

July 21, 1986

Docket No. 50-331

Mr. Lee Liu
Chairman of the Board and
Chief Executive Officer
Iowa Electric Light and Power Company
Post Office Box 351
Cedar Rapids, Iowa 52406

Dear Mr. Liu:

The Commission has issued the enclosed Amendment No. 134 to Facility Operating License No. DPR-49 for the Duane Arnold Energy Center. This amendment consists of changes to the Technical Specifications in response to your application dated January 27, 1984, as revised October 11, 1985 and January 13, 1986.

The amendment revises the Technical Specifications to incorporate the limiting conditions for operation for post accident containment pressure monitor, water level monitor, and hydrogen monitor.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Biweekly Federal Register Notices.

Sincerely,

/s/

Mohan C. Thadani, Project Manager
BWR Project Directorate #2
Division of BWR Licensing

Enclosures:

1. Amendment No. 134 to License No. DPR-49
2. Safety Evaluation

cc w/enclosures:
See next page

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Mr. Lee Liu
Iowa Electric Light and Power Company

Duane Arnold Energy Center

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UNITED STATES
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WASHINGTON, D. C. 20555

IOWA ELECTRIC LIGHT AND POWER COMPANY
CENTRAL IOWA POWER COOPERATIVE
CORN BELT POWER COOPERATIVE

DOCKET NO. 50-331

DUANE ARNOLD ENERGY CENTER

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 134
License No. DPR-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Iowa Electric Light and Power Company, et al, dated January 27, 1984, as revised October 11, 1985 and January 13, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-49 is hereby amended to read as follows:

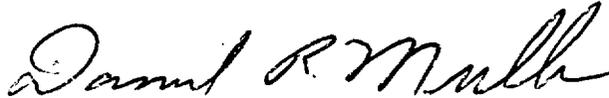
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 134, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of the date of issuance, and must be implemented in 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Daniel R. Muller, Director
BWR Project Directorate #2
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 21, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 134

FACILITY OPERATING LICENSE NO. DPR-49

DOCKET NO. 50-331

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

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TABLE 3.2-F

SURVEILLANCE INSTRUMENTATION

Total No. Channels Provided	Minimum No. Channels Required	Instrument	Type Indication and Range	Action
3	2	Reactor Water Level	Recorder, Indicator 158"-218"*	(1) (2) (3)
3	2	Reactor Pressure	Recorder, Indicator 0-1200 psig	(1) (2) (3)
2	2	Drywell Pressure	Recorder -10 to +90 psig	(1) (2) (3)
8	2	Drywell Temperature	Recorder 0-350° F	(1) (2) (3)
2	2	Torus Water Temperature	Recorder 0-350° F	(1) (2) (3)
2	2	Torus Water Level	Recorder -10"/0/+10" H ₂ O	(1) (2) (3)
2	2	Containment Water Level	Recorder, Indicator -20 to +80 feet	(1) (2) (3)
2	1	Control Rod Position	Process Computer, Full Core Display, Four Rod Group Display	
4	3	Neutron Monitoring	SRM***(10^{-1} to 10^6 CPS)	(1) (2) (3) (4)
3(per Trip System)	2(per Trip System)	Neutron Monitoring	IRM,*** APRM 0 to 125% power	(1) (2) (3) (4)
1	1	Drywell/Torus ΔP	Alarm Indicator 0-10 psid	
1	1	Drywell Pressure	Local Indicator,** 0-100 psia	(5)
1	1	Torus Pressure	Local Indicator, ** 1-100 psia	(5)

*Indicator scale is referenced to the Top of Active Fuel (TAF), defined as 344.5 inches above vessel zero.

**Capable of ±0.1 psi

***Not required when in the Run mode.

NOTES FOR TABLE 3.2-F

1. From and after the date that one of these parameters is reduced to one indication, when required, continued operation is permissible during the succeeding thirty days unless such instrumentation is sooner made operable.
2. From and after the date that one of these parameters is not indicated in the control room, continued operation is permissible during the succeeding seven days unless such instrumentation is sooner made operable.
3. If the requirements of notes (1) and (2) cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a Cold Condition within 24 hours.
4. These surveillance instruments are considered redundant to each other.
5. From and after the date that one of these parameters is not indicated, continued operation is permissible provided control room drywell pressure indication is available.

TABLE 3.2-H

ACCIDENT MONITORING INSTRUMENTATION

Instrument	Total Number of Channels Provided	Type Indication and Range	Minimum No. Channels Required	Action
Safety/Relief Valve Position Indicator (Primary Detector)	1/Valve(1)	N/A	1/Valve	(2)
Safety/Relief Valve Position Indicator (Backup-Thermocouple)	1/Valve	N/A	0	
Safety Valve Position Indicator (Primary Detector)	1/Valve(1)	N/A	1/Valve	(2)
Safety Valve Position Indicator (Backup-Thermocouple)	1/Valve	N/A	0	
Reactor Coolant, Containment Atmosphere, and Torus Water Post-Accident Sampling	2(each)	N/A	1(each)	(4)(5)
Extended Range Effluent Radiation Monitors:				/
a) Reactor Building Exhaust Stack	3	Recorder, Indicator 5×10^{-2} to 10^5 $\mu\text{Ci/cc}$	1	(6)
b) Turbine Building Exhaust Stack	1	Recorder, Indicator 5×10^{-2} to 10^5 $\mu\text{Ci/cc}$	1	(6)
c) Offgas Stack	1	Recorder, Indicator 5×10^{-2} to 10^5 $\mu\text{Ci/cc}$	1	(6)
Drywell/Torus Radiation Monitor	4	Recorder, Indicator 1 to 10^7 R/hr	2	(6)
Drywell Pressure Monitor	2	Recorder, Indicator 0-225 psig	2	(7)(8)
Drywell Pressure Monitor	2	Recorder, Indicator -5 to +5 psig	2	(7)(8)
Torus Water Level Monitor	2	Recorder, Indicator 0-30 feet	2	(9)(10)
Containment Hydrogen/Oxygen Concentration (3)	2	Recorder, Indicator 0-10% or 0-25% O_2 0-10% or 0-20% H_2 Volume oxygen/hydrogen	2	(11)(12)

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NOTES FOR TABLE 3.2-H

NOTES FOR TABLE 3.2-H

- (1) Each channel is comprised of three instruments (pressure switches) which are arranged in a "two out of three" logic connected to a relay.
- (2) From and after the date that a channel is inoperable, the torus temperature will be monitored at least once per shift to observe any unexplained temperature increase which might be indicative of an open SRV; continued reactor operation is permissible only during the succeeding 30 days, unless such channel is sooner made operable.
- (3) Normal condition is with monitor in standby mode.
- (4) When the ability to obtain a sample has been lost:
 - a. Within 7 days confirm a sample can be obtained within 24 hours of the time a decision is made to sample; and
 - b. Within 90 days, restore the sampling capability.
 - c. If the requirements of notes 4(a) and 4(b) cannot be met, be in at least a HOT SHUTDOWN Condition within the next 24 hours.
- (5) When the ability to analyze a sample has been lost:
 - a. Within 7 days, confirm that alternative sample analytical support services can be initiated within 24 hours of the time a decision is made to sample; and
 - b. Within 90 days, restore sample analysis capability.
 - c. If the requirements of notes 5(a) and 5(b) cannot be met, be in at least a HOT SHUTDOWN Condition within the next 24 hours.
- (6) With the number of OPERABLE channels (both indicator and recorder inoperable) less than the Minimum Number Channels Required, initiate the preplanned alternate method of monitoring the appropriate parameter(s) within 72 hours, and:
 - a. either restore the inoperable channel(s) to OPERABLE status within seven (7) days following the event, or
 - b. prepare and submit a Special Report to the Commission within 14 days following the event describing the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

NOTES FOR TABLE 3.2-H (cont.)

- (7) If the number of OPERABLE channels (both indicator and recorder inoperable) is reduced to less than the Minimum Number Channels Required (e.g., reduced to one channel) follow either step (a) or (b) below.
- Operation may continue for the next thirty (30) days provided at least one (1) channel of instrumentation specified in Table 3.2-F for the identical parameter is OPERABLE¹ or follow step (c) below.
 - Restore the inoperable channel to OPERABLE status within 7 days, should neither channel of instrumentation specified in Table 3.2-F for the identical parameter be OPERABLE, or follow step (c) below.
 - Within the following 12 hours be in at least HOT STANDBY and within the next 24 hours be in COLD SHUTDOWN.
- (8) If the number of OPERABLE channels (both indicator and recorder inoperable) is reduced to zero (e.g., no channels available) restore the inoperable channel(s) to OPERABLE status within 48 hours or within the following 12 hours be in at least HOT STANDBY and within the next 24 hours be in COLD SHUTDOWN.
- (9) If the number of OPERABLE channels (both indicator and recorder inoperable) is reduced to less than the Minimum Number Channels Required (e.g., reduced to one channel) follow either step (a) or (b) below.
- Operation may continue for the next thirty (30) days provided at least one torus water level channel and one containment water level channel is available.² If these conditions cannot be met, follow step (b) below.
 - Operation may continue for the next 7 days if one torus water level channel is available and there are no other containment water level channels available. If these conditions cannot be met, follow step (c) below.
 - Within the following 12 hours be in at least HOT STANDBY and within the next 24 hours be in COLD SHUTDOWN.
- (10) If the number of OPERABLE channels (both indicator and recorder inoperable) is reduced to zero (e.g., no channels available) restore at least one channel to OPERABLE status within 48 hours or within the following 12 hours be in at least HOT STANDBY and within the next 24 hours be in COLD SHUTDOWN.
- (11) If the number of OPERABLE channels (both indicator and recorder inoperable) is less than the Minimum Number Channels Required (e.g., reduced to one channel) follow either step (a) or (b) below.
- Within 30 days, increase the number of OPERABLE channels to the Minimum Number Channels Required or follow step (c) below.

¹The instruments in Table 3.2-F which measure the identical parameters are the -10 to 90 psig drywell pressure monitors.

²The containment water level monitors provide indication from -20 to +80 feet.

NOTES FOR TABLE 3.2-H (cont.)

- b. Within 30 days, and at least once every 7 days thereafter, demonstrate the ability to obtain and analyze containment samples for hydrogen and oxygen or follow step (c) below. If this sampling is done, but the number of OPERABLE channels is not increased to the Minimum Number Channels Required within 60 days from the time of initial loss, follow step (c) below.
 - c. Within the following 12 hours have the reactor in at least HOT STANDBY and within the next 24 hours have the reactor in COLD SHUTDOWN.
- (12) If the number of OPERABLE channels (both indicator and recorder inoperable) is reduced to zero (e.g., no channels available) follow either step (a) or step (b) below.
- a. Restore at least one channel to OPERABLE status within 7 days or follow step (c) below.
 - b. Within 7 days, and at least every other day thereafter, demonstrate the ability to obtain and analyze containment samples for hydrogen and oxygen or follow step (c) below. If this sampling is done, but the number of OPERABLE channels is not increased to one channel within 14 days from the time of initial loss, follow step (c) below.
 - c. Within the following 12 hours be in at least HOT STANDBY and within the next 24 hours be in COLD SHUTDOWN.

TABLE 4.2-H

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>Instrument</u>	<u>Calibration Frequency</u>	<u>Instrument Check (2)</u>
Safety/Relief Valve Position Indicator (Primary) (1)(2)	Once/operating cycle	Once/month
Safety/Relief Valve Position Indicator (Backup-Thermocouple)	Once/operating cycle	Once/month
Safety Valve Position Indicator (Primary) (1)(2)	Once/operating cycle	Once/month
Safety Valve Position Indicator (Backup-Thermocouple)	Once/operating cycle	Once/month
Drywell/Torus Radiation Monitor	Once/refueling (3)	Once/month
Extended Range Effluent Radiation Monitors		
a) Reactor Building Exhaust Stacks	Once/operating cycle (4)	Once/week
b) Turbine Building Exhaust Stack	Once/operating cycle (4)	Once/week
c) Offgas Stack	Once/operating cycle (4)	Once/week
Reactor Coolant, Containment Atmosphere, and Torus Water Post-Accident Sampling	Once/operating cycle (5)	N/A
Drywell Pressure Monitors	Once/operating cycle	Once/month
Torus Water Level Monitor	Once/operating cycle	Once/month
Containment Hydrogen/Oxygen Concentration	Once/6 months (6)	Once/month (6)

NOTES FOR TABLE 4.2-H

- Functional test of the relay is done once/3 months.
- Instrument check shall consist of the qualitative assessment of channel behavior during operation by observation. This determination shall include, where possible, comparison of the channel indication and/or status with other indications and/or status derived from independent instrument channels (e.g. backup thermocouple) measuring the same parameter.
- Channel calibration shall consist of an electronic calibration of the channel for ranges above 10 R/hr and a one point calibration check of the detector below 10 R/hr with a portable gamma source.
- Accident range effluent monitors shall be calibrated by means of a built-in check source or a known radioactive source.
- Not a calibration, but demonstration of system operability.
- Monitors shall be tested for operability using standard bottled H₂ and O₂.

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Surveillance tests other than a monthly functional check of the bus power monitors for the RHR, Core Spray, ADS, HPCI and RCIC trip systems are not required since they serve as annunciators for complete loss of power and do not monitor reduction of voltage. The subject functional check consists of opening the appropriate circuit breakers and observing the loss of power annunciator activation.

The accident monitoring instrumentation listed in Table 3.2-H were specifically added to comply with the requirements of NUREG-0737 and Generic Letter 83-36. The instrumentation listed is designed to provide plant status for accidents that exceed the design basis accidents discussed in Chapter 15 of the DAEC UFSAR.

Footnote 9 of Table 3.2-H deviates from the guidance of Generic Letter 83-36 as continued operation for 30 days (instead of 7 days as recommended in the generic letter) is allowed with one of two torus water level monitor (TWLM) channels inoperable. Continued operation is justified by the following considerations:

- 1) Redundancy is available in that at least one channel of the containment water level monitor (CWLM) instrumentation must be available. Since the CWLM envelopes the span measured by the TWLM, the torus water level can be monitored by the CWLM system.

LIMITING CONDITION FOR OPERATION

6. Containment Atmosphere Dilution
- a. Whenever the reactor is in power operation, the Post-LOCA Containment Atmosphere Dilution System must be operable and capable of supplying nitrogen to the containment for atmosphere dilution if required by post-LOCA conditions. If this specification cannot be met, the system must be restored to an operable condition within 7 days or the reactor must be taken out of power operation.
- b. Whenever the reactor is in power operation, the post-LOCA Containment Atmosphere Dilution System shall contain a minimum of 50,000 scf of N₂ as determined by pressure and temperature measurements. If this specification cannot be met, the minimum volume will be restored within 7 days or the reactor must be taken out of power operation.
- c. The limiting conditions for operation for the CAD system H₂ and O₂ analyzers serving the drywell and the suppression chamber are specified in Table 3.2-H.

SURVEILLANCE REQUIREMENT

6. Containment Atmosphere Dilution
- a. The post-LOCA containment atmosphere dilution system shall be functionally tested once per operating cycle.
- b. The volume in the N₂ storage bank shall be recorded weekly.
- c. Surveillance requirements for the CAD system H₂ and O₂ analyzers are specified in Table 4.2-H. The atmosphere analyzing system shall be functionally tested once per operating cycle in conjunction with specification 4.7.A.6.a.

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
<p>7. <u>Drywell-Suppression Chamber Differential Pressure</u></p> <p>a. Differential pressure between the drywell and suppression chamber shall be maintained at equal to or greater than 1.10 psid except as specified in (1) and (2) below:</p> <p>(1) Within the 24-hour period subsequent to placing the reactor in the Run Mode following a shutdown, the differential pressure shall be established. The differential pressure may be decreased to less than 1.10 psid 24 hours prior to a scheduled shutdown.</p> <p>(2) This differential pressure may be decreased to less than 1.10 psid for a maximum of four hours during required operability testing of the HPCI system pump, the RCIC system pump, the drywell-suppression chamber vacuum breakers, the suppression chamber to reactor building vacuum breakers, and to perform leak rate testing required by specification 4.7.A.2.d.4, and to allow for inerting operations to satisfy specification 3.7.A.5 requirements.</p> <p>b. If the differential pressure of specification 3.7.A.7.a cannot be maintained, and the differential pressure cannot be restored within the subsequent six (6) hour period, an orderly shutdown shall be initiated and the reactor shall be in a Cold Shutdown condition within the following 24 hours.</p>	<p>7. <u>Drywell-Suppression Chamber Differential Pressure</u></p> <p>a. The pressure differential between the drywell and suppression chamber shall be recorded at least once each shift.</p>



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 134 TO LICENSE NO. DPR-49

IOWA ELECTRIC LIGHT AND POWER COMPANY
CENTRAL IOWA POWER COOPERATIVE
CORN BELT POWER COOPERATIVE

DUANE ARNOLD ENERGY CENTER

DOCKET NO. 50-331

1.0 INTRODUCTION

In November 1980, the staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements," which included all TMI Action Plan items approved by the Commission for implementation at nuclear power reactors. NUREG-0737 identifies those items for which Technical Specifications (TSs) were scheduled for implementation after December 31, 1981. The staff provided guidance on the scope of Technical Specifications for all of these items in Generic Letter 83-36. Generic Letter 83-36 was issued to all Boiling Water Reactor (BWR) licensees on November 1, 1983. In this Generic Letter, the staff requested licensees to:

1. review their facility's Technical Specifications to determine if they were consistent with the guidance provided in the Generic Letter, and
2. submit an application for a license amendment where deviations or absence of Technical Specifications were found.

By letter dated January 27, 1984, Iowa Electric Light and Power Company (the licensee) responded to Generic Letter 83-36 by submitting Technical Specification change requests for the Duane Arnold Energy Center (DAEC). This evaluation covers the TMI Action Plan Items II.F.1.4, II.F.1.5, and II.F.1.6 related to containment water, pressure, and hydrogen monitors only. The remaining item III.D.3.4 related to control room habitability Technical Specification changes requested in the January 27, 1984 application will be handled in a separate action.

2.0 EVALUATION

In its January 27, 1984 application, the licensee proposed a revision to the Technical Specifications which included Technical Specification changes related to TMI Action Items II.F.1.4, II.F.1.5, and II.F.1.6.

In its January 27, 1984 application, the licensee proposed a revision to the Technical Specifications to incorporate limiting conditions for operation and surveillance requirements required by the TMI Action Items II.F.1.4, II.F.1.5, and II.F.1.6, related to post accident containment pressure, water level and hydrogen monitors. The staff's review of the licensee's proposal indicated that the proposed changes deviated from the guidance provided in Generic Letter 83-36. By a letter dated September 9, 1984, we informed the licensee of our finding and requested that the licensee resubmit its Technical Specification changes for TMI Action Items II.F.1.4, II.F.1.5, and II.F.1.6 conforming to the guidance of Generic Letter No. 83-36 as closely as possible. As a result of our September 9, 1984 request and subsequent discussions, the licensee by letters dated October 11, 1985, and January 13, 1986 revised the wording of the requested changes and gave commitments that the instruments proposed as backup to containment water level and pressure monitors were Class 1E and met Seismic Category 1 criteria.

Item II.F.1.4 - Post Accident Pressure Monitors

Our review of the licensee's October 11, 1985 submittal showed that the licensee's proposed change meets the guidance of the Generic Letter 83-36 except for the action to be taken where only one instrument channel is operable. The Generic Letter 83-36 permits plant operation for 7 days to permit the repair and restoration of second channel. The licensee has proposed that the plant can continue to operate for 30 days with at least one post accident containment pressure monitor operable and at least one backup pressure monitor operable. We judge that the 23 day time extension proposed by the licensee is reasonable and is acceptable because the backup monitor will be available if needed. The proposed Technical Specification change for Item II.F.1.4 is therefore acceptable.

Item II.F.1.5 - Post Accident Water Level Monitors

Our review of the licensee's October 11, 1985 submittal shows that the proposed change meets the guidance of the Generic Letter No. 83-36 except for the action to be taken where only one Post Accident Water Level Monitor Channel is operable. The Generic Letter permits continued operation for 7 days to permit repair and restoration of the inoperable channel. The licensee has proposed to continue operation for 30 days provided at least one proposed backup monitor is operational. We judge that the 23 day time extension for plant operation is reasonable and acceptable because at least one backup monitor will be operable and available if needed. The proposed Technical Specification change II.F.1.5 is therefore acceptable.

Item II.F.1.6 - Post Accident Hydrogen Monitors

Our review of the licensee's October 11, 1985 submittal showed that the licensee's proposed change meets the guidance of the Generic

Letter 83-36 except for the actions to be taken if only one channel is operable and if no channel is operable. The Generic Letter guidance requires that if only one channel is operable, the inoperable channel should be restored in 30 days and if no channel is operable, at least one channel be restored to operable status within 7 days. The licensee has proposed to allow the plant operation for 30 days if at least one channel is operable, and a backup is provided by the analysis of post accident sampling system. Similarly, if no channel is operable, the licensee proposes to continue plant operation for 14 days to permit restoration of at least one instrument channel to operable status, provided a backup is provided by the operable post accident sampling system for the containment hydrogen analysis.

We judge that the extension of plant operation proposed by the licensee is reasonable based on the existence and demonstrated operability of post accident sampling system. The proposed change is therefore acceptable because the post accident sampling system will be available if needed to monitor the hydrogen concentration of the containment.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: P. Kapo
Dated: July 21, 1986