

JUNE 21 1982

Docket No. 50-331

NRC PDR  
Local PDR  
ORB#2 Rdg  
D. Eisenhut  
OELD  
OI&E (2)  
S. Miner  
K. Eccleston  
S. Norris  
T. Barnhart (4)  
L. Schneider  
D. Brinkman  
ACRS (10)  
OPA  
R. Diggs  
NSIC  
ASLAB *Gray*

Mr. Duane Arnold, President  
Iowa Electric Light and Power Company  
Post Office Box 351  
Cedar Rapids, Iowa 52406

Dear Mr. Arnold:

The Commission has issued the enclosed Amendment No. 77 to Facility Operating license No. DPR-49 for the Duane Arnold Energy Center (DAEC). This amendment consists of changes to the Technical Specifications (TS) in final response to your application dated June 17, 1980.

The Technical Specification changes incorporate a detailed definition of the terms OPERABLE and Limiting Condition for Operation (LCO) as requested in our April 10, 1980 letter to all power reactor licensees.

Your proposed TS changes did not incorporate the general Action statement of the Standard Technical Specifications (STS) for BWRs (NUREG-0123 Rev.3), concerning operation under circumstances in excess of those addressed in the specifications. We have reviewed the necessity of having such a statement in the DAEC TS and find that the existing TS were formulated to preserve the single failure criterion for systems that are relied upon in the Duane Arnold Safety Analysis Report. Your existing TS contain LCOs which (1) require that all redundant components of safety-related systems be OPERABLE and (2) specify appropriate Actions to be taken in the event that the minimum operability requirements are not satisfied. Therefore, we agree that this change is not necessary for DAEC.

We have reviewed these proposed Technical Specification changes and have determined that you have incorporated the definition of the terms OPERABLE and Limiting Conditions for Operation from NUREG 0123 Rev. 3. The Standard Technical Specifications pages 1-4 and 3/4 0-1, pertaining to the definition of the term Operable and the applicability of associated Limiting conditions for Operation, are recognized by the staff as an acceptable means of implementing our requirements concerning this issue.

Since your existing Technical Specifications were formulated to preserve the single failure criterion for systems relied upon in the DAEC Safety Analysis Report and your proposed TS changes incorporate the applicable specifications of NUREG-0123 Rev. 3, we conclude that these changes are acceptable.

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P PDR

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We have evaluated the potential for environmental impact of plant operation in accordance with the enclosed amendment and have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR 51.5(d) (4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Based on the foregoing we have determined that the amendment does not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. It does not involve a significant increase in the probability or consequences of an accident, does not involve a significant decrease in a safety margin and, therefore, does not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

A copy of the related Notice of Issuance is also enclosed.

Sincerely,

ORIGINAL SIGNED BY

Domenic B. Vassallo, Chief  
 Operating Reactors Branch #2  
 Division of Licensing

Enclosures:

1. Amendment No. 77 to DPR-49
2. Notice of Issuance

cc: w/enclosures  
 See next page

*No legal objection to notice + amendment*

OFFICE	DL: ORB#2	DL: ORB#2 KJE	DL: ORB#2	DL: ORB#4	DL: OR	OELD	
SURNAME	SNOFFTS	KECCTESTON:pob	DBVASSATTO	SMINER SM	INOVAK	W.D. PATON	
DATE	6/10/82	6/14/82	6/14/82	6/14/82	6/15/82	6/16/82	

Mr. Duane Arnold  
Iowa Electric Light & Power Company

cc:

Mr. Robert Lowenstein, Esquire  
Harold F. Reis, Esquire  
Lowenstein, Newman, Reis and Axelrad  
1025 Connecticut Avenue, N. W.  
Washington, D. C. 20036

Office for Planning and Programming  
523 East 12th Street  
Des Moines, Iowa 50319

Chairman, Linn County  
Board of Supervisors  
Cedar Rapids, Iowa 52406

Iowa Electric Light & Power Company  
ATTN: D. L. Mineck  
P. O. Box 351  
Cedar Rapids, Iowa 52406

U.S. Environmental Protection Agency  
Region VII Office  
Regional Radiation Representative  
324 East 11th Street  
Kansas City, Missouri 64106

Cedar Rapids Public Library  
428 Third Avenue, S.E.  
Cedar Rapids, Iowa 52401

U.S. Nuclear Regulatory Commission  
Resident Inspector's Office  
Rural Route #1  
Palo, Iowa 52324

James G. Keppler  
Regional Administrator, Region III  
U.S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, IL 60137



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

IOWA ELECTRIC LIGHT AND POWER COMPANY  
CENTRAL IOWA POWER COOPERATIVE  
CORN BELT POWER COOPERATIVE

DOCKET NO. 50-331

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 77  
License No. DPR-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Iowa Electric Light & Power Company, et al, dated June 17, 1980 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-49 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.77, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Enclosures:  
Changes to the Technical  
Specifications

Date of Issuance: June 21, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 77

FACILITY OPERATING LICENSE NO. DPR-49

DOCKET NO. 50-331

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

REMOVE

1.0-1

1.0-2

INSERT

1.0-1

1.0-2

## 5. OPERABLE-OPERABILITY

A system, subsystem, train, component or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified function(s). Implicit in this definition shall be the assumption that all necessary attendant instrumentation, controls, normal and emergency electrical power sources, cooling or seal water, lubrication or other auxiliary equipment that are required for the system, subsystem, train, component or device to perform its function(s) are also capable of performing their related support function(s).

## 6. OPERATING

Operating means that a system or component is performing its intended functions in its required manner.

## 7. IMMEDIATE

Immediate means that the required action will be initiated as soon as practicable considering the safe operation of the unit and the importance of the required action.

## 8. REACTOR POWER OPERATION

Reactor power operation is any operation with the mode switch in the "Startup" or "Run" position with the reactor critical and above 1% rated power.

## 9. HOT STANDBY CONDITION

Hot standby condition means operation with coolant temperature greater than 212°F, reactor vessel pressure less than 1035 psig, and the mode switch in the Startup/Hot Standby position.

## 10. COLD CONDITION

Reactor coolant temperature equal to or less than 212°F.

## 11. HOT SHUTDOWN

The reactor is in the shutdown mode and the reactor coolant temperature greater than 212°F.

## 12. COLD SHUTDOWN

The reactor is in the shutdown mode, the reactor coolant temperature equal to or less than 212°F, and the reactor vessel is vented to atmosphere.

## 1.0 DEFINITIONS

The succeeding frequently used terms are explicitly defined so that a uniform interpretation of the specifications may be achieved.

1. SAFETY LIMIT

The safety limits are limits below which the reasonable maintenance of the cladding and primary systems are assured. Exceeding such a limit requires unit shutdown and review by the Atomic Energy Commission before resumption of unit operation. Operation beyond such a limit may not in itself result in serious consequences but it indicates an operational deficiency subject to regulatory review.

2. LIMITING SAFETY SYSTEM SETTING (LSSS)

The limiting safety system settings are settings on instrumentation which initiate the automatic protective action at a level such that the safety limits will not be exceeded. These settings take into consideration the instrumentation tolerances and the instruments are required to be periodically calibrated as specified in these Technical Specifications. The limiting safety system setting plus the tolerance of the instrument as given in the system design control document gives the limiting trip point for operation. This additional margin has been established so that with proper operation of the instrumentation the safety limits will never be exceeded. The inequality sign which may be given merely signifies the preferred direction of operational trip setting.

3. LIMITING CONDITIONS FOR OPERATION (LCO)

The limiting conditions specify the minimum acceptable levels of system performance necessary to assure safe startup and operation of the facility. When these conditions are met, the plant can be operated safely and abnormal situations can be safely controlled.

When a system, subsystem, train, component or device is determined to be inoperable solely because its emergency power source is inoperable, or solely because its normal power source is inoperable, it may be considered OPERABLE for the purpose of satisfying the requirements of its applicable Limiting Condition for Operation, provided: (1) its corresponding normal or emergency power source is OPERABLE; and (2) all of its redundant system(s), subsystem(s), train(s), components(s) and device(s) are OPERABLE, or likewise satisfy the requirements of this specification.

4. DELETED

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-331IOWA ELECTRIC LIGHT AND POWER COMPANY, ET ALNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (The Commission) has issued Amendment No. 77 to Facility Operating License No. DPR-49 issued to Iowa Electric Light and Power Company, Central Iowa Power Cooperative, and Corn Belt Power Cooperative, which revises the Technical Specifications for operation of the Duane Arnold Energy Center (DAEC), located in Linn County, Iowa. The amendment is effective as of its date of issuance.

The amendment modifies the Technical Specification to incorporate a detailed definition of the terms OPERABLE and Limiting Conditions for Operation.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated June 17, 1980, (2) Amendment No. 77 to License No. DPR-49 and (3) the Commission's letter to Iowa Electric Light and Power Company dated June 21, 1982. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W. Washington, D.C. and at the Cedar Rapids Public Library, 426 Third Avenue, S.E., Cedar Rapids, Iowa 52401. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 21st day of June 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing