

July 1, 1987

Docket No. 50-331

Mr. Lee Liu
Chairman of the Board and
Chief Executive Officer
Iowa Electric Light and Power Company
Post Office Box 351
Cedar Rapids, Iowa 52406

Dear Mr. Liu:

SUBJECT: TECHNICAL SPECIFICATION CORRECTION

On January 30, 1987, the Commission issued Amendment No. 140 to Facility Operating License No. DPR-49 for the Duane Arnold Energy Center. We have recently become aware that the Commission's official copy of the Technical Specifications (TSs) contains two typographical errors on page 3.7-7 which occurred with the issuance of this amendment. One of these errors involves the inadvertent omission of a line in TS 4.7.A.2.f. To correct our records, we have reissued the enclosed corrected TS page 3.7-7.

Sincerely,

Original signed by

Anthony J. Cappucci, Project Manager
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

Enclosure:
Corrected TS page
3.7-7

cc: service list

LA/PD31:DRSP
RIngram/vl
7/1/87

AK
PM/PD31:DRSP
ACappucci:lt
7/1/87

mw
D/PD31:DRSP
MVirgilio
7/1/87

8707080149 870701
PDR ADOCK 05000331
P PDR

Mr. Lee Liu
Iowa Electric Light and Power Company

Duane Arnold Energy Center

cc:

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Mr. Thomas Houvenagle
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Distribution for DA TS Correction Memo dtd 7/ 1/87

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LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENT

3) Type C Tests

Type C tests shall be performed during each reactor shutdown for major refueling or other convenient interval but in no case at intervals greater than two years.*

4) Additional Periodic Tests

Additional purge system isolation valve leakage integrity testing shall be performed at least once every three months in order to detect excessive leakage of the purge isolation valve resilient seats. The purge system isolation valves will be tested in three groups, by penetration: drywell purge exhaust group (CV-4302 and CV-4303), torus purge exhaust group (CV-4300 and CV-4301), and drywell/torus purge supply group (CV-4307, CV-4308 and CV-4306).

e. Seal Replacement

The T-ring inflatable seals for purge isolation valves CV-4300, CV-4301, CV-4302, CV-4303, CV-4306, CV-4307 and CV-4308 shall be replaced at intervals not to exceed four years.*

The baseline for this requirement shall be established during the Cycle 6/7 refuel outage.

f. Containment Modification

Any major modification, replacement of a component which is part of the primary reactor containment boundary, or resealing a seal-welded door, performed after the preoperational leakage rate test shall be followed by either a Type A, Type B, or Type C test, as applicable, for the area affected by the modification.

*This interval may be extended, on a one-time-only basis, for Cycle 8 operation until April 17, 1987. The subsequent test interval will begin with the actual Cycle 8/9 Refuel Outage test date (Appendix J testing only).