) Service Water system are demonstrated to be operable as required. This is merely a clarification, consistent with the existing Bases, to indicate that the individual pumps and valves are to be demonstrated operable, as opposed to requiring a system test. An editorial change is also included to clarify that there are only two RHR Service Water subsystems. These changes are editorial and are, therefore, acceptable.

Surveillance Requirement 4.6.A.2 and the associated Bases are revised to reflect the completion of inservice inspection and testing requirements performed at the end of 6 effective-full-power years. The revision retains the requirement that the next surveillance capsule shall be withdrawn at 15 effective-full-power years and tested in accordance with 10 CFR Part 50, Appendix H. As stated in the revised Bases, the results of the surveillance testing performed following 6 effective-full-power years have been incorporated into the curves of Figure 3.6-1. With the completion of the previously required surveillance, this is an administrative change and is, therefore, acceptable.

Surveillance Requirement 4.6.D.4 was added to the DAEC TSs by Amendment 102, dated June 25, 1984, and inadvertently deleted by Amendment 105, dated August 24, 1984. Restoration of this requirement is an administrative change and is acceptable.

Table 3.14-1, Action 20 is revised to more explicitly state the radioactive effluent sampling requirements and criteria when certain liquid effluent monitors are inoperable. This clarification will make the wording of ACTION 20 consistent with TS 4.14.B.3 and Table 4.14-2. This clarification is editorial and is, therefore, acceptable.

Table 6.11-1 is revised to restore the reporting requirement for the Annual Safety/Relief Valve Challenge report approved in Amendment 96, but inadvertently deleted in Amendment 109. Also, Appendix I reporting requirements are added to Table 6.11-1. These requirements, described in TSs 6.11.1.f and g, were added by Amendment 109, but were inadvertently left out of Table 6.11-1. Also, Amendment 109 relocated Table 6.11-1 to pages 6.11-8 and 9, but failed to delete the Table from pages 6.11-12 and 13. Therefore, pages 6.11-12 and 13 can be deleted. These changes are administrative and are, therefore, acceptable.

Finally, pages 6.11-15 and 6.13-1 were deleted in Amendment 133. These pages may be removed from the DAEC TS (deleted pages 6.11-14 and 6.12-1 had previously been removed). This change is editorial and is acceptable.

On the bases discussed above, the staff finds the proposed changes to be acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATIONS

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or a change to a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). This amendment also involves changes in record-keeping, reporting or administrative procedures or requirements. Accordingly, with respect to these items, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

### 4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: James R. Hall

Dated: September 28, 1990