

Docket No. 50-331

Mr. Duane Arnold, President
Iowa Electric Light and
Power Company
P. O. Box 351
Cedar Rapids, Iowa 52406

Dear Mr. Arnold:

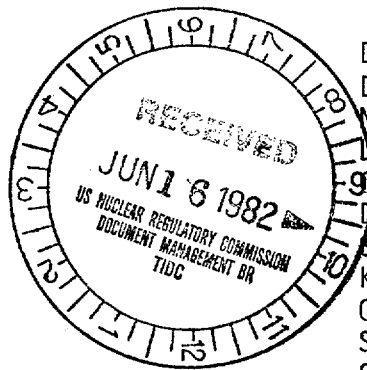
The Commission has issued the enclosed Amendment No. 75 to Facility Operating License No. DPR-49 for the Duane Arnold Energy Center. This amendment consists of changes to the Technical Specifications in response to your application dated April 23, 1982.

The Technical Specification changes incorporate revisions to the tables of containment isolation valves to reflect modifications made to satisfy the requirements of NUREG-0737, Item II.B.3 "Post Accident Sampling."

As a result of these modifications, additional valves, which are containment isolation valves and are subject to 10 CFR 50 Appendix J leak test requirements have been added. Iowa Electric has proposed modifying Tables 3.7-2 (Containment Isolation Valves Subject to Type C Test Requirements) and 3.7-3 (Primary Containment Power Operated Isolation Valves) of the Duane Arnold Technical Specifications to reflect the addition of these valves. We have reviewed these proposed changes and find them acceptable, since they correct the list of valves to which the Technical Specifications apply.

We have evaluated the potential for environmental impact of plant operation in accordance with the enclosed amendment and have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Based on the foregoing we have determined that the amendment does not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. It does not involve a significant



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increase in the probability or consequences of an accident, does not involve a significant decrease in a safety margin and, therefore, does not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

A copy of the related Notice of Issuance is also enclosed.

Sincerely,

Original signed by

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:

1. Amendment No. 75 to DPR-49
2. Notice of Issuance

cc w/encs:

See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

IOWA ELECTRIC LIGHT AND POWER COMPANY
CENTRAL IOWA POWER COOPERATIVE
CORN BELT POWER COOPERATIVE

DOCKET NO. 50-331

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 75
License No. DPR-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Iowa Electric Light & Power Company, et al, dated April 23, 1982 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-49 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 75, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

Mr. Duane Arnold
Iowa Electric Light & Power Company

cc:

Mr. Robert Lowenstein, Esquire
Harold F. Reis, Esquire
Lowenstein, Newman, Reis and Axelrad
1025 Connecticut Avenue, N. W.
Washington, D. C. 20036

Office for Planning and Programming
523 East 12th Street
Des Moines, Iowa 50319

Chairman, Linn County
Board of Supervisors
Cedar Rapids, Iowa 52406

Iowa Electric Light & Power Company
ATTN: D. L. Mineck
P. O. Box 351
Cedar Rapids, Iowa 52406

U.S. Environmental Protection Agency
Region VII Office
Regional Radiation Representative
324 East 11th Street
Kansas City, Missouri 64106

Cedar Rapids Public Library
428 Third Avenue, S.E.
Cedar Rapids, Iowa 52401

U.S. Nuclear Regulatory Commission
Resident Inspector's Office
Rural Route #1
Palo, Iowa 52324

James G. Keppler
Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in dark ink, appearing to read "D. Vassallo", is written over the typed name.

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 9, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 75

FACILITY OPERATING LICENSE NO. DPR-49

DOCKET NO. 50-331

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3.7-23
3.7-26

Insert

3.7-23
3.7-26

TABLE 3.7-2 (Continued)

CONTAINMENT ISOLATION VALVES
SUBJECT TO TYPE C TEST REQUIREMENTS

| <u>PEN #</u> | | <u>BOUNDARY VALVES</u> |
|------------------|---|--|
| 25 | Drywell Purge Outlet | CV-4302, CV-4303, CV-4310 |
| 26, 220 | Drywell and Torus Purge Supply | CV-4306, CV-4307, CV-4308 |
| 26, 220 | Drywell and Torus Nitrogen Makeup | CV-4311, CV-4312, CV-4313 |
| 32D | Containment Compressor Suction | CV-4378A, CV-4378B |
| 32E | Recirc Pump "A" Seal Purge | V-17-96, V-17-84 |
| 32E | Recirc Pump "A" Seal Purge | CV-1804B, V-17-84 |
| 32F | Recirc Pump "B" Seal Purge | V-17-83, V-17-80 |
| 32F | Recirc Pump "B" Seal Purge | CV-1804A, V-17-80 |
| 36 ¹ | CRD Return | V-17-54, V-17-52 |
| 36 ¹ | CRD Return | V-17-54, V-17-53 |
| 41 | Recirc Loop Sample | CV-4639, CV-4640 |
| 46E | O ₂ Analyzer | SV-8105B, SV-8106B |
| 48 | Drywell Equipment Drain Discharge | CV-3728, CV-3729 |
| 50B,E,D | O ₂ Analyzer | SV-8101A, SV-8102A, SV-8103A, SV-8104A, SV-8105A, SV-8106A |
| 54 | Reactor Building Closed Cooling Water Return | MO-4841A, V-12-64, V-12-65, V-12-68 |
| 55 | Reactor Building Closed Cooling Water Supply | MO-4841B, V-12-62, V-12-63, V-12-66 |
| 56C,D | O ₂ Analyzer | SV-8101B, SV-8102B, SV-8103B, SV-8104B |
| 205 | Torus Purge Outlet | CV-4300, CV-4301, CV-4309 |
| 229B,C,G,F | O ₂ Analyzer | SV-8107A, SV-8108A, SV-8109A, SV-8110A, SV-8107B, SV-8108B, SV-8109B, SV-8110B |
| 231 | Torus Vacuum Breakers | CV-4304, ZS-4329 |
| 231 | Torus Vacuum Breakers | CV-4305, ZS-4330 |
| 219 ³ | HPCI/RCIC Exhaust Vacuum Breaker | MO-2290A, MO-2290B, V-22-60 |
| 209A | Post-Accident Sampling System Liquid Sample Return | SV-8772A, SV-8772B |

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-331IOWA ELECTRIC LIGHT AND POWER COMPANY, ET ALNOTICE OF ISSUANCE OF AMENDMENT TO FACILITYOPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 75 to Facility Operating License No. DPR-49 issued to Iowa Electric Light and Power Company, Central Iowa Power Cooperative, and Corn Belt Power Cooperative, which revises the Technical Specifications for operation of the Duane Arnold Energy Center (DAEC), located in Linn County, Iowa. The amendment is effective as of its date of issuance.

The amendment modifies the Technical Specifications to incorporate revisions to the tables of containment isolation valves to reflect modifications made to satisfy the requirements of NUREG-0737 Item II.B.3 "Post Accident Sampling."

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d)(4) an environmental impact statement or negative declaration

and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated April 23, 1982, (2) Amendment No. 75 to License No. DPR-49, and (3) the Commission's letter to Iowa Electric Light and Power Company dated June 9, 1982 . All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D. C. and at the Cedar Rapids Public Library, 426 Third Avenue, S.E., Cedar Rapids, Iowa 52401. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 9th day of June 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

TABLE 3.7-3 (Continued)
PRIMARY CONTAINMENT POWER OPERATED ISOLATION VALVES

| Isolation Group (Note 1) | Valve Identification | Number of Power Operated Valves | Maximum Operating Time (Seconds) | Normal Position | Action on Initiating Signal |
|--------------------------------|--|--|---|--------------------|-----------------------------------|
| 5 | RWCU Supply | 2 | 20 | 0 | GC |
| 5 | RWCU Return | 1 | 10 | 0 | GC |
| 6 | Steam to HPCI Turbine | 2 | 13 | 0 | GC |
| 6 | HPCI Discharge to Feedwater | 1 | 20 | C | GC |
| 6 | Steam to RCIC Turbine | 2 | 20 | 0 | GC |
| 6 | RCIC Discharge to Feedwater | 1 | 15 | C | GC |
| 8 | Condensate from HPCI | 2 | NA | 0 | GC |
| 8** | Condensate from RCIC | 2 | NA | 0 | GC |
| 3 | *Containment Compressor Discharge | 3 | NA | 0 | GC |
| 7 | *Reactor Building Closed Cooling Water Supply/Return | 2 | 20 | 0 | GC |
| 7 | *Well Cooling Water Supply/Return | 8 | NA | 0 | GC |
| 9 | HPCI/RCIC Exhaust Vacuum Breaker | 2 | 10 | 0 | GC |
| 3 | Post-Accident Sampling Liquid Sample Return | 2 | NA | C | SC |

**Low-Low Water Level Only