

September 4, 1984

Docket No. 50-331

Mr. Lee Liu
Chairman of the Board and
Chief Executive Officer
Iowa Electric Light and Power Company
Post Office Box 351
Cedar Rapids, Iowa 52406

Dear Mr. Liu:

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The Commission has issued the enclosed Amendment No. 107 to Facility Operating License No. DPR-49 for the Duane Arnold Energy Center. This amendment consists of changes to the Technical Specifications in response to your application dated January 27, 1984.

The amendment revises the Technical Specifications to incorporate changes to the Duane Arnold Energy Center (DAEC) Technical Specifications. The January 27, 1984 application requested several changes related to NUREG-0737 requirements described in the NRC Generic Letter 83-36, and other miscellaneous items. This amendment relates to Item II.F.1.3 involving containment high-range monitors only. Other items in the January 27, 1984 application will be handled in separate actions.

A copy of the related Safety Evaluation is also enclosed.

Sincerely,

Original signed by:
Mohan C. Thadani, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosures:

- Amendment No. 107 to License No. DPR-49
- Safety Evaluation

cc w/enclosures:
See next page

DL:ORB#2
SNorris:jk
8/23/84
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AD/RE
DMuller
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immediately pass to issuing amendment check to see there are no comments or

hearing request If there are, report back to (617)

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Mr. Lee Liu
Iowa Electric Light and Power Company
Duane Arnold Energy Center

cc:

Jack Newman, Esquire
Harold F. Reis, Esquire
Newman and Holtzinger
1025 Connecticut Avenue, N. W.
Washington, D. C. 20036

Mr. Thomas Houvenagle
Regulatory Engineer
Iowa Commerce Commission
Lucas State Office Building
Des Moines, Iowa 50319

Office for Planning and Programming
523 East 12th Street
Des Moines, Iowa 50319

Chairman, Linn County
Board of Supervisors
Cedar Rapids, Iowa 52406

Iowa Electric Light and Power Company
ATTN: D. L. Mineck
Post Office Box 351
Cedar Rapids, Iowa 52406

U. S. Environmental Protection
Agency
Region VII Office
Regional Radiation Representative
324 East 11th Street
Kansas City, Missouri 64106

U. S. Nuclear Regulatory Commission
Resident Inspector's Office
Rural Route #1
Palo, Iowa 52324

James G. Keppler
Regional Radiation Representative
Region III Office
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

IOWA ELECTRIC LIGHT AND POWER COMPANY
CENTRAL IOWA POWER COOPERATIVE
CORN BELT POWER COOPERATIVE

DOCKET NO. 50-331

DUANE ARNOLD ENERGY CENTER

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 107
License No. DPR-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Iowa Electric Light & Power Company, et al, dated January 27, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-49 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance: September 4, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 107

FACILITY OPERATING LICENSE NO. DPR-49

DOCKET NO. 50-331

Revise the Appendix A Technical Specifications by removing the current page and inserting the revised page listed below. The revised areas are identified by vertical lines.

LIST OF AFFECTED PAGES

3.2 - 34a

TABLE 4.2-II

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>Instrument</u>	<u>Calibration Frequency</u>	<u>Instrument Check (2)</u>
Safety/Relief Valve Position Indicator (Primary) (1)(2)	Once/operating cycle	Once/month
Safety/Relief Valve Position Indicator (Backup-Thermocouple)	Once/operating cycle	Once/month
Safety Valve Position Indicator (Primary) (1)(2)	Once/operating cycle	Once/month
Safety Valve Position Indicator (Backup-Thermocouple)	Once/operating cycle	Once/month
Drywell/Torus Radiation Monitor	Once/refueling (3)	Once/month
a) Reactor Building Exhaust Stacks	Once/operating cycle (4)	Once/week
b) Turbine Building Exhaust Stack	Once/operating cycle (4)	Once/week
c) Offgas Stack	Once/operating cycle (4)	Once/week
Reactor Coolant, Containment Atmosphere, and Torus Water Post-Accident Sampling	Once/operating cycle (5)	N/A

NOTES FOR TABLE 4.2-II

1. Functional test of the relay is done once/3 months.
2. Instrument check shall consist of the qualitative assessment of channel behavior during operation by observation. This determination shall include, where possible, comparison of the channel indication and/or status with other indications and/or status derived from independent instrument channels (e.g. backup thermocouple) measuring the same parameter.
3. Channel calibration shall consist of an electronic calibration of the channel for ranges above 10 R/hr and a one point calibration check of the detector below 10 R/hr with a portable gamma source.
4. Accident range effluent monitors shall be calibrated by means of a built-in check source or a known radioactive source.
5. Not a calibration, but demonstration of system operability.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 107 TO LICENSE NO. DPR-49

IOWA ELECTRIC LIGHT AND POWER COMPANY
CENTRAL IOWA POWER COOPERATIVE
CORN BELT POWER COOPERATIVE

DUANE ARNOLD ENERGY CENTER

DOCKET NO. 50-331

1.0 Introduction

By letter dated January 27, 1984, the Iowa Electric Light and Power Company (licensee) requested changes to Duane Arnold Energy Center (DAEC) Technical Specifications. The licensee's request was in response to the requirements of NUREG-0737 as outlined in the NRC Generic Letter 83-36. The request also contained some other miscellaneous items. This Safety Evaluation relates to NUREG-0737 action item II.F.1.3 only. Other items requested in the January 27, 1984 application will be considered in separate actions.

2.0 Evaluation

In the January 27, 1984 application, the licensee requested changes to the DAEC Technical Specifications for TMI Action Items outlined in Generic Letter 83-36. The requested changes included monitoring and surveillance requirements for Item II.F.1.3 related to Containment High-Range Radiation Monitor. The proposed changes for Item II.F.1.3 meet our guidance provided in Generic Letter 83-36 and are, therefore, acceptable.

3.0 Environmental Considerations

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

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4.0 Conclusion

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: M. Lamastra

Dated: September 4, 1984