

March 10, 1983

Docket No. 50-331

Mr. Duane Arnold
Chairman of the Board and Chief
Executive Officer
Iowa Electric Light and Power Company
P. O. Box 351
Cedar Rapids, Iowa 52406

Dear Mr. Arnold:

The Commission has issued the enclosed Amendment No. 83 to Facility Operating License No. DPR-49 for the Duane Arnold Energy Center. This amendment consists of changes to the Technical Specifications in response to your application dated August 20, 1982, as revised November 24, 1982.

This change to the Technical Specifications allows the licensee to perform a manual stroke test on the mechanical snubbers in the forthcoming February 1983 refueling outage in lieu of the required functional test.

Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Frank L. Apicella, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosures:

1. Amendment No. 83 to DPR-49
2. Safety Evaluation
3. Notice

cc w/enclosures
See next page

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Mr. Duane Arnold
Iowa Electric Light & Power Company

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

IOWA ELECTRIC LIGHT AND POWER COMPANY
CENTRAL IOWA POWER COOPERATIVE
CORN BELT POWER COOPERATIVE

DOCKET NO. 50-331

DUANE ARNOLD ENERGY CENTER

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 83
License No. DPR-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Iowa Electric Light & Power Company, et al, dated August 20, 1982 as revised November 24, 1982 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-49 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 83, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 10, 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 83

FACILITY OPERATING LICENSE NO. DPR-49

DOCKET NO. 50-331

Revise the Appendix A Technical Specifications by removing page 3.6-13 and inserting revised pages 3.6-13 and 3.6-13A. The revised area is identified by a vertical line.

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENT

both the failed snubber (if it is repaired and installed in another position) and the spare snubber shall be retested. Test results of these snubbers may not be included for the re-sampling.

If any snubber selected for functional testing either fails to lockup or fails to move, i.e., frozen in place, the cause will be evaluated and if caused by manufacturer or design deficiency all snubbers of the same design subject to the same defect shall be functionally tested. This testing requirement shall be independent of the requirements stated above for snubbers not meeting the functional test acceptance criteria.

For the snubber(s) found inoperable, an engineering evaluation shall be performed on the components which are supported by the snubber(s).

The purpose of this engineering evaluation shall be to determine if the components supported by the snubber(s) were adversely affected by the inoperability of the snubber(s) in order to ensure that the supported component remains capable of meeting the designed service.

Note: The functional tests of mechanical snubbers described in this section will be implemented during the first scheduled refueling outage beginning after June 30, 1983. In the interim, a manual stroke test will be substituted for the functional test.

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENT4. Hydraulic Snubbers Functional
Test Acceptance Criteria

The hydraulic snubber functional test shall verify that:

1. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 83 TO LICENSE NO. DPR-49

IOWA ELECTRIC LIGHT AND POWER COMPANY
CENTRAL IOWA POWER COOPERATIVE
CORN BELT POWER COOPERATIVE

DOCKET NO. 50-331

DUANE ARNOLD ENERGY CENTER

1.0 Introduction

By letter dated August 20, 1982, as revised on November 24, 1982, the Iowa Electric Light and Power Company, (the licensee), submitted a request to amend the Facility Operating License DPR-49 for the Duane Arnold Energy Center. The proposed amendment will permit the licensee to perform a manual stroke test on the mechanical snubbers in the forthcoming February 1983 refueling outage in lieu of the required functional test.

The justification for delaying the implementation of the surveillance requirements for functional testing of mechanical snubbers until the first scheduled refueling outage beginning after June 30, 1983 is the lack of availability of a reliable test machine for conducting the required tests.

2.0 Evaluation

During the February 1983 refueling outage, the licensee proposes to perform a manual stroke test of a representative sample of mechanical snubbers in place of the functional test required by the Technical Specifications. Section 4.6.H.3 of the Technical Specifications states, that at least once every eighteen months, during shutdown, a representative sample of each type of snubber in use in the plant shall be functionally tested either in place or in a bench test. The licensee states that the implementation of this surveillance requirement for functional testing of mechanical snubbers cannot be accomplished because of the lack of availability of a reliable test machine. The licensee appears to have made a reasonable effort to obtain the equipment necessary for the performance of functional tests, and has determined that the equipment will not be available during the February 1983 refueling outage. The licensee proposes a manual stroke test as the most meaningful alternative.

In our review of the proposed specification change, we evaluated the mechanical snubber functional test acceptance criteria. The acceptance criteria of Section 4.6.H.5 of the technical specifications require that, the force that initiates free movement of the snubber rod in either tension or compression is less than the specified maximum drag force; the drag force on any snubber shall not have increased more than 50% since the last functional

test; activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression; and snubber release rate, where required, is within the specified range in compression or tension, or the ability of the snubber to withstand load without displacement.

Manual stroke testing will provide sufficient assurance that the mechanical snubbers will remain free to move as required for normal plant operation and anticipated transients. The fact that the snubbers have the capability to move through the required range of motion will confirm that design loading in piping and other fluid systems components will not be exceeded during operation. The absence of unusual sounds or other abnormalities in snubber motion that are detected during manual stroke testing also offers evidence that the snubber is capable of performing until the next refueling outage beginning after June 30, 1983.

Based on the above we conclude that delaying the plant schedule beyond the schedule outage would be unwarranted and that the licensee's proposal is acceptable.

3.0 Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

4.0 Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a safety margin, the amendment does not involve a significant hazards consideration (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 10, 1983

Principal Contributor: H. Shaw
F. L. Apicella

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-331IOWA ELECTRIC LIGHT AND POWER COMPANY, ET ALNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 83 to Facility Operating License No. DPR-49 issued to Iowa Electric Light and Power Company, Central Iowa Power Cooperative, and Corn Belt Power Cooperative, which revises the Technical Specifications for operation of the Duane Arnold Energy Center, located in Linn County Iowa. The amendment is effective as of its date of issuance.

This change to the Technical Specifications allows the licensee to perform a manual stroke test on the mechanical snubbers in the forthcoming February 1983 refueling outage in lieu of the required functional test.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d)(4) and environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated August 20, 1982 as revised November 24, 1982 (2) Amendment No. 83 to License No. DPR-49, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Cedar Rapids Public Library, 426 Third Avenue, S.E., Cedar Rapids, Iowa 52401. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 10th day of March 1983.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing