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Docket

December 2, 1975

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Docket No. 50-331

Iowa Electric Light & Power Company
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JSaltzman, OAI (w/o Tech Specs)
~~XXXXXXXXXXXXXXXXXXXX~~ CParrish
WPaulson
GLEar
KRGoller
SKari (w/o Tech Specs)

Gentlemen:

The Commission has issued the enclosed Amendment No. 12 to Facility Operating License No. DPR-49 for the Duane Arnold Energy Center. This amendment includes Change No. 13 to the Technical Specifications, Appendix A, and is in response to your request dated November 26, 1974.

The amendment incorporates into the Duane Arnold Energy Center Technical Specifications changes to the reporting requirements. Changes to your proposal were necessary to meet our requirements. These have been discussed with your staff. The Technical Specifications are based on Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4.

We request that you use the formats presented in the Appendices to Regulatory Guide 1.16, Revision 4, for reporting operating information and that you report events of the type described under the section "Events of Potential Public Interest". Instructions for using these reporting formats are contained in Regulatory Guide 1.16 (a copy is enclosed for your use), and AEC report OOE-SS-001 titled "Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File" of which you were previously provided a copy. This report is modified by updated instructions dated August 21, 1975 which are enclosed. Copy requirements are summarized in Regulatory Guide 10.1, "Compilation of Reporting Requirements for Persons Subject to NRC Regulations", a copy of which is also enclosed. This guide will assist you in identifying reports that are required by the Commission's regulations set forth in Title 10 Code of Federal Regulations but are not contained in your Technical Specifications. Reports that are required by the regulations have not been repeated in your Technical Specifications.

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

Q

see self

OFFICE >	RL:ORB#3	RL:ORB#3	RL:ORB#3		
SURNAME >	CParrish	WPaulson:acr	GLEAR George Lear, Chief		
DATE >	11/14/75	12/1/75	12/1	Operating Reactors Branch #3	Division of Reactor Licensing

Enclosures:

1. Amendment No. 12
2. Regulatory Guide 1.16
3. Updated Instructions
4. Regulatory Guide 10.1
5. Safety Evaluation
6. Federal Register Notice

cc w/encls:

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DATE >						

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

IOWA ELECTRIC LIGHT AND POWER COMPANY

CENTRAL IOWA POWER COOPERATIVE

CORN BELT POWER COOPERATIVE

DOCKET NO. 50-331

DUANE ARNOLD ENERGY CENTER

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 12
License No. DPR-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Iowa Electric Light and Power Company, Central Iowa Power Cooperative Corn Belt Power Cooperative (the licensees) dated November 26, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to to common defense and security or to the health and safety of the public.

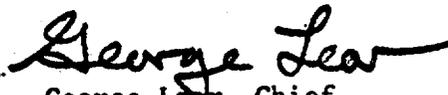
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility License No. DPR-49 is hereby amended to read as follows:

"(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 13"

3. This license amendment is effective 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

Attachment:
Change No. 13
Technical Specifications

Date of Issuance: December 2, 1975

ATTACHMENT TO LICENSE AMENDMENT NO. 12
CHANGE NO. 13 TO THE TECHNICAL SPECIFICATIONS
FACILITY OPERATING LICENSE NO. DPR-49

DOCKET NO. 50-331

Replaces pages iii, iv, 1.0-1 thru 1.0-4, 6.5-1, 6.5-2, 6.6-1, 6.10-1, 6.11-1 thru 6.11-15 with the attached revised pages (no changes have been made on pages iii, 1.0-1, 6.5-2).

<u>LIMITING CONDITION FOR OPERATION</u>	<u>SURVEILLANCE REQUIREMENTS</u>	
3.6 Primary System Boundary (Continued)		
F. Jet Pump Flow Mismatch	F	3.6-7
G. Structural Integrity	G	3.6-8
3.7 Containment Systems	4.7	3.7-1
A. Primary Containment	A	3.7-1
B. Standby Gas Treatment	B	3.7-14
C. Secondary Containment	C	3.7-17
D. Primary Containment Power Operated Isolation Valves	D	3.7-18
3.8 Auxiliary Electrical Systems	4.8	3.8-1
A. Auxiliary Electrical Equipment	A	3.8-1
B. Operation with Inoperable Components	B	3.8-3
C. Emergency Service Water System	C	3.8-6
3.9 Core Alterations	4.9	3.9-1
A. Refueling Interlocks	A	3.9-1
B. Core Monitoring	B	3.9-4
C. Spent Fuel Pool Water Level	C	3.9-4
3.10 Additional Safety Related Plant Capabilities	4.10	3.10-1
A. Main Control Room Ventilation	A	3.10-1
B. Emergency Shutdown Control Panel	B	3.10-2
3.11 River Level Specification	4.11	3.11-1

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5.0 Design Features	5.1-1
5.1 Site	5.1-1
5.2 Reactor	5.2-1
5.3 Reactor Vessel	5.3-1
5.4 Containment	5.4-1
5.5 Spent and New Fuel Storage	5.5-1
6.0 Administrative Controls	6.1-1
6.1 Management - Authority and Responsibility	6.1-1
6.2 Plant Staff Organization	6.2-1
6.3 Plant Staff Qualifications	6.3-1
6.4 Retraining and Replacement Training	6.4-1
6.5 Review and Audit	6.5-1
6.6 Reportable Occurrence Action	6.6-1
6.7 Action to be Taken if a Safety Limit is Exceeded	6.7-1
6.8 Plant Operating Procedures	6.8-1
6.9 Radiological Procedures	6.9-1
6.10 Records Retention	6.10-1
6.11 Plant Reporting Requirements	6.11-1

1.0 DEFINITIONS

The succeeding frequently used terms are explicitly defined so that a uniform interpretation of the specifications may be achieved.

1. Safety Limit

The safety limits are limits below which the reasonable maintenance of the cladding and primary systems are assured. Exceeding such a limit requires unit shutdown and review by the Atomic Energy Commission before resumption of unit operation. Operation beyond such a limit may not in itself result in serious consequences but it indicates an operational deficiency subject to regulatory review.

2. Limiting Safety System Setting (LSSS)

The limiting safety system settings are settings on instrumentation which initiate the automatic protective action at a level such that the safety limits will not be exceeded. These settings take into consideration the instrumentation tolerances and the instruments are required to be periodically calibrated as specified in these Technical Specifications. The limiting safety system setting plus the tolerance of the instrument as given in the

the system design control document gives the limiting trip point for operation. This additional margin has been established so that with proper operation of the instrumentation the safety limits will never be exceeded. The inequality sign which may be given merely signifies the preferred direction of operational trip setting.

3. Limiting Conditions for Operation (LCO)

The limiting conditions specify the minimum acceptable levels of system performance necessary to assure safe startup and operation of the facility. When these conditions are met, the plant can be operated safely and abnormal situations can be safely controlled.

4. DELETED

DELETED

5. Operable

A system or component shall be considered operable when it is capable of performing its intended function in its required manner.

6. Operating

Operating means that a system or component is performing its intended functions in its required manner.

7. Immediate

Immediate means that the required action will be initiated as soon as practicable considering the safe operation of the unit and the importance of the required action.

6.5 REVIEW AND AUDIT

6.5.1 Operations Committee

A committee of technically qualified plant staff members shall be appointed by the Chief Engineer to perform timely and continuing reviews of plant operations. The committee's activities shall be governed by a written charter which shall include:

1. Specification of committee membership and designation of its chairman. The qualifications of the regular members shall be maintained at a level equal to or greater than that specified in ANSI N18.1-1971.
2. The administrative procedures by which it functions, including specification of a quorum, meeting frequency, maintenance of records, and transmitting of its decisions.
3. Its authority.
4. Its review responsibilities.
 - a. Review of all events which are required by regulations or Technical Specifications to be reported to the NRC in writing within 24 hours.

6.6 REPORTABLE OCCURRENCE ACTION

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6.6.1 Any reportable occurrence shall be reported immediately to the Chief Engineer and to the General Production Manager, and promptly reviewed by the Operations Committee.

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6.6.2 The Operations Committee shall prepare a separate report for each reportable occurrence. This report shall include an evaluation of the cause of the occurrence, a record of the corrective action taken, and also recommendations for appropriate action to prevent or reduce the probability of a recurrence.

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6.6.3 Copies of all such reports shall be submitted to the Safety Committee for review and to the General Production Manager for review and approval of any recommendations.

6.10 RECORDS RETENTION

6.10.1 The following records shall be retained for at least 5 years:

1. Records and logs of facility operation covering time interval at each power level.
2. Records and logs of principal maintenance activities, inspections, and repair and replacement of principal items of equipment related to nuclear safety.
3. Reportable occurrence reports.
4. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
5. Records of reactor tests and experiments.
6. Records of changes made to Operating Procedures.
7. Records of radioactive shipments.
8. Records of sealed source leak test and results.
9. Records of annual physical inventory verifying accountability of sources on record.

6.11 REPORTING REQUIREMENTS

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the Director of the appropriate Regional Office of Inspection and Enforcement unless otherwise noted.

6.11.1 Routine Reports

- a. Startup Report. A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

Startup reports shall be submitted with (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

- b. Annual Operating Report.^{1/} Routine operating reports covering the operation of the unit during the previous calendar year should be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.

The annual operating reports made by licensees shall provide a comprehensive summary of the operating experience gained during the year, even though some repetition of previously reported information may be involved. References in the annual operating report to previously submitted reports shall be clear.

^{1/} A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

Each annual operating report shall include:

- (1) A narrative summary of operating experience during the report period relating to safe operation of the facility, including safety-related maintenance not covered in item 6.11.1.b(2)(e) below.
- (2) For each outage or forced reduction in power^{2/} of over twenty percent of design power level where the reduction extends for greater than four hours:
 - (a) The proximate cause and the system and major component involved (if the outage or forced reduction in power involved equipment malfunction);
 - (b) a brief discussion of (or reference to reports of) any reportable occurrences pertaining to the outage or power reduction;
 - (c) corrective action taken to reduce the probability of recurrence, if appropriate;

^{2/} The term "forced reduction in power" is normally defined in the electric power industry as the occurrence of a component failure or other condition which requires that the load on the unit be reduced for corrective action immediately or up to and including the very next weekend. Note that routine preventive maintenance, surveillance and calibration activities requiring power reductions are not covered by this section.

- (d) operating time lost as a result of the outage or power reduction (for scheduled or forced outages,^{3/} use the generator off-line hours; for forced reductions in power, use the approximate duration of operation at reduced power);
- (e) a description of major safety-related corrective maintenance performed during the outage or power reduction, including the system and component involved and identification of the critical path activity dictating the length of the outage or power reduction; and
- (f) a report of any single release of radioactivity or radiation exposure specifically associated with the outage which accounts for more than 10% of the allowable annual values.

^{3/} The term "forced outage" is normally defined in the electric power industry as the occurrence of a component failure or other condition which requires that the unit be removed from service for corrective action immediately or up to and including the very next weekend.

(3) A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions, ^{4/} e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

(4) Indications of failed fuel resulting from irradiated fuel examinations, including eddy current tests, ultrasonic tests, or visual examinations completed during the report period.

c. Monthly Operating Report. Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Office of Inspection and Enforcement U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the appropriate Regional Office, to

^{4/} This tabulation supplements the requirements of §20.407 of 10 CFR Part 20.

arrive no later than the tenth of each month following the calendar month covered by the report.

- d. Table 6.11-1 lists some of the routine reports required by 10 CFR Parts 20, 40, 50, and 70 including those listed in Specification 6.11.1.

6.11.2. REPORTABLE OCCURRENCES

Reportable occurrences, including corrective actions and measures to prevent reoccurrence, shall be reported to the NRC.

Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

- a. Prompt Notification With Written Followup. The types of events listed below shall be reported as expeditiously as possible, but within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the appropriate Regional Office, or his designate no later than the first working day following the event, with a written followup report within two weeks. The written followup report shall include, as a minimum, a completed copy of a licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.
 - (1) Failure of the reactor protection system or other systems subject to limiting safety system settings to

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initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function.

Note: Instrument drift discovered as a result of testing need not be reported under this item but may be reportable under items 6.11.2.a(5), 6.11.2.a(6) or 6.11.2.b(1) below.

- (2) Operation of the unit or affected systems when any parameter or operation subject to a limiting condition is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.

Note: If specified action is taken when a system is found to be operating between the most conservative and the least conservative aspects of a limiting condition for operation listed in the technical specifications, the limiting condition for operation is not considered to have been violated and need not be reported under this item, but it may be reportable under item 6.11.2.b(2) below.

- (3) Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

Note: Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

- (4) Reactivity anomalies, involving disagreement with the predicted value of reactivity balance under steady state conditions during power operation, greater than or equal to $1\% \Delta k/k$; a calculated reactivity balance indicating a shut-down margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if sub-critical, an unplanned reactivity insertion of more than $0.5\% \Delta k/k$ or occurrence of any unplanned criticality.
- (5) Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the SAR.
- (6) Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the SAR.

Note: For items 6.11.2.a(5) and 6.11.2.a(6) reduced redundancy that does not result in a loss of system function need not be reported under this section but may be reportable under items 6.11.2.b(2) and 6.11.2.b(3) below.

- (7) Conditions arising from natural or man-made events that, as a direct result of the event require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.
- (8) Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.
- (9) Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

Note: This item is intended to provide for reporting of potentially generic problems.

- b. Thirty Day Written Reports. The reportable occurrences discussed below shall be the subject of written reports to the Director of

the appropriate Regional Office within thirty days of occurrence of the event. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- (1) Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.
- (2) Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

Note: Routine surveillance testing, instrument calibration, or preventative maintenance which require system configurations as described in items 6.11.2.b(1) and 6.11.2.b(2) need not be reported except where test results themselves reveal a degraded mode as described above.

- (3) Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.

TABLE 6.11-1

REPORTING SUMMARY - ROUTINE REPORTS

<u>Requirement</u>	<u>Report</u>	<u>Timing of Submittal</u>
TS ¹	Startup	Within (1) 90 days of following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If all three events are not completed, supplementary reports every 3 months.
TS	First Year Operation	Within 60 days after completion of the first year of operation.
TS	Semiannual	Within 60 days after January 1 and July 1 of each year.
§20.407	Personnel Exposure and Monitoring	Within first quarter of each calendar year.
§20.408	Personnel Exposure on Termination of Employment or Work	Within 30 days after the exposure of the individual has been determined or 90 days after date of termination of employment or work assignment, whichever is earlier.
§40.64 (a)	Transfer of Source Material	Promptly upon transfer.
§40.64 (a)	Receipt of Source Material	Within 10 days after material is received.
§40.64 (b)	Source Material Inventory	Within 30 days after June 30 of each year.
§50.59 (b)	Changes, Tests, and Experiments	Annually or at shorter intervals as may be specified in the license.

TABLE 6.11-1 (Continued)
 REPORTING SUMMARY - ROUTINE REPORTS

<u>Requirement</u>	<u>Report</u>	<u>Timing of Submittal</u>
§70.53	Special Nuclear Material Status	Within 30 days after June 30 and December 31 of each year.
§70.54	Transfer of Special Nuclear Material	Promptly upon transfer.
§70.54	Receipt of Special Nuclear Material	Within 10 days after material is received.
Appendix G to 10 CFR Part 50	Fracture Toughness	On an individual-case basis at least 3 years prior to the date when the predicted fracture toughness levels will no longer satisfy the requirements of section V.B. of Appendix G to 10 CFR Part 50.
Appendix H to 10 CFR Part 50	Reactor Vessel Material Surveillance	Completion of tests after each capsule withdrawal.
Appendix J to 10 CFR Part 50	Reactor Containment Building Integrated Leak Rate Test	Approximately 3 months following conduct of test.

¹ Technical Specifications

TABLE 6.11-2

REPORTING SUMMARY - NONROUTINE REPORTS

<u>Requirement</u>	<u>Report</u>	<u>Notification</u>	Initial Written Report Within				
			<u>10 days</u>	<u>15 days</u>	<u>30 days</u>	<u>3 mo</u>	
TS ¹	Reportable Occurrence	Within 24 hours	X				13
TS	Unusual Events				X		
§20.405	Overexposures and Excessive Levels of Radiation and Concentration of Radioactive Material				X		
§20.402	Theft or Loss of Material	Immediately			X		DAEC-1
§20.403(a)	Severe Accident Involving Licensed Material	Immediately					
§20.403(b)	Accident Involving Licensed Material	Within 24 hours					
§40.64(c)	Theft or Unlawful Diversion of Source Material	Promptly			X		
§50.59(d)	Authorization of Changes, Tests, and Experiments	x ²					
§70.52	Accidental Criticality or Loss of Special Nuclear Material	Promptly					13
§73.42	Unaccounted for Shipments, Suspected Theft, or Unlawful Diversion of Special Nuclear Material	Immediately			X		
TS	Unique					x ³	13

6.11-14

NOTES TO TABLE 6.11-2

¹Technical Specifications.

²AEC authorization is required prior to performing a change, test, or experiment in this category.

³Unique reports covering inspections, tests, and maintenance that are appropriate to assure safe operation of the facility. The frequency and content of these special reports are determined on an individual case basis and designated in the Technical Specifications. Such reports include in-service inspection, tendon surveillance program study, fuel inspection, and containment structural tests.

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 12 TO FACILITY LICENSE NO. DPR-49

CHANGE NO. 13 TO TECHNICAL SPECIFICATIONS

IOWA ELECTRIC LIGHT AND POWER COMPANY

DUANE ARNOLD ENERGY CENTER

DOCKET NO. 50-331

Introduction

By letter dated November 26, 1974, Iowa Electric Light & Power Company proposed changes to the Technical Specifications appended to Facility Operating License No. DPR-49, for the Duane Arnold Energy Center. The proposed changes involve changes to the reporting requirements.

Discussion

The proposed changes would be administrative in nature and would affect the conduct of operation. The proposed changes are intended to provide uniform license requirements. Areas covered by the proposed uniform specifications include reporting requirements and an abnormal occurrence definition change.

In Section 208 of the Energy Reorganization Act of 1974 "abnormal occurrences" is defined as an unscheduled incident or event which the Commission determines is significant from the standpoint of public health or safety. The term "abnormal occurrence" is reserved for usage by NRC. Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4, enumerates required reports consistent with Section 208. The proposed change to required reports identifies the reports required of all licensees not already identified by the regulations and those unique to this facility. The proposal would formalize present reporting and would delete any reports no longer needed for assessment of safety related activities.

Evaluation

The new guidance for reporting operating information does not identify any event as an "abnormal occurrence". The proposed reporting requirements also delete reporting of information no longer required and duplication of reported information. The standardization of required reports and desired format for the information will permit more rapid recognition of potential problems.

During our review of the proposed changes, we found that certain modifications to the proposal were necessary to have conformance with the desired regulatory position. These changes were discussed with your staff and have been incorporated into the proposal.

We have concluded that the proposal as modified improves the licensee's program for evaluating plant performance and the reporting of the operating information needed by the Commission to assess safety related activities and is acceptable. The modified reporting program is consistent with the guidance provided by Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4. The administrative controls are consistent with requirements being incorporated in Technical Specifications for new licensed facilities.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: December 2, 1975

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-331

IOWA ELECTRIC LIGHT AND POWER COMPANY

CENTRAL IOWA POWER COOPERATIVE

CORN BELT POWER COOPERATIVE

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 12 to Facility Operating License No. DPR-49 issued to Iowa Electric Light and Power Company, Central Iowa Power Cooperative Corn Belt Power Cooperative which revised Technical Specifications for operation of the Duane Arnold Energy Center, located in Linn County, Iowa. The amendment is effective 30 days from the date of issuance.

The amendment modifies the reporting requirements of the Technical Specifications for the Duane Arnold Energy Center.

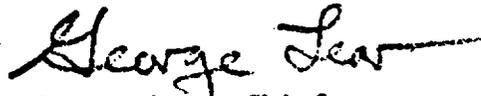
The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

For further details with respect to this action, see (1) the application for amendment dated November 26, 1974, (2) Amendment No. 12 to License No. DPR-49, with Change No. 13 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Cedar Rapids Public Library, 426 Third Avenue, S. E., Cedar Rapids, Iowa 52401.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 2 day of December, 1975

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-331

IOWA ELECTRIC LIGHT AND POWER COMPANY

CENTRAL IOWA POWER COOPERATIVE

CORN BELT POWER COOPERATIVE

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 12 to Facility Operating License No. DPR-49 issued to Iowa Electric Light and Power Company, Central Iowa Power Cooperative Corn Belt Power Cooperative which revised Technical Specifications for operation of the Duane Arnold Energy Center, located in Linn County, Iowa. The amendment is effective 30 days from the date of issuance.

The amendment modifies the reporting requirements of the Technical Specifications for the Duane Arnold Energy Center.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

OFFICE ➤	RL:ORB#3	RL:ORB#3	RL:ORB#3			
SURNAME ➤	CParrish <i>cp</i>	WPaulson:acr <i>WPA</i>	Glear <i>GL</i>			
DATE ➤	11/14/75	12/1/75	12/1/75			

For further details with respect to this action, see (1) the application for amendment dated November 26, 1974, (2) Amendment No. 12 to License No. DPR-49, with Change No. 13 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Cedar Rapids Public Library, 426 Third Avenue, S. E., Cedar Rapids, Iowa 52401.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 2 day of Dec. 1975

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

OFFICE						
SURNAME						
DATE						