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Docket No. 50-331

Iowa Electric Light and Power Company
ATTN: Mr. Duane Arnold, President
Security Building
P. O. Box 351
Cedar Rapids, Iowa 52406

Gentlemen:

The Commission has issued the enclosed Amendment No. 6 to Facility Operating License No. DPR-49 for the Duane Arnold Nuclear Center. The amendment incorporates Change No. 7 in the Technical Specifications in accordance with your request dated February 24, 1975.

The amendment authorizes a temporary change in the Technical Specifications to permit power operation of the reactor for an additional 48 hours without meeting the minimum volume of nitrogen required to be in the containment atmosphere dilution system.

A copy of the related Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

/S/

George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

Enclosures:
1. Amendment No. 6
2. Safety Evaluation
3. Federal Register Notice

cc: See next page

CP (1)

OFFICE	ORB#3	ORB#3	ORB#3	OELD	DRL:AD/ORs
SURNAME	SATeets/dg	Paulson/dg	GLear		KRGoller
DATE	2/27/75	2/27/75	2/28/75	3/3/75	3/4/75

Concurred 3/12/75

Form AEC-318 (Rev. 9-53) AECM 0240 ☆ U. S. GOVERNMENT PRINTING OFFICE: 1974-526-166

cc: w/enclosures

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Mr. Dudley Henderson
Chairman, Linn County
Board of Supervisors
Cedar Rapids, Iowa 52406

Mr. Ed Vest
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Reference Service
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Cedar Rapids, Iowa 52401

OFFICE ➤						
SURNAME ➤						
DATE ➤						

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

IOWA ELECTRIC LIGHT & POWER COMPANY

DOCKET NO. 50-331

DUANE ARNOLD NUCLEAR CENTER

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 6
License No. DPR-49

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Iowa Electric Light & Power Company (the licensee) dated February 24, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of the Facility License No. DPR-49 is hereby amended to read as follows:

   "(2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 7".

AMERICAN REVOLUTION BICENTENNIAL 1776-1976

3. This amendment is effective as of February 25, 1975.

FOR THE NUCLEAR REGULATORY COMMISSION

Karl R. Goller

Karl R. Goller, Assistant Director
for Operating Reactors
Division of Reactor Licensing

Attachment:
Change No. 7 to the
Technical Specifications

Date of Issuance: MAR 5 1975

ATTACHMENT TO LICENSE AMENDMENT NO. 6

CHANGE NO. 7 TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-49

DOCKET NO. 50-331

Replace pages 3.7-13 - 3.7-14 with the attached revised pages. (No change has been made on page 3.7-14).

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENTS
6. Containment Atmosphere Dilution	6. Containment Atmosphere Dilution
a. Whenever the reactor is in power operation, the Post-LOCA Containment Atmosphere Dilution System must be operable and capable of supplying nitrogen to the containment for atmosphere dilution if required by post-LOCA conditions. If this specification cannot be met, the system must be restored to an operable condition within 7 days or the reactor must be taken out of power operation.	a. The post-LOCA containment atmosphere dilution system shall be functionally tested once per operating cycle.
b. Whenever the reactor is in power operation, the post-LOCA Containment Atmosphere Dilution System shall contain a minimum of 50,000 scf of N_2 as determined by pressure and temperature measurements. If this specification cannot be met, the minimum volume will be restored within 7 days or the reactor must be taken out of power operation, except that for the period from February 25, 1975 to February 26, 1975, the minimum volume will be restored within 9 days or the reactor must be taken out of power operation.	b. The volume in the N_2 storage bank shall be recorded weekly.
c. Whenever the reactor is in power operation, there shall be at least one CAD system H_2 and O_2 analyzer serving the drywell and the suppression chamber. If this specification cannot be met, the reactor	c. The CAD system H_2 and O_2 analyzers shall be tested for operability using standard bottled H_2 and O_2 once per month and shall be calibrated once per 6 months. The atmosphere analyzing system shall be

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LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENTS
must be taken out of power operation.	functionally tested once per operating cycle in conjunction with specification 4.7.A.6.a. Should one of the two H_2 or O_2 analyzers serving the drywell or suppression pool be found inoperable the remaining analyzer of the same type serving the same compartment shall be tested for operability once per week until the defective analyzer is made operable.
7. If the specifications of 3.7.A.1 through 3.7.A.5 cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition within 24 hours.	
B. <u>Standby Gas Treatment System</u>	B. <u>Standby Gas Treatment System</u>
1. Except as specified in 3.7.B.2 below, both trains of the standby gas treatment system and the diesel generators required for operation of such trains shall be operable at all times when secondary containment integrity is required.	1.a At least once per operating cycle it shall be demonstrated that pressure drop across the combined high efficiency and charcoal filters is less than 11 inches of water at 4,000 cfm. b. At least once per operating cycle demonstrate that the inlet heaters on each train are capable of an output of at least 11 Kw.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR

REACTOR REGULATION

AMENDMENT NO. 6 TO FACILITY OPERATING

LICENSE NO. DPR-49

CHANGE NO. 7 TO THE TECHNICAL SPECIFICATIONS

IOWA ELECTRIC LIGHT & POWER COMPANY

DUANE ARNOLD ENERGY CENTER

DOCKET NO. 50-331-8

Introduction

By oral request on February 24, 1975, Iowa Electric Light and Power requested emergency authorization to change the Technical Specifications appended to Facility Operating License No. DPR-49 for the Duane Arnold Energy Center. The proposed change involves allowing power operation of the reactor for nine days without meeting the minimum volume of nitrogen (50,000 scf) required to be in the containment atmosphere dilution system rather than seven days for the period from February 25, 1975 to February 26, 1975.

Discussion

Section 3.7.A.6.b. of the Duane Arnold Technical Specifications requires that the post-LOCA containment atmosphere dilution system contain a minimum of 50,000 standard cubic feet (scf) of nitrogen whenever the reactor is in power operation. If this specification cannot be met, the Technical Specifications require that the minimum volume be restored within seven days or the reactor must be taken out of power operation. This seven-day period has expired. Iowa Electric Light & Power Company has requested authorization to continue power operation of the reactor for an additional 48 hours beyond the seven days specified in the Technical Specifications.

Iowa Electric has stated that they have 44,000 scf in the containment atmosphere dilution system. Iowa Electric has also verbally stated that the drywell is inerted in accordance with the Technical Specifications.

AMERICAN REVOLUTION BICENTENNIAL 1776-1976

Evaluation

The basis for the Technical Specification requiring 50,000 scf of nitrogen in the post-LOCA containment dilution system is to ensure that there is a seven-day supply of nitrogen available for use in diluting the hydrogen-oxygen mixture that may occur in the unlikely event of a postulated loss-of-coolant accident. The nitrogen presently on-site in the containment atmosphere dilution system (44,000 scf) is adequate to provide approximately 6.2 days requirement of nitrogen in the unlikely event of a loss-of-coolant accident. Iowa Electric has further stated verbally that a truck with sufficient nitrogen to meet the Technical Specification limit of 50,000 scf was scheduled to be at the site on the morning of February 25, 1975; however, because of a snow storm that has paralyzed traffic, there may be a delay in receiving the nitrogen.

Based on our evaluation of the amount of nitrogen that is available in the containment dilution system and considering that sufficient nitrogen is scheduled to be delivered within the requested 48 hours extension to meet the Technical Specification requirement, and further considering the low probability of occurrence of a postulated loss-of-coolant accident during this 48-hour period, we conclude that (1) Iowa Electric's request for a 48-hour extension to meet the minimum volume of nitrogen (50,000 scf) in the containment atmosphere dilution system is acceptable; and (2) the proposed change will not introduce a significant safety consideration.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: MAR 1 1975

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-331

IOWA ELECTRIC LIGHT & POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 6 to Facility Operating License No. DPR-49 issued to Iowa Electric Light & Power Company (the licensee) which revised Technical Specifications for operation of the Duane Arnold Nuclear Center, located in Linn County, Iowa. The amendment is effective as of February 25, 1975.

The amendment authorizes a temporatry change in the Technical Specifications to permit power operation of the reactor for an additional 48 hours without meeting the minimum volume of nitrogen required to be in the containment atmosphere dilution system.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

For furhter details with respect to this action, see (1) the application for amendment dated February 24, 1975, (2) Amendment No. 6 to License No. DPR-49, with Change No. 7, and (3) the Commission's related

Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Reference Service, Cedar Rapids Public Library, 426 Third Avenue, S. E., Cedar Rapids, Iowa 52401.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this MAY 5 1975

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear

George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing