

October 8, 1986

Docket No. 50-331

Mr. Lee Liu
Chairman of the Board and
Chief Executive Officer
Iowa Electric Light and Power Company
Post Office Box 351
Cedar Rapids, Iowa 52406

Dear Mr. Liu:

The Commission has issued the enclosed Amendment No. 138 to Facility Operating License No. DPR-49 for the Duane Arnold Energy Center (DAEC). This amendment consists of changes to the Technical Specifications in response to your application dated December 6, 1985.

This amendment revises the DAEC Technical Specifications to incorporate containment isolation valves for the loop B Jet Pump Sample line in Technical Specification Tables 3.7-2 and 3.7-3.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

Sincerely,

Original signed by
Mohan C. Thadani

Mohan C. Thadani, Project Manager
BWR Project Directorate #2
Division of BWR Licensing

Enclosures:

1. Amendment No. 138 to License No. DPR-49
2. Safety Evaluation

cc w/enclosures:

See next page

DISTRIBUTION

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DBL:PD#2
SNorris:rs
9/2/86

MThadani
DBL:PD#2
MThadani
10/6/86

No legal objection with changes discussed
M. Thadani
OGC - Bethesda
S H Lewis
9/16/86

DBL:PD#2
DMiller
10/7/86

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P PDR

Mr. Lee Liu
Iowa Electric Light and Power Company

Duane Arnold Energy Center

cc:

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Kathleen H. Shea, Esquire
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Mr. Thomas Houvenagle
Regulatory Engineer
Iowa Commerce Commission
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

IOWA ELECTRIC LIGHT AND POWER COMPANY
CENTRAL IOWA POWER COOPERATIVE
CORN BELT POWER COOPERATIVE

DOCKET NO. 50-331

DUANE ARNOLD ENERGY CENTER

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 138
License No. DPR-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Iowa Electric Light and Power Company, et al, dated December 6, 1985 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-49 is hereby amended to read as follows:

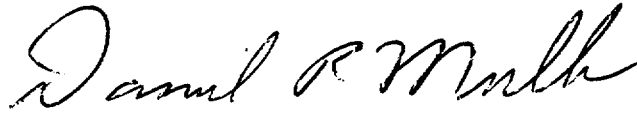
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.138 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of the date of issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "Daniel R. Muller".

Daniel R. Muller, Director
BWR Project Directorate #2
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 8, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 138

FACILITY OPERATING LICENSE NO. DPR-49

DOCKET NO. 50-331

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Pages

3.7-23

3.7-26

TABLE 3.7-2 (Continued)

CONTAINMENT ISOLATION VALVES
SUBJECT TO TYPE C TEST REQUIREMENTS

<u>PENETRATION #</u>	<u>SYSTEM</u>	<u>BOUNDARY VALVES</u>
32D	Containment Compressor Suction	CV-4378A, CV-4378B
32E	Recirc Pump "A" Seal Purge	V-17-96, CV-1804B
32F	Recirc Pump "B" Seal Purge	V-17-83, CV-1804A
35A,B,C,D	T.I.P Drives	T.I.P Ball Valves and Check Valve on X-35A
36 ¹	CRD Return	V-17-53, V-17-52, V-17-54
39A	Containment Spray/CAD Supply	SV-4332A, SV-4332B
39B	Containment Spray/CAD Supply	SV-4331A, SV-4331B
40C,D	Post-Accident Sampling/Jet Pump Sample	SV-4594A, SV-4594B, SV-4595A, SV-4595B
41	Recirc Loop Sample	CV-4639 ⁴ , CV-4640
42	Standby Liquid Control	V-26-8, V-26-9
46E	O ₂ Analyzer	SV-8105B, SV-8106B
48	Drywell Equipment Drain Discharge	CV-3728, CV-3729
50B	O ₂ Analyzer	SV-8101A, SV-8102A,
50E	O ₂ Analyzer	SV-8103A, SV-8104A,
50D	O ₂ Analyzer	SV-8105A, SV-8106A
54 ³	Reactor Building Closed Cooling Water Return	MO-4841A
55 ³	Reactor Building Closed Cooling Water Supply	MO-4841B
56C	O ₂ Analyzer	SV-8101B, SV-8102B,
56D	O ₂ Analyzer	SV-8103B, SV-8104B
205	Torus Purge Outlet	CV-4300 ⁴ , CV-4301, CV-4309
211A	Torus Spray/CAD Supply	SV-4333A, SV-4333B
211B	Torus Spray/CAD Supply	SV-4334A, SV-4334B
212 ¹	RCIC Turbine Exhaust	V-24-8 ⁴ , V-24-23 V-24-46, V-24-47
214 ¹	HPCI Turbine Exhaust	V-22-16, V-22-17 ⁴ V-22-63, V-22-64

TABLE 3.7-3 (Continued)

PRIMARY CONTAINMENT POWER OPERATED ISOLATION VALVES

Isolation Group (Note 1)	Valve Identification	Number of Power Operated Valves	Maximum Operating Time (Seconds)	Normal Position	Action on Initiating Signal
5	RWCU Supply	2	20	0	GC
5	RWCU Return	1	10	0	GC
6	Steam to HPCI Turbine	2	13	0	GC
6***	HPCI Discharge to Feedwater	1	20	C	GC
6	Steam to RCIC Turbine	2	20	0	GC
6***	RCIC Discharge to Feedwater	1	15	C	GC
8	Condensate from HPCI	2	NA	0	GC
8**	Condensate from RCIC	2	NA	0	GC
3	*Containment Compressor Discharge	3	NA	0	GC
7	*Reactor Building Closed Cooling Water Supply/Return	2	20	0	GC
7	*Well Cooling Water Supply/Return	4	NA	0	GC
9	HPCI/RCIC Exhaust Vacuum Breaker	2	10	0	GC
3	Post-Accident Sampling Liquid Sample Return	2	NA	C	SC
3	Post-Accident Sampling Jet Pump Sample	4	NA	C	SC

*Due to plant operational limitations, these valves will be subject to the requirements of 4.7.D.1.a only.

**Low-Low Water Level Only

***These valves close only upon sensing closure of their respective turbine steam supply or turbine stop valve closure.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 138 TO FACILITY OPERATING LICENSE NO. DPR-49

IOWA ELECTRIC LIGHT AND POWER COMPANY
CENTRAL IOWA POWER COOPERATIVE
CORN BELT POWER COOPERATIVE

DUANE ARNOLD ENERGY CENTER

DOCKET NO. 50-331

1.0 INTRODUCTION

By letter dated December 6, 1985, Iowa Electric Light and Power Company (the licensee) proposed to revise Technical Specification Tables 3.7-2 and 3.7-3 to include the containment isolation valves for the loop B jet pump sample line in the Post Accident Sampling System (PASS). This sample line was added as part of the long-term modifications to the PASS. The loop B sample line is functionally identical to the loop A jet pump sample line whose containment isolation valves are listed in Tables 3.7-2 and 3.7-3 of the present Technical Specifications.

2.0 EVALUATION

Modifications for loop B jet pump PASS sample line have added two new primary containment isolation valves to Duane Arnold Energy Center. (Modifications were found acceptable in our Safety Evaluation dated June 11, 1985). Compliance with the Commission regulations 10 CFR 50, Appendix J, requires that the new isolation valves be included in the Technical Specifications and be subject to Type C testing. Accordingly, the licensee has proposed to add the new valves to the Technical Specifications Table 3.7-2 related to containment isolation valves subject to Type C Test Requirements, and Table 3.7-3 related to Primary Containment Power Operated Isolation Valves characteristics. Our evaluation finds that the incorporation of the new valves in the Technical Specifications meets the requirements of the Commission's regulations and is acceptable.

3.0 ENVIRONMENTAL CONSIDERATIONS

This amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding.

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Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
(2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: F. Witt

Dated: October 8, 1986