

June 11, 1985

Docket No. 50-331

Mr. Lee Liu
Chairman of the Board and
Chief Executive Officer
Iowa Electric Light and Power Company
Post Office Box 351
Cedar Rapids, Iowa 52406

Dear Mr. Liu:

The Commission has issued the enclosed Amendment No. 123 to Facility Operating License No. DPR-49 for the Duane Arnold Energy Center. This amendment consists of changes to the Technical Specifications in response to your application dated December 5, 1984, as clarified by your letter dated January 24, 1985.

This amendment revises the Technical Specifications to (1) permit changing the well cooling water backwash automatic valves to manual valves and to keep them locked shut, (2) correct some inconsistencies in the present specifications and as-built logic circuits for Groups 6 and 7 containment isolation valves, and (3) make the Technical Specifications clearer and more complete.

Of the 23 changes requested by you, we have sufficient information to approve 16 items in this amendment. The remaining seven items (3, 4, 5, 6, 13, 16 and 22) which require additional information from you, will be the subject of a future action, after the requisite information has been received.

A copy of the Safety Evaluation supporting our conclusions for the 16 approved items is also enclosed.

Sincerely,

Original signed by/

Mohan C. Thadani, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosures:

- Amendment No. 123 to License No. DPR-49
- Safety Evaluation

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
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IOWA ELECTRIC LIGHT AND POWER COMPANY
CENTRAL IOWA POWER COOPERATIVE
CORN BELT POWER COOPERATIVE

DOCKET NO. 50-331

DUANE ARNOLD ENERGY CENTER

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 123
License No. DPR-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Iowa Electric Light and Power Company, et al, dated December 5, 1984, as clarified by letter dated January 24, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-49 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 123, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 11, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 123

FACILITY OPERATING LICENSE NO. DPR-49

DOCKET NO. 50-331

Revise the Appendix A Technical Specifications by removing the current pages and inserting the revised pages listed below. The revised areas are identified by vertical lines.

LIST OF AFFECTED PAGES

3.2-5
3.2-5a
3.2-8
3.2-9
3.2-24
3.7-22
3.7-23
3.7-24
3.7-26
3.7-29
3.7-29a

TABLE 3.2-A
INSTRUMENTATION THAT INITIATES PRIMARY CONTAINMENT ISOLATION

Minimum No. of Operable Instrument Channels Per Trip System (1)	Instrument	Trip Level Setting	Number of Instrument Channels Provided by Design	Valve Groups Operated by Signal	Action (2)
2 (6)	Reactor Low Water Level	$> +170''$ Indicated Level (3)	4	2,3,4,5 (Sec. Cont., 3	A E)
1	Reactor Low Pressure (Shutdown Cooling Isolation)	≤ 135 psig	2	4	C
2	Reactor Low-Low-Low Water Level	$> +18.5''$ indicated Level (3)	4	1	A
2 (6)	High Drywell Pressure	≤ 2.0 psig	4	2,3,4,8,9* (Sec. Cont., 3	A E)
2	High Radiation Main Steam Line Tunnel	≤ 3 X Normal Rated Power Background	4	1	B
2	Low Pressure Main Steam Line	≥ 850 psig (7)	4	1	B
2 (5)	High Flow Main Steam Line	$\leq 140\%$ of Rated Steam Flow	4	1	B
2	Main Steam Line Tunnel/Turbine Bldg. High Temperature	$\leq 200^\circ$ F.	4	1	B
1	Reactor Cleanup System High Diff. Flow	≤ 40 gpm	2	5	D

*Group 9 valves isolate on high drywell pressure combined with reactor steam supply low pressure

TABLE 3.2-A

INSTRUMENTATION THAT INITIATES PRIMARY CONTAINMENT ISOLATION (continued)

Minimum No. of Operable Instrument Channels Per Trip System (1)	Instrument	Trip Level Setting	Number of Instrument Channels Provided by Design	Valve Groups Operated by Signal	Action (2)
1	Reactor Cleanup Area Ambient High Temperature	130°F	3	5	D
1	Reactor Cleanup Area Differential High Temperature	Δ 14°F*	3	5	D
2	Loss of Main Condensor Vacuum	\leq 10 in Hg Vacuum	4	1	B
2	Reactor Low-Low Water Level	$>$ +119.5" indicated Level (3)	4	8	A

*Note: The actual setpoint shall be Δ 14°F above the 100% operation ambient temperature conditions as determined by DAEC Plant Test Procedure.

Amendment No. ~~110~~, 123

3.2-8

TABLE 3.2-B

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT COOLING SYSTEMS

Minimum No. of Operable Instrument Channels Per Trip System (1)	Trip Function	Trip Level Setting	Number of Instrument Channels Provided by Design	Remarks
2	Reactor Low-Low Water Level	> + 119.5 in. indicated Level (4)	4 HPCI & RCIC Instrument Channels 4 LPCI loop select Instrument Channels	Initiates HPCI & RCIC Initiates LPCI loop select logic
2	Reactor Low-Low-Low Water Level	> + 18.5 in. indicated Level (4)	4 Core Spray & RHR Instrument Channels 4 ADS Instrument Channels	<ol style="list-style-type: none"> 1. In conjunction with Low Reactor Pressure initiates operation of Core Spray and LPCI valves. Starts pumps if not already started from 2 psig drywell signal. 2. In conjunction with confirmatory low level High Drywell Pressure, 120 second time delay and LPCI or Core Spray pump interlock initiates Auto Blowdown (ADS) 3. Initiates starting of Diesel Generator 4. Closes group 7 isolation valves
2	Reactor High Water Level	< + 211 in. indicated Level (4)	2 Instrument Channels	Trips HPCI and RCIC turbines

TABLE 3.2-B (Continued)

INSTRUMENTATION THAT INITIATES OR CONTROLS THE CORE AND CONTAINMENT COOLING SYSTEMS

Minimum No. of Operable Instrument Channels Per Trip System (1)	Trip Function	Trip Level Setting	Number of Instrument Channels Provided by Design	Remarks
1	Reactor Low Level (inside shroud)	> + 305.5 in. above Vessel zero (2/3 core height)	2 Instrument Channels	Prevents inadvertent operation of containment spray during accident condition
2	Containment High Pressure	$1 < p < 2$ psig	4 Instrument Channels	Prevents inadvertent operation of containment spray during accident condition
1	Confirmatory Low Level	$\leq + 170$ in. indicated level (4)	2 Instrument Channels	ADS Permissive
2	High Drywell Pressure	≤ 2.0 psig	4 HPCI Instrument Channels	Initiates Core Spray, LCPI and HPCI pumps. In conjunction with 450 psig reactor pressure, initiates operation of LPCI and core spray valves.
2	Reactor Low Pressure	≥ 450 psig	4 Instrument Channels	Permissive for open Core Spray and LCPI Injection valves. (High drywell pressure starts LPCI and Core Spray pumps). In conjunction with triple low water level, cycles core spray and LPCI injection valves open.

TABLE 4.2-A
MINIMUM TEST AND CALIBRATION FREQUENCY FOR PCIS

<u>Instrument Channel (5)</u>	<u>Instrument Functional Test (9)</u>	<u>Calibration Frequency (9)</u>	<u>Instrument Check</u>
1) Reactor Low Pressure (Shutdown Cooling Permissive)	(1)	Once/3 months	None
2) Reactor Low-Low Water Level	(1)	Once/3 months	Once/shift
3) Main Steam High Temp.	(1)	Once/operating cycle	Once/day
4) Reactor Low Water Level	(1)	Once/operating cycle	Once/shift
5) Main Steam High Flow	(1)	Once/3 months	Once/shift
5) Main Steam Low Pressure	(1)	Once/3 months	None
7) Reactor Water Cleanup High Flow (7)	(1)	Once/3 months	Once/day
8) High Drywell Pressure	(1)	Once/3 months	None
9) Reactor Cleanup Area High Temp. (8)	(1)	Once/operating cycle	None
10) High Radiation Main Steam Line Tunnel	(1)	Once/operating cycle	Once/shift
11) Loss of Main Condenser Vacuum	(1)	Once/operating cycle	None
<u>Logic System Functional Test (4) (6)</u>			
1) Main Steam Line Isolation Valves Main Steam Line Drain Valves Reactor Water Sample Valves		Once/6 months	
2) RHR - Isolation Valve Control Shutdown Cooling Valves Head Spray		Once/6 months	
3) Reactor Water Cleanup Isolation		Once/6 months	

TABLE 3.7-2

CONTAINMENT ISOLATION VALVES
SUBJECT TO TYPE C TEST REQUIREMENTS

<u>PENETRATION #</u>	<u>SYSTEM</u>	<u>BOUNDARY VALVES</u>
7A	Main Steam Line	CV-4412 ⁴ , 4413
7B	Main Steam Line	CV-4415 ⁴ , 4416
7C	Main Steam Line	CV-4418 ⁴ , 4419
7D	Main Steam Line	CV-4420 ⁴ , 4421
8	Main Steam Line Drain	MO-4424
9A	Feedwater & HPCI Feed	V-14-3
9A ²	Feedwater & HPCI Feed	MO-4441, MO-2312
9B	Feedwater	V-14-1
9B ²	Feedwater & RCIC Feed & RWCU Return	MO-2740, MO-4442, MO-2512
10	RCIC Condensate Return	CV-2411
10	Steam to RCIC Turbine	MO-2401
11	Steam to HPCI Turbine	MO-2239
11	HPCI Condensate Return	CV-2212
15	RWCU Supply	MO-2700, MO-2701
16A	Core Spray Pump Discharge	MO-2115, MO-2117
16B	Core Spray Pump Discharge	MO-2135, MO-2137
19	Drywell Floor Drain Discharge	CV-3704, CV-3705
20	Demineralized Water Supply	V-09-65, V-09-111
21	Service Air Supply	V-30-287, Blind Flange
22, 229	Containment Compressor Discharge	CV-4371A, CV-4371C, V-43-214
23A ³ , B ³	Well Cooling Water Supply	CV-5718A, CV-5718B, V-57-75, V-57-76,
24A ³ , B ³	Well Cooling Water Return	CV-5704A, CV-5704B, V-57-77, V-57-78
25	Drywell Purge Outlet	CV-4302 ⁴ , CV-4303, CV-4310
26, 220	Drywell and Torus Purge Supply	CV-4306, CV-4307 ⁴ , CV-4308 ⁴
26, 220	Drywell and Torus Nitrogen Makeup	CV-4311, CV-4312, CV-4313

TABLE 3.7-2 (Continued)

CONTAINMENT ISOLATION VALVES
SUBJECT TO TYPE C TEST REQUIREMENTS

<u>PENETRATION #</u>	<u>SYSTEM</u>	<u>BOUNDARY VALVES</u>
32D	Containment Compressor Suction	CV-4378A, CV-4378B
32E	Recirc Pump "A" Seal Purge	V-17-96, CV-1804B
32F	Recirc Pump "B" Seal Purge	V-17-83, CV-1804A
35A,B,C,D	T.I.P Drives	T.I.P Ball Valves and Check Valve on X-35A
36 ¹	CRD Return	V-17-53, V-17-52, V-17-54
39A	Containment Spray/CAD Supply	SV-4332A, SV-4332B
39B	Containment Spray/CAD Supply	SV-4331A, SV-4331B
40D	Post-Accident Sampling/Jet Pump Sample	SV-4594A, SV-4594B
41	Recirc Loop Sample	CV-4639 ⁴ , CV-4640
42	Standby Liquid Control	V-26-8, V-26-9
46E	O ₂ Analyzer	SV-8105B, SV-8106B
48	Drywell Equipment Drain Discharge	CV-3728, CV-3729
50B	O ₂ Analyzer	SV-8101A, SV-8102A,
50E	O ₂ Analyzer	SV-8103A, SV-8104A,
50D	O ₂ Analyzer	SV-8105A, SV-8106A
54 ³	Reactor Building Closed Cooling Water Return	MO-4841A
55 ³	Reactor Building Closed Cooling Water Supply	MO-4841B
56C	O ₂ Analyzer	SV-8101B, SV-8102B,
56D	O ₂ Analyzer	SV-8103B, SV-8104B
205	Torus Purge Outlet	CV-4300 ⁴ , CV-4301, CV-4309
211A	Torus Spray/CAD Supply	SV-4333A, SV-4333B
211B	Torus Spray/CAD Supply	SV-4334A, SV-4334B
212 ¹	RCIC Turbine Exhaust	V-24-8 ⁴ , V-24-23 V-24-46, V-24-47
214 ¹	HPCI Turbine Exhaust	V-22-16, V-22-17 ⁴ V-22-63, V-22-64

NOTES TO TABLE 3.7-2

¹Test volume is filled with demineralized water then pressurized to 1.10 P_a with air or nitrogen for test. For all other penetrations (except 7A-D), test volumes are pressurized to P_a with air or nitrogen for test.

²MO-4441, MO-4442 will be remote manually closed.

³In accordance with 10 CFR 50, Appendix A, General Design Criterion 57, the redundant barriers are a single isolation valve outside containment and a closed system inside. Testing of the single isolation valve only is required. Manual valves V-57-75, V-57-76, V-57-77 and V-57-78 will be normally locked closed.

⁴Tested in reverse direction.

TABLE 3.7-3 (Continued)

PRIMARY CONTAINMENT POWER OPERATED ISOLATION VALVES

Isolation Group (Note 1)	Valve Identification	Number of Power Operated Valves	Maximum Operating Time (Seconds)	Normal Position	Action on Initiating Signal
5	RWCU Supply	2	20	0	GC
5	RWCU Return	1	10	0	GC
6	Steam to HPCI Turbine	2	13	0	GC
6***	HPCI Discharge to Feedwater	1	20	C	GC
6	Steam to RCIC Turbine	2	20	0	GC
6***	RCIC Discharge to Feedwater	1	15	C	GC
8	Condensate from HPCI	2	NA	0	GC
8**	Condensate from RCIC	2	NA	0	GC
3	*Containment Compressor Discharge	3	NA	0	GC
7	*Reactor Building Closed Cooling Water Supply/Return	2	20	0	GC
7	*Well Cooling Water Supply/Return	4	NA	0	GC
9	HPCI/RCIC Exhaust Vacuum Breaker	2	10	0	GC
3	Post-Accident Sampling Liquid Sample Return	2	NA	C	SC
3	Post-Accident Sampling Jet Pump Sample	2	NA	C	SC

*Due to plant operational limitations, these valves will be subject to the requirements of 4.7.D.1.a only.

**Low-Low Water Level Only

***These valves close only upon sensing closure of their respective turbine steam supply or turbine stop valve closure.

Group 6:

The valves in Group 6 are closed upon any signal representing a steam line break in the HPCI system's or RCIC system's respective steam line.

Group 7:

The valves in Group 7 are closed upon low-low-low reactor water level signal. (Note: The level sensors utilized for this function are part of the core and containment cooling logic.)

Group 8:

The valves in Group 8 are closed upon any of the following conditions:

1. Reactor vessel low-low water level
2. High drywell pressure

Group 9:

The valves in Group 9 auto-isolate on the combined trip of both reactor steam supply low pressure (PS-N001A-D) and high drywell pressure (PSE11-N011A-D).

KEY: 0 = Open
C = Closed
SC = Stays Closed
GC = Goes Closed



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 123 TO LICENSE NO. DPR-49

IOWA ELECTRIC LIGHT AND POWER COMPANY
CENTRAL IOWA POWER COOPERATIVE
CORN BELT POWER COOPERATIVE

DUANE ARNOLD ENERGY CENTER

DOCKET NO. 50-331

1.0 INTRODUCTION

By letters dated December 5, 1984 and January 24, 1985, Iowa Electric Light and Power Company (IELP), the licensee for the Duane Arnold Energy Center (DAEC), requested changes to the Technical Specifications resulting from the discovery of an error in the logic circuits that actuate the Well Cooling Water Automatic Backwash Valves. To correct the logic circuit problem, the IELP requested that the Well Cooling Water Automatic Backwash Valves be changed to manual valves and kept locked shut. IELP has observed that the backwash valves have never been used in the operating history of DAEC and no future need for use of these valves is foreseen. Additionally, the licensee has observed some inconsistencies between the as-built DAEC Group 6 and 7 containment isolation valve logic circuits and the DAEC Technical Specifications (TS). To correct this problem, the licensee has requested additional Technical Specification changes to reflect the as-built logic circuits. Other changes are requested to make the Technical Specifications clearer, more concise, or complete.

Of the 23 changes requested in the licensee's applications dated December 5, 1984 and January 24, 1985, seven items (3, 4, 5, 6, 13, 16 and 22) require additional information from the licensee, and will be evaluated in a separate action. The remaining 16 items have been evaluated in this Safety Evaluation (SE).

2.0 EVALUATION

The 16 items of requested Technical Specification changes evaluated in this SE (identified with the item numbers used in the licensee's application) are:

- (1) In Table 3.2-A on page 3.2-5, the trip level setting for Reactor Low-low-low Water Level is made consistent with the rest of the table by replacing "at or above" with "greater than."
- (2) In Table 3.2-A on page 3.2-5a, the trip level setting for Reactor Low-low-low Water Level is reworded to be consistent with the format of the rest of the table.

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- (7) In Table 3.2-B on page 3.2-8, low pressure coolant injection (LPCI) loop select information is added in columns 4 and 5 for the Reactor Low-low-low Water Level trip function to better reflect as-designed instrumentation.
- (8) In Table 3.2-B on page 3.2-8, Remark 1 for the Reactor Low-low-low Water Level trip function is clarified to distinguish the operation of the valves from the operation of the pumps.
- (9) In Table 3.2-B on page 3.2-8, Remark 4 is added to indicate that the Group 7 isolation valves are closed upon receipt of a Reactor Low-low-low Water Level signal. This is an administrative change, since this information already appears in the notes for Table 3.7-3 on page 3.7-29.
- (10) On pages 3.2-9 and 3.2-10, the numbering of remarks is deleted since each trip function has only one remark.
- (11) In Table 3.2-B on page 3.2-9, the remarks for High Drywell Pressure and Reactor Low Pressure Trip functions are modified to clarify their relation to each other and with Reactor Low-low-low Water Level.
- (12) In Table 4.2-A on page 3.2-24, a spelling error is corrected on item 4. In the Instrument Check column "One" is replaced by "Once."
- (14) In Table 3.7-2 on page 3.7-22, the four Well Cooling Water Automatic Isolation Valves, CV-5719A, CV-5719B, CV-5703A, CV-5703B, are changed to Manual Valves, V-57-75, V-57-76, V-57-77, V-57-78, to be consistent with hardware change to the well water backwash valves.
- (15) In Table 3.7-2 on page 3.7-23, boundary valve V-17-54 is added to the entries for control rod drive (CRD) return. It was removed in error by a previous amendment.
- (17) In Table 3.7-2 on page 3.7-23, the listing of boundary valves is adjusted to make the listing more organized and easier to read.
- (18) In Table 3.7-3, for clarity the one asterisk note is moved from page 3.7-29a to page 3.7-26.
- (19) In Table 3.7-3 on page 3.7-26, the three-asterisk note is added to clarify the operation of the high pressure coolant injection (HPCI) Discharge to Feedwater valve and the reactor core isolation cooling (RCIC) Discharge to Feedwater valve.
- (20) In Table 3.7-3 on page 3.7-26, the number of Well Cooling Water Automatic Isolation Valves is changed from 8 to 4 to be consistent with hardware change. The 4 Well Cooling Water Backwash Valves are being changed to Manual Valves, but the 4 Well Cooling Water Automatic Isolation Valves used for normal Well Water Cooling are being left intact.

- (21) In the notes to Table 3.7-3 on page 3.7-29, the following remark is added to the Group 7 Containment Isolation Valve actuation description: "(Note: The level sensors utilized for this function are part of the core and containment cooling logic)."
- (23) In the notes to Table 3.7-2 on page 3.7-24, the sentence "Manual valves V-57-75, V-56-76, V-57-77 and V-57-78 will be normally closed" is added to conform to the action prescribed in conjunction with the hardware change HA1.

The evaluation of the proposed modification of the 4 well water valves and the above 16 items of Technical Specification changes is as follows:

Having the proposed Containment Isolation Valves locked closed is equivalent to a current requirement that the open valves be capable of automatic closure by an isolation signal. We find the requested change to be acceptable.

- (1) This is an administrative change, and thus we find it acceptable.
- (2) This is an administrative change, and thus we find it acceptable.
- (7, 8 and 11) These Technical Specification changes to the "remarks" column of Table 3.2-B (Instrumentation that initiates or Controls the Core and Containment Cooling Systems) are for clarification purposes only. No Technical Specification requirements are affected. The added remarks more clearly identify the safety-related equipment actuated by existing protection system instrumentation channels, which have been previously reviewed and accepted by the NRC. Therefore, we find these Technical Specification changes to be acceptable.
- (9) As is stated in the description of this addition to the Technical Specifications, this addition simply repeats information already in the Technical Specifications, and thus we find it acceptable.
- (10) This is an administrative change, and thus we find it acceptable.
- (12) This is an administrative change, and thus we find it acceptable.
- (14) The Manual Isolation Valve numbers are correct, and thus we find the change of valve numbers to be acceptable. However, the manual valves are improperly labelled in the Technical Specifications (valves are correctly identified in the plant). Valves V-57-75 and V-57-76 should be labelled "Well Cooling Backwash Return" and valves V-57-77 and V-57-78 should be labelled "Well Cooling Backwash Supply." IELP has committed to a future submittal of an administrative Technical Specification amendment application to correct this mislabelling.
- (15) This change corrects a previous error in the Technical Specifications, and thus we find the change acceptable.

- (17) This change rearranges Table 3.7-2 without changing any of the information, and thus we find the change acceptable.
- (18) This change is an administrative change, and thus we find it acceptable.
- (19) We find the addition of the three-asterisk footnote acceptable, since it describes exactly what signal actuates the 2 Discharge to Feedwater valves in Group 6, namely a signal that the corresponding steam turbine inlet or outlet valves have closed. However, in the notes for Technical Specifications Table 3.7-3 the specific signals which actuate the steam turbine valves are not listed, but are specified to as "any signal representing a steam line break." IELP has committed to a future submittal of an administrative Technical Specification amendment application to add the precise signals which actuate the closure of the steam turbine valves to the notes for Table 3.7-3.
- (20) This change is consistent with the modifications of the well water backwash valves discussed above, and thus we find it acceptable.
- (21) This added note is a clarification with no action associated with it, and thus we find it acceptable.
- (23) This addition is in conformance with hardware change to the well water backwash valves, and thus we find this addition to the Technical Specifications acceptable. However, to make the statement in the Technical Specifications more concise, we requested that the Technical Specifications be modified to read "Manual valves V-57-75, V-57-76, V-57-77 and V-57-78 will be normally locked closed." IELP has added the word "locked" to this sentence, and the Technical Specification page has been appropriately corrected.

3.0 ENVIRONMENTAL CONSIDERATIONS

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public

will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: P. Kapo

Dated: June 11, 1985