

DEC 23 1977

Dockets Nos. 50-266
and 50-301

Wisconsin Electric Power Company
Wisconsin Michigan Power Company
ATTN: Mr. Sol Burstein
Executive Vice President
231 West Michigan Street
Milwaukee, Wisconsin 53201

Gentlemen:

The Commission has issued the enclosed Amendments Nos. 31 and 35 to Facility Operating Licenses Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Units Nos. 1 and 2. The amendments consist of changes to the Technical Specifications and are in accordance with your applications dated September 19, October 11 (as amended by letter dated October 31) and October 27, 1977.

These amendments (1) revise the pressurizer heatup rate limits, (2) revise the leak test requirement for certain sealed radioactive sources, and (3) modify the requirements for annual and monthly reports.

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

Original signed by

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:

1. Amendment No. 31 to License DPR-24
2. Amendment No. 35 to License DPR-27
3. Safety Evaluation
4. Federal Register Notice

cc w/enclosure:
See next page

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Wisconsin Electric Power Company
Wisconsin Michigan Power Company

- 2 -

cc:

Mr. Bruce Churchill, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N. W.
Washington, D. C. 20036

Mr. Norman Clapp, Chairman
Public Service Commission
of Wisconsin
Hill Farms State Office Building
Madison, Wisconsin 53702

Mr. Arthur M. Fish
Document Department
University of Wisconsin -
Stevens Point Library
Stevens Point, Wisconsin 54481

Wisconsin Electric Power Company
ATTN: Mr. Glen Reed
Manager, Nuclear Power Division
Point Beach Nuclear Plant
231 West Michigan Street
Milwaukee, Wisconsin 53201

Chief, Energy Systems Analysis Branch (AW-459)
Office of Radiation Programs
U. S. Environmental Protection Agency
Room 645, East Tower
401 M Street, S. W.
Washington, D. C. 20460

U. S. Environmental Protection Agency
Federal Activities Branch
Region V Office
ATTN: EIS COORDINATOR
230 S. Dearborn Street
Chicago, Illinois 60604

Walter L. Meyer
Town Chairman
Town of Two Creeks, Wisconsin
Route 3, Two Rivers, Wisconsin 54241



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY
WISCONSIN MICHIGAN POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 31
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Wisconsin Electric Power Company and Wisconsin Michigan Power Company (the licensees) dated September 19, October 11 (as amended by letter dated October 31) and October 27, 1977, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-24 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 31, are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for Howard J. Decker
George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 23, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 31

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-24

DOCKET NO. 50-266

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised page is identified by Amendment number and contains vertical lines indicating the area of change.

Remove

15.3.1-4
15.4.12-1
15.4.12-2
15.6.9-2
15.6.9-3

Replace

15.3.1-4
15.4.12-1
15.4.12-2
15.6.9-2
15.6.9-3

B. Pressure/Temperature Limits

Specification:

1. The Reactor Coolant System temperature and pressure shall be limited in accordance with the limit lines shown in Figure 15.3.1-1 and 15.3.1-2 (Unit 1) and 15.3.1-3 and 15.3.1-4 (Unit 2) during heatup, cooldown, criticality, and inservice leak and hydrostatic testing with:
 - a. A maximum heatup of 100°F in any one hour,
 - b. A maximum cooldown of 100°F in any one hour, and
 - c. An average temperature change of $\leq 10^\circ\text{F}$ per hour during inservice leak and hydrostatic testing operations.
2. The secondary side of the steam generator will not be pressurized above 200 psig if the temperature of the steam generator vessel shell is below 70°F.
3. The pressurizer temperature shall be limited to:
 - a. A maximum heatup of 100°F in any one hour and a maximum cooldown of 200°F in any one hour, and
 - b. A maximum spray water temperature differential between the pressurizer and spray fluid of not greater than 320°F.
4. The reactor vessel material irradiation surveillance specimens shall be removed and examined in accordance with the schedules presented in Table 15.3.1-1 (Unit 1) and 15.3.1-2 (Unit 2) to determine changes in material properties. The results of these examinations shall be considered in the evaluation of the prediction method to be used to update Figures 15.3.1-1, 15.3.1-2, 15.3.1-3, and 15.3.1-4. Revised figures shall be provided to the Commission at least sixty (60) days before the calculated exposure of the applicable reactor vessel exceeds the exposure for which the figures apply.

15.4.12 MISCELLANEOUS RADIOACTIVE MATERIALS SOURCES

Applicability

Applies to periodic testing requirements of miscellaneous radioactive materials sealed sources.

Objective

To verify that potential leakage of sealed radioactive sources is maintained within acceptable limits.

Specification

A. Source Leakage Test

1. Sealed radioactive sources as described below shall be leak tested for contamination. The leakage test shall be capable of detecting the presence of 0.005 microcurie of radioactive material on the test sample. If the test reveals the presence of 0.005 microcurie or more of removable contamination, it shall immediately be withdrawn from use, decontaminated, and repaired, or be disposed of in accordance with Commission regulations.
2. Sealed sources containing quantities of radioactive material either in excess of 100 microcuries of beta and/or gamma activity or in excess of 5 microcuries of alpha activity shall be leak tested in accordance with the Surveillance Requirements below.

B. Surveillance Requirements

Tests for leakage and/or contamination shall be performed by the licensee or by other persons specifically authorized by the Commission or an agreement State, as follows:

1. Each sealed source, except startup sources and sources in fission detectors subject to core flux, containing radioactive material, other than Hydrogen-3, with a half-life greater than thirty days and in any form other than gas shall be tested for leakage and/or contamination at intervals not to exceed six months.
2. The periodic leak test required does not apply to sealed sources that are stored and not being used. Such stored sources shall be tested for leakage prior to any use or transfer to another user unless they have been leak tested within six months prior to the date of use or transfer.
3. Startup sources shall be leak tested prior to and following any repair or maintenance and before being subjected to core flux.

Basis

Ingestion or inhalation of source material may give rise to total body or organ irradiation. This specification assures that leakage from radioactive material sources does not exceed allowable limits. In the unlikely event that those quantities of radioactive by-product materials of interest to this specification which are exempt from leakage testing are ingested or inhaled, they represent less than one maximum permissible body burden for total body irradiation. The limits for all other sources (including alpha emitters) are based upon 10 CFR 70.39(c) limits for plutonium.

- a. 90 days following completion of the startup tests.
- b. 90 days following resumption or commencement of commercial power operation.
- c. 9 months following initial criticality.

B. Annual Results and Data Report

1. A results and data report covering the period of the previous calendar year shall be submitted prior to March 1 of each year.
2. This report shall include:
 - a. Complete results of steam generator tube inservice inspection completed during the calendar year as required by specification 15.4.2.c.2.
 - b. A tabulation on an annual basis of the number of station, utility, and other personnel receiving exposures greater than 100 mrem/year and their associated man-rem exposure according to work and job functions. The dose assignments to various duty functions may be estimates based on pocket dosimeter, TLD or film badge measurements. Small exposures totalling less than 20 percent of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.
 - c. A description of facility changes, tests or experiments as required pursuant to 10 CFR 50.59(b).

C. Monthly Operating Reports

1. Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis under the titles "Operating Data Report", "Average Daily Power Levels" and "Unit Shutdowns" and "Power Reduction". In addition, the report shall contain a narrative summary of operating experience that describes the operation of the facility, including major safety-related maintenance for the monthly report period.
2. Completed reports shall be sent to the Director, Office of Management Information and Program Control, U. S. NRC, Washington, D. C., 20555, by the tenth of each month following the calendar month covered by the report.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY
WISCONSIN MICHIGAN POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 35
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Wisconsin Electric Power Company and Wisconsin Michigan Power Company (the licensees) dated September 19, October 11 (as amended by letter dated October 31), and October 27, 1977, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-27 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 35, are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for Howard J. DeWitt
George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 23, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 35

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NO. 50-301

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised page is identified by Amendment number and contains vertical lines indicating the area of change.

Remove

15.3.1-4
15.4.12-1
15.4.12-2
15.6.9-2
15.6.9-3

Replace

15.3.1-4
15.4.12-1
15.4.12-2
15.6.9-2
15.6.9-3

B. Pressure/Temperature Limits

Specification:

1. The Reactor Coolant System temperature and pressure shall be limited in accordance with the limit lines shown in Figure 15.3.1-1 and 15.3.1-2 (Unit 1) and 15.3.1-3 and 15.3.1-4 (Unit 2) during heatup, cooldown, criticality, and inservice leak and hydrostatic testing with:
 - a. A maximum heatup of 100°F in any one hour,
 - b. A maximum cooldown of 100°F in any one hour, and
 - c. An average temperature change of $\leq 10^\circ\text{F}$ per hour during inservice leak and hydrostatic testing operations.
2. The secondary side of the steam generator will not be pressurized above 200 psig if the temperature of the steam generator vessel shell is below 70°F.
3. The pressurizer temperature shall be limited to:
 - a. A maximum heatup of 100°F in any one hour and a maximum cooldown of 200°F in any one hour, and
 - b. A maximum spray water temperature differential between the pressurizer and spray fluid of not greater than 320°F.
4. The reactor vessel material irradiation surveillance specimens shall be removed and examined in accordance with the schedules presented in Table 15.3.1-1 (Unit 1) and 15.3.1-2 (Unit 2) to determine changes in material properties. The results of these examinations shall be considered in the evaluation of the prediction method to be used to update Figures 15.3.1-1, 15.3.1-2, 15.3.1-3, and 15.3.1-4. Revised figures shall be provided to the Commission at least sixty (60) days before the calculated exposure of the applicable reactor vessel exceeds the exposure for which the figures apply.

15.4.12 MISCELLANEOUS RADIOACTIVE MATERIALS SOURCES

Applicability

Applies to periodic testing requirements of miscellaneous radioactive materials sealed sources.

Objective

To verify that potential leakage of sealed radioactive sources is maintained within acceptable limits.

Specification

A. Source Leakage Test

1. Sealed radioactive sources as described below shall be leak tested for contamination. The leakage test shall be capable of detecting the presence of 0.005 microcurie of radioactive material on the test sample. If the test reveals the presence of 0.005 microcurie or more of removable contamination, it shall immediately be withdrawn from use, decontaminated, and repaired, or be disposed of in accordance with Commission regulations.
2. Sealed sources containing quantities of radioactive material either in excess of 100 microcuries of beta and/or gamma activity or in excess of 5 microcuries of alpha activity shall be leak tested in accordance with the Surveillance Requirements below.

B. Surveillance Requirements

Tests for leakage and/or contamination shall be performed by the licensee or by other persons specifically authorized by the Commission or an agreement State, as follows:

1. Each sealed source, except startup sources and sources in fission detectors subject to core flux, containing radioactive material, other than Hydrogen-3, with a half-life greater than thirty days and in any form other than gas shall be tested for leakage and/or contamination at intervals not to exceed six months.
2. The periodic leak test required does not apply to sealed sources that are stored and not being used. Such stored sources shall be tested for leakage prior to any use or transfer to another user unless they have been leak tested within six months prior to the date of use or transfer.
3. Startup sources shall be leak tested prior to and following any repair or maintenance and before being subjected to core flux.

Basis

Ingestion or inhalation of source material may give rise to total body or organ irradiation. This specification assures that leakage from radioactive material sources does not exceed allowable limits. In the unlikely event that those quantities of radioactive by-product materials of interest to this specification which are exempt from leakage testing are ingested or inhaled, they represent less than one maximum permissible body burden for total body irradiation. The limits for all other sources (including alpha emitters) are based upon 10 CFR 70.39(c) limits for plutonium.

- a. 90 days following completion of the startup tests.
- b. 90 days following resumption or commencement of commercial power operation.
- c. 9 months following initial criticality.

B. Annual Results and Data Report

1. A results and data report covering the period of the previous calendar year shall be submitted prior to March 1 of each year.
2. This report shall include:
 - a. Complete results of steam generator tube inservice inspection completed during the calendar year as required by specification 15.4.2.c.2.
 - b. A tabulation on an annual basis of the number of station, utility, and other personnel receiving exposures greater than 100 mrem/year and their associated man-rem exposure according to work and job functions. The dose assignments to various duty functions may be estimates based on pocket dosimeter, TLD or film badge measurements. Small exposures totalling less than 20 percent of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.
 - c. A description of facility changes, tests or experiments as required pursuant to 10 CFR 50.59(b).

C. Monthly Operating Reports

1. Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis under the titles "Operating Data Report", "Average Daily Power Levels" and "Unit Shutdowns" and "Power Reduction". In addition, the report shall contain a narrative summary of operating experience that describes the operation of the facility, including major safety-related maintenance for the monthly report period.
2. Completed reports shall be sent to the Director, Office of Management Information and Program Control, U. S. NRC, Washington, D. C., 20555, by the tenth of each month following the calendar month covered by the report.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENTS NOS. 31 AND 35 TO FACILITY LICENSES DPR-24 AND DPR-27

WISCONSIN ELECTRIC POWER COMPANY

WISCONSIN MICHIGAN POWER COMPANY

POINT BEACH UNITS NOS. 1 AND 2

DOCKETS NOS. 50-266 AND 50-301

Introduction

By letters dated September 19, 1977, October 11, 1977 (as amended by letter dated October 31, 1977), and October 27, 1977, Wisconsin Electric Power Company (WEPCO) requested changes to the Technical Specifications appended to Facility Operating Licenses Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant Units Nos. 1 and 2. These proposed changes would: (1) revise the pressurizer heatup rate limits, (2) revise the leak test requirements for certain sealed radioactive sources, and (3) modify the requirements for annual and monthly reports.

Evaluation

1. Pressurizer Temperature Rate of Change

The proposal submitted on September 19, 1977 requests revision of Section 15.3.1.B.3.a. which contains the pressurizer temperature limits. The change involves decreasing the maximum heatup rate for the pressurizer from 200°F to 100°F in any hour. The maximum cooldown rate will remain at 200°F in any hour. This change is necessitated because the engineering specifications for the Point Beach Nuclear Plant pressurizers specify a maximum heatup rate of 100°F/hour. This inconsistency was brought to the licensees' attention by a letter from Westinghouse Electric Corporation dated August 1, 1977.

The maximum achievable heatup rate of the Point Beach Nuclear Plant pressurizers, utilizing all heaters, is shown in Table 4.1-3 of the Final Facility Description and Safety Analysis Report to be only 55°F/hour.

Since the decrease in the maximum heatup rate provides more conservatism than the current limits and is in accordance with the staff's position as stated in a letter to the licensees dated October 13, 1977, we find this proposal to be acceptable.

2. Leak Testing of Sources

The application submitted on October 11, 1977 requested a change to the size of sealed radioactive sources required to be leak tested. The present Technical Specification 15.4.12 requires leak testing of all sealed sources containing byproduct material that exceeds the quantities listed in 10 CFR §30.71, Schedule B. As a result of this wording, even the small check sources built into various radiation monitors throughout the Point Beach Plant have required leak tests. In order to obviate leak tests on such small sealed sources, WEPCO proposed to increase the quantities above which leak testing would be required to one hundred times those given in the above mentioned schedule.

The staff recognized the impracticality of performing semiannual leak tests on small check sources and has previously incorporated language in Standard Technical Specifications (STS) which provided sufficient flexibility in the testing requirements. The licensees were informed of the STS wording and agreed that such a specification would provide adequate operational flexibility. A revision to the submittal was made on October 31, 1977 which incorporated the STS wording into Specification 15.4.12 for the Point Beach Nuclear Plant Units Nos. 1 and 2.

Since this revised submittal is consistent with the requirements of other plants and with the STS we find this proposal to be acceptable.

3. Reporting Requirements

An application was submitted by the licensees on October 27, 1977 to modify (a) the requirements for submitting an annual operating report and (b) the content of the monthly reports in accordance with a request made by the NRC on September 22, 1977. The NRC requested that the licensees (a) propose a change to the Technical Specifications that would delete the requirement for an annual operation report, (b) modify the content of the monthly report, (c) provide any missing data in the summary compilation which will be prepared and transmitted to the licensee by NRC, and (d) provide a narrative of operating experience for the year 1977.

This request was made after a review of the content of the annual reports revealed that much of the information is also contained in other reports. The section on Occupational Radiation Exposure which had been contained in the annual report is now to be included in the monthly reports or another appropriate report. The licensees have included in the proposed Technical Specification change a requirement for an Annual Results and Data Report which will transmit the required occupational radiation exposure information to the NRC. The licensees further agree to review and correct the summary compilation for 1977 when provided by the NRC and to provide a narrative summary of operating experience for the year 1977 not later than February 28, 1978.

Since this proposal was submitted in accordance with and is responsive to a request by the NRC, and since the change involves only administrative matters which will have no adverse affect on the safe operation of the Point Beach Nuclear Plant Units Nos. 1 and 2, we find that the proposed change is acceptable.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 23, 1977

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-266 AND 50-301

WISCONSIN ELECTRIC POWER COMPANY
WISCONSIN MICHIGAN POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 31 and 35 to Facility Operating Licenses Nos. DPR-24 and DPR-27 issued to Wisconsin Electric Power Company and Wisconsin Michigan Power Company, which revised Technical Specifications for operation of the Point Beach Nuclear Plant Units Nos. 1 and 2, located in the town of Two Creeks, Manitowoc County, Wisconsin. The amendments are effective as of the date of issuance.

These amendments consist of changes in the Technical Specifications that: (1) revise the pressurizer heatup rate limits, (2) revise the leak test requirements for certain sealed radioactive sources, and (3) modify the requirements for annual and monthly reports.

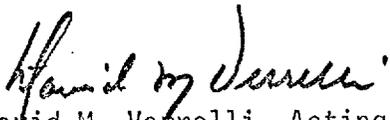
The applications for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the applications for amendments dated September 19, October 11 (as amended by letter dated October 31) and October 27, 1977, (2) Amendment No. 31 to License No. DPR-24, (3) Amendment No. 35 to License No. DPR-27, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the University of Wisconsin - Stevens Point Library, ATTN: Mr. Arthur M. Fish, Stevens Point, Wisconsin 54481. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland this 23 day of December 1977.

FOR THE NUCLEAR REGULATORY COMMISSION


David M. Verrelli, Acting Chief
Operating Reactors Branch #3
Division of Operating Reactors