

August 22, 1984

Docket No. 50-331

Mr. Lee Liu
Chairman of the Board and
Chief Executive Officer
Iowa Electric Light and Power Company
Post Office Box 351
Cedar Rapids, Iowa 52406

Dear Mr. Liu:

The Commission has issued the enclosed Amendment No. 104 to Facility Operating License No. DPR-49 for the Duane Arnold Energy Center. This amendment consists of changes to the Technical Specifications in response to your application dated January 27, 1984.

The January 27, 1984 application requested several changes related to NUREG-0737 requirements described in the NRC Generic Letter 83-36, and other miscellaneous items. This amendment relates only to Item II.B.3 - Post Accident Sampling and also corrects two typographical errors on page 3.2-23b of your Technical Specifications. Other items in the January 27, 1984 application will be handled in separate actions.

A copy of the related Safety Evaluation is also enclosed.

Sincerely,

Original signed by/

Mohan C. Thadani, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosures:

1. Amendment No.104 to License No. DPR-49
2. Safety Evaluation

cc w/enclosures:
See next page

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08/6/84	08/7/84	08/7/84	08/10/84	08/21/84

(no legal objection to letter & Amendment)

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Mr. Lee Liu
Iowa Electric Light and Power Company
Duane Arnold Energy Center

cc:

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Agency
Region VII Office
Regional Radiation Representative
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U. S. Nuclear Regulatory Commission
Resident Inspector's Office
Rural Route #1
Palo, Iowa 52324

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Region III Office
U. S. Nuclear Regulatory Commission
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

IOWA ELECTRIC LIGHT AND POWER COMPANY
CENTRAL IOWA POWER COOPERATIVE
CORN BELT POWER COOPERATIVE

DOCKET NO. 50-331

DUANE ARNOLD ENERGY CENTER

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 104
License No. DPR-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Iowa Electric Light & Power Company, et al, dated January 27, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-49 is hereby amended to read as follows:

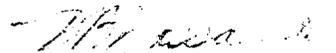
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 104, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 22, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 104

FACILITY OPERATING LICENSE NO. DPR-49

DOCKET NO. 50-331

Revise the Appendix "A" Technical Specifications by removing the current pages and inserting the revised pages listed below. The revised areas are identified by vertical lines.

LIST OF AFFECTED PAGES

3.2-23b

3.2-34a

NOTES FOR TABLE 3.2-H

NOTES FOR TABLE 3.2-H

- (1) Each channel is comprised of three instruments (pressure switches) which are arranged in a "two out of three" logic connected to a relay.
- (2) From and after the date that a channel is inoperable, the torus temperature will be monitored at least once per shift to observe any unexplained temperature increase which might be indicative of an open SRV; continued reactor operation is permissible only during the succeeding 30 days, unless such channel is sooner made operable.
- (3) When the ability to obtain a sample has been lost:
 - a. Within 7 days confirm a sample can be obtained within 24 hours of the time a decision is made to sample; and
 - b. Within 90 days, restore the sampling capability.
 - c. If the requirements of notes 3 (a) and 3 (b) cannot be met, be in at least a HOT SHUTDOWN Condition within the next 24 hours
- (4) When the ability to analyze a sample has been lost:
 - a. Within 7 days, confirm that alternative sample analytical support services can be initiated within 24 hours of the time a decision is made to sample; and
 - b. Within 90 days, restore sample analysis capability.
 - c. If the requirements of notes 4 (a) and 4 (b) cannot be met, be in at least a HOT SHUTDOWN Condition within the next 24 hours.
- (5) With the number of operable channels (both indicator and recorder inoperable) less than the Minimum Channels Operable Requirement, initiate the preplanned alternate method of monitoring the appropriate parameter(s) within 72 hours, and:
 - a. either restore the inoperable channel(s) to operable status within seven (7) days following the event, or
 - b. prepare and submit a Special Report to the Commission within 14 days following the event describing the action taken, the cause of the inoperability and the plans and schedule for restoring the system to operable status.

TABLE 4.2-II

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>Instrument</u>	<u>Calibration Frequency</u>	<u>Instrument Check (2)</u>
Safety/Relief Valve Position Indicator (Primary) (1)(2)	Once/operating cycle	Once/month
Safety/Relief Valve Position Indicator (Backup-Thermocouple)	Once/operating cycle	Once/month
Safety Valve Position Indicator (Primary) (1)(2)	Once/operating cycle	Once/month
Safety Valve Position Indicator (Backup-Thermocouple)	Once/operating cycle	Once/month
Extended Range Effluent Radiation Monitors:		
a) Reactor Building Exhaust Stacks	Once/operating cycle (3)	Once/week (
b) Turbine Building Exhaust Stack	Once/operating cycle (3)	Once/week
c) Offgas Stack	Once/operating cycle (3)	Once/week
Reactor Coolant, Containment Atmosphere, and Torus Water Post-Accident Sampling	Once/operating cycle (4)	N/A

NOTES FOR TABLE 4.2-H

1. Functional test of the relay is done once/3 months.
2. Instrument check shall consist of the qualitative assessment of channel behavior during operation by observation. This determination shall include, where possible, comparison of the channel indication and/or status with other indications and/or status derived from independent instrument channels (e.g. backup thermocouple) measuring the same parameter.
3. Accident range effluent monitors shall be calibrated by means of a built-in check source or a known radioactive source.
4. Not a calibration, but demonstration of system operability.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 104 TO LICENSE NO. DPR-49

IOWA ELECTRIC LIGHT AND POWER COMPANY
CENTRAL IOWA POWER COOPERATIVE
CORN BELT POWER COOPERATIVE

DUANE ARNOLD ENERGY CENTER

DOCKET NO. 50-331

1.0 Introduction

In November 1980, the staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements," which included all TMI Action Plan items approved by the Commission for implementation at nuclear power reactors. NUREG-0737 identifies those items for which Technical Specifications (TSs) were scheduled for implementation after December 31, 1981. The staff provided guidance on the scope of Technical Specifications for all of these items in Generic Letter 83-36. Generic Letter 83-36 was issued to all Boiling Water Reactor (BWR) licensees on November 1, 1983. In this Generic Letter, the staff requested licensees to:

1. review their facility's Technical Specifications to determine if they were consistent with the guidance provided in the Generic Letter, and
2. submit an application for a license amendment where deviations or absence of Technical Specifications were found.

By letter dated January 27, 1984, Iowa Electric Light and Power Company (the licensee) responded to Generic Letter 83-36 by submitting Technical Specification change requests for the Duane Arnold Energy Center (DAEC). This evaluation covers the TMI Action Plan Item II.B.3 - Post Accident Sampling only. Other items requested in the January 27, 1984 application will be handled in separate actions.

2.0 Evaluation

In its January 27, 1984 application, the licensee proposed a revision to the Technical Specifications which included TMI Action Item II.B.3 related to the post accident sampling system. The proposed Technical Specification changes for the limiting condition of operation and surveillance requirements on the post accident sampling system satisfy the guidance provided in Generic Letter 83-36. Therefore, we conclude that the licensee has acceptable Technical Specifications for the Post Accident Sampling System.

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3.0 Environmental Considerations

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 Conclusion

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: C. Patel

Dated: August 22, 1984