JAN 2 3 1978

Dockets Nos. 50-266
and 50-301

Wisconsin Electric Power Company
Wisconsin Michigan Power Company
ATTN: Mr. Sol Burstein
Executive Vice President
231 West Michigan Street
Milwaukee, Wisconsin 53201

Gentlemen:

The Commission has issued the enclosed Amendments Nos. 32 and 36 to Facility Operating Licenses Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Units Nos. 1 and 2. The amendments consist of changes to the Technical Specifications and are in accordance with your application dated July 28, 1977 as revised by letters dated October 5 and December 12, 1977.

These amendments consist of changes in the Technical Specifications that incorporate the Fire Protection System into the Limiting Conditions for Operation, Surveillance Requirements and Administrative Controls.

In order to achieve expeditious implementation of the Fire Protection Technical Specifications, Specification 15.6.2.2.f. is being issued at this time with the minimum number of on-site fire brigade members specified as 4 as you proposed. This number is less than the minimum number given in the generic staff position, Minimum Fire Brigade Shift Size, which was an attachment to the Safety Evaluation Report issued with our letter to you dated November 23, 1977. However, we are presently evaluating your justification for this smaller brigade size and when the evaluation is completed the minimum number will be increased if we do not agree with your position.

These amendments revise: (1) Specification 15.6.4.2 to reflect the title change of the Fire Protection Supervisor (the title had been Fire Protection Program Manager), (2) Figure 15.6.2-4 to indicate the Fire Protection Supervisor is no longer the Assistant to the Operations Superintendent, and (3) Specification 15.6.8.1.8 to clarify when the fire protection procedures must be provided (as agreed to by the licensee). Since these revisions are administrative matters, the staff finds them acceptable.

Const. 1
GD

OFFICE ➤						
SURNAME ➤						
DATE ➤						

All other Specifications are the same as those transmitted to you on November 23, 1977, and are supported by the Safety Evaluation which was attached to that letter.

Copies of the related FEDERAL REGISTER Notice also are enclosed.

Sincerely,

Original signed by

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:
1. Amendment No. 32 to License DPR-24
2. Amendment No. 30 to License DPR-27
3. FEDERAL REGISTER Notice

cc w/enclosures:
See next page

*SEE PREVIOUS YELLOW FOR CONCURRENCES

OFFICE➤	ORB #3	ORB #3	ORB #1	OELD	ORB #3	
SURNAME➤	CParrish*	*PWagner:acr	*TWambach		GLear	
DATE➤	1/4/78	1/5/78	1/10/78	1/ /78	1/ /78	

All other Specifications are the same as those transmitted to you on November 23, 1977, and are supported by the Safety Evaluation contained therein.

Copies of the related Federal Register Notice also are enclosed.

Sincerely,

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:
1. Amendment No. to License DPR-24
2. Amendment No. to License DPR-27
3. Federal Register Notice

cc w/enclosures:
See next page

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cc:

Mr. Bruce Churchill, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N. W.
Washington, D. C. 20036

Chairman
Public Service Commission
of Wisconsin
Hill Farms State Office Building
Madison, Wisconsin 53702

Mr. Arthur M. Fish
Document Department
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Wisconsin Electric Power Company
ATTN: Mr. Glen Reed
Manager, Nuclear Power Division
Point Beach Nuclear Plant
231 West Michigan Street
Milwaukee, Wisconsin 53201

Chief, Energy Systems Analysis Branch (AW-459)
Office of Radiation Programs
U. S. Environmental Protection Agency
Room 645, East Tower
401 M Street, S. W.
Washington, D. C. 20460

U. S. Environmental Protection Agency
Federal Activities Branch
Revion V Office
ATTN: EIS COORDINATOR
230 S. Dearborn Street
Chicago, Illinois 60604

Walter L. Meyer
Town Chairman
Town of Two Creeks, Wisconsin
Route 3, Two Rivers, Wisconsin 54241

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY
WISCONSIN MICHIGAN POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 32
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Electric Power Company and Wisconsin Michigan Power Company (the licensees) dated July 28, 1977 as revised by letters dated October 5 and December 12, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-24 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 32, are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 23, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 32

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-24

DOCKET NO. 50-266

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised page is identified by Amendment number and contains vertical lines indicating the area of change.

Remove	Replace
15i	15i
-	15ii
15.1-5	15.1-5
-	15.3.14-1
-	15.3.14-2
-	15.3.14-3
-	15.3.14-4
Table 15.4.1-2 (Cont)	Table 15.4.1-2 (Cont)
-	15.4.15-1
-	15.4.15-2
-	15.4.15-3
-	15.6.2-2
Figure 15.6.2-2	Figure 15.6.2-2
-	Figure 15.6.2-3
-	Figure 15.6.2-4
15.6.3/4/5-1	15.6.3/4/5-1
15.6.5-3	15.6.5-3
-	15.6.5-9
15.6.8-1	15.6.8-1

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TABLE OF CONTENTS (CONT'D)

l. Degree of Redundancy

Degree of redundancy is defined as the difference between the number of operable channels and the minimum number of channels which when tripped will cause an automatic shutdown.

m. Reactor Critical

The reactor is said to be critical when the neutron chain reaction is self-sustaining and $k_{eff} = 1.0$.

n. Low Power Operation

The reactor is in the low power operating condition when the reactor is critical and the average neutron flux of the power range instrumentation indicates less than or equal to 2% of rated power.

o. Fire Suppression Water System

A FIRE SUPPRESSION WATER SYSTEM shall consist of: a water source; pump(s); and distribution piping with associated sectionalizing control or isolation valves. Such valves shall include yard post indicating valves and the first valve ahead of the water flow alarm device on each sprinkler, hose standpipe or spray system riser.

15.3.14 Fire Protection System

Applicability

Applies to the operability of the fire protection system components which provide fire suppression capability for equipment required for safe plant shutdown at all times when those systems are required to be operable.

Objective

To specify the functional requirements for fire protection system components which would be employed to mitigate the consequences of fires which could affect equipment required for safe plant shutdown.

Specification

A. Fire Suppression Water System

One of the following conditions shall be provided:

1. Both fire pumps shall be operable at rated capacity.
2. With one fire pump inoperable, the other fire pump shall be demonstrated operable once per day; or,
3. With both fire pumps inoperable,
   a. Establish a backup FIRE SUPPRESSION WATER SYSTEM within 24 hours.
   b. Furnish prompt notification with written followup to the Commission pursuant to Specification 15.6.9.2 outlining the actions taken and the plans and schedule for restoring the system to OPERABLE status.
   c. If a. above cannot be fulfilled, place the reactor in Hot Standby within the next 6 hours and in Cold Shutdown within the following thirty (30) hours.

B. Spray and/or Sprinkler Systems

The following systems shall be operable.

Diesel Generator 3D Dry Pipe Sprinkler System

Diesel Generator 4D Dry Pipe Sprinkler System

1. With an above listed system inoperable, local hose station fire suppression equipment for the affected area shall be verified operable within 1 hour.
2. Additional portable fire suppression equipment shall be provided for the affected area.
3. A fire watch inspection shall be performed in the affected area twice per shift. Activity within the affected area shall be restricted to that which is necessary for continued operation.

C. Fire Hose Stations

The following hose stations shall be operable:

HR-13 South Wall Control Building Elev. 8'-0"

HR-15 South Wall Control Building Elev. 44'-0"

HR-16 North Wall Control Building Elev. 8'-0"

HR-18 North Wall Control Building Elev. 44'-0"

HR-31 West Wall Auxiliary Building Elev. 8'-0"

1. With a hose station inoperable, backup portable fire suppression equipment for the affected area shall be verified operable within 1 hour.
2. Appropriate backup portable fire suppression equipment shall be provided for the affected area.

D. Fire Detection

The fire detection instrumentation for each fire detection zone shown in Table 15.3.14-1 shall be operable.

1. With a fire detection instrument inoperable, the affected area shall be inspected to assure that potential fire hazards are minimized.

2. A fire watch inspection shall be performed in the affected area once per hour. Activity in the affected area shall be restricted to that which is necessary for continued operation.

E. Fire Barrier Penetration Fire Seals

All penetration fire barriers protecting safety related areas shall be functional.

1. In the event of a penetration fire barrier impairment a continuous fire watch shall be established on at least one side of the affected penetration within 1 hour.

Basis

The overall fire protection program at Point Beach Nuclear Plant utilizes the principles of defense in depth. This includes minimization of combustibles, early warning fire detection, primary and backup fire suppression capability and priority maintenance procedures to restore inoperable equipment to operable status as soon as possible. Collectively these measures provide adequate capability to minimize potential damage to safety related equipment and to allow for safe plant shutdown in the event of a potential fire occurrence.

Should a portion or component of the fire protection system be inoperable, these specifications provide assurance that alternate methods of fire protection are strengthened and that the capability to mitigate the consequences of a potential fire is maintained.

TABLE 15.3.14-1

	Area	Quantity	Elevation	Type
1.	Cable Spreading Room	2	26'-0"	Smoke
2.	Switchgear Room	1	8'-0"	Smoke
3.	Diesel Generator 3D Room	1	8'-0"	Smoke
4.	Diesel Generator 4D Room	1	8'-0"	Smoke
5.	Fuel Oil Pumphouse	1	25'-7"	Smoke
6.	Unit 1 Electrical Equip. Room	1	46'-0"	Smoke
7.	Unit 2 Electrical Equip. Room	1	46'-0"	Smoke
8.	Circulating Water Pumphouse	6	7'-0"	Smoke

TABLE 15.4.1-2 (CONTINUED)

		Test	Frequency	FSAR Section Reference
(14)	Refueling System Interlocks	Functioning	Each refueling shutdown	9.4.5
(15)	Service Water System	Functioning	Each refueling shutdown	9.5.5
(16)	Primary System Leakage	Evaluate	Monthly(6)	4
(17)	Diesel Fuel Supply	Fuel inventory	Daily	8.2.3
(18)	Turbine Stop and Governor Valves	Functioning	Monthly(6) (9)	10
(19)	Low Pressure Turbine Rotor Inspection(5)	Visual and magnetic particle or liquid penetrant	Every five years	10
(20)	Boric Acid System	Storage Tank Temperature	Daily	
(21)	Boric Acid System	Visual observation of piping temperatures (all ≥145°F)	Daily	
(22)	Boric Acid Piping Heat Tracing	Electrical circuit operability	Monthly	

(1) A radiochemical analysis for this purpose shall consist of a quantitative measurement of each radionuclide with half life of >30 minutes such that at least 95% of total activity of primary coolant is accounted for.
(2) $\bar{E}$ determination will be started when the gross activity analysis of a filtered sample indicates ≥10 μc/cc and will be redetermined if the primary coolant gross radioactivity of a filtered sample increases by more than 10 μc/cc.
(3) Drop tests shall be conducted at rated reactor coolant flow. Rods shall be dropped under both cold and hot conditions, but cold drop tests need not be timed.
(4) Drop tests will be conducted in the hot condition for rods on which maintenance was performed.
(5) As accessible without disassembly of rotor.
(6) Not required during periods of refueling shutdown.
(7) At least once per week during periods of refueling shutdown.
(8) At least three times per week (with maximum time of 72 hours between samples) during periods of refueling shutdown.
(9) a. The monthly functional test interval for Point Beach Unit No. 2 for February 1976 is extended seven days to February 26, 1976.
b. The requirement of the monthly functional test for Point Beach Unit No. 1 is waived for the one month period of September, 1977.

15.4.15 Fire Protection System

Applicability

Applies to the periodic inspection and testing requirements of fire protection equipment.

Objective

To verify the operability of fire protection equipment.

Specification

Testing of fire protection system equipment, as a minimum, shall be done as follows:

A. Fire Suppression Water System

	Test	Frequency
1.	Verify valves in the Flow Path (automatic and manual) in the correct position.	Monthly
2.	Fire Pump Functional Test	Monthly
3.	Fire Pump Capacity Test	Yearly
4.	Automatic system and valve actuation and flowpath valve cycle tests	Yearly
5.	System flow Test (In accordance with Ch. 5, Sect. 11 F.P. Handbook)	3 Years

B. Spray and/or Sprinkler Systems

	Test	Frequency
1.	Complete Cycle of Each Testable Valve	Yearly
2.	Simulated System Functional Test	Yearly
3.	Visually inspect headers and nozzles	18 mo.
4.	Air flow test to verify open head nozzles unobstructed	3 years

C. Fire Hose Stations

	Test	Frequency
1.	Visual Inspection	Monthly
2.	Hose Hydro-Test	Yearly
3.	Partially open each hose station valve to verify operability and no blockage	3 years

D. Fire Detection

	Test	Frequency
1.	Channel Functional Test	2 mo.

E. Fire Barrier Penetration Fire Seals

	Test	Frequency
1.	Visual Inspection	18 mo. and following repairs or maintenance

F. Fire Pump Diesel Engine

		Test	Frequency
1.	a.	Verify 200 gallons of fuel in fuel storage tank	Monthly
	b.	Verify diesel starts from ambient conditions and operates for at least 20 minutes.	Monthly
2.		Sample diesel fuel per ASTM-D270-65 and verify acceptable per Table 1 of ASTM-D975-74 with respect to viscosity, water content and sediment.	Quarterly
3.	a.	Inspect diesel per procedures prepared in conjunction with its manufacturer's recommendations	18 months
	b.	Verify diesel starts from ambient conditions and operates for $\geq$20 minutes while loaded with the fire pump	18 months

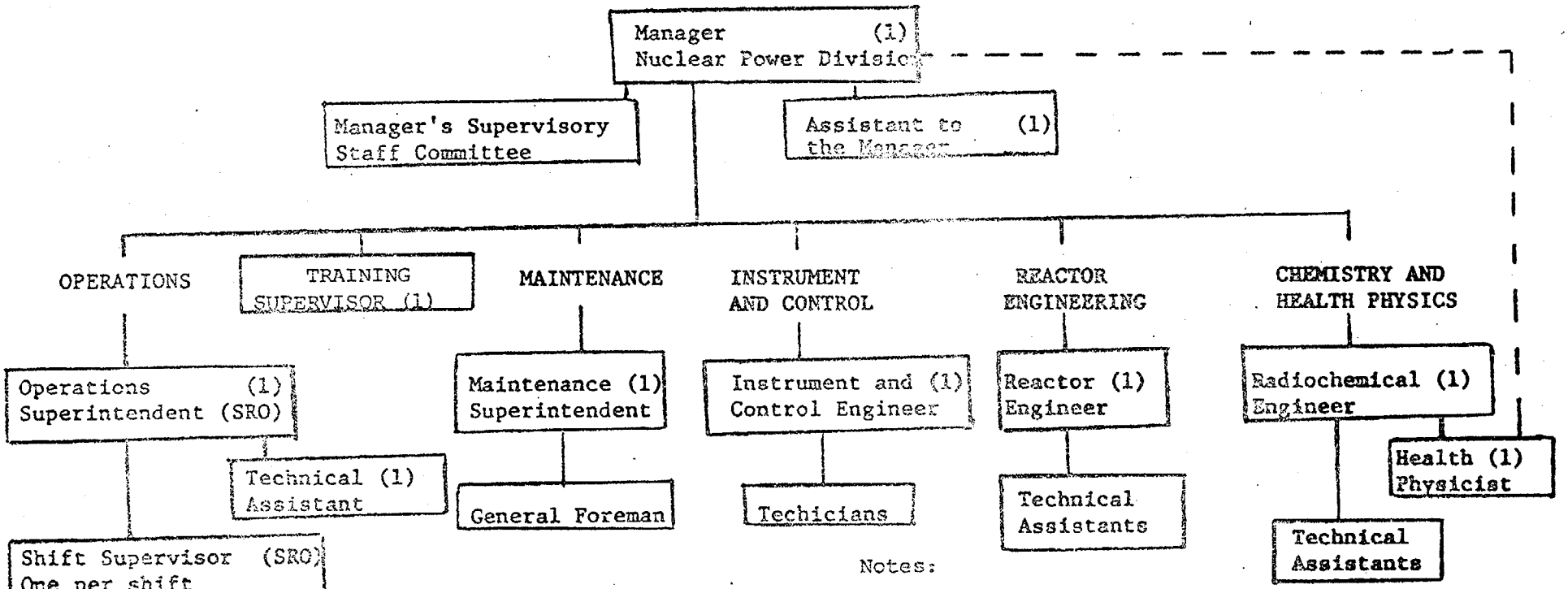
G. Fire Pump Diesel Battery and Charger

	Test	Frequency
1.	a. Verify electrolite level above the plates	Weekly
	b. Verify that the overall battery voltage is $\geq$ 24 volts	Weekly
2.	Verify the specific gravity is appropriate for continued service of the battery	Quarterly
3.	a. Verify that the battery, cell plates and battery racks show no visual indication of physical damage or abnormal deterioration	18 months
	b. Verify that the battery to battery and terminal connections are clean, tight, free of corrosion and coated with anti-corrosion material	18 months

Basis

Normally, the fire protection system is not in use. However, the system components are required to perform as designed in the event of a fire emergency. The National Fire Protection Association and the plant insurance carrier have specified periodic tests and inspections to demonstrate fire protection equipment operability. The listed tests and inspections include and exceed the requirements of these organizations. Testing more frequently than that listed is not considered necessary to insure operability and performance.

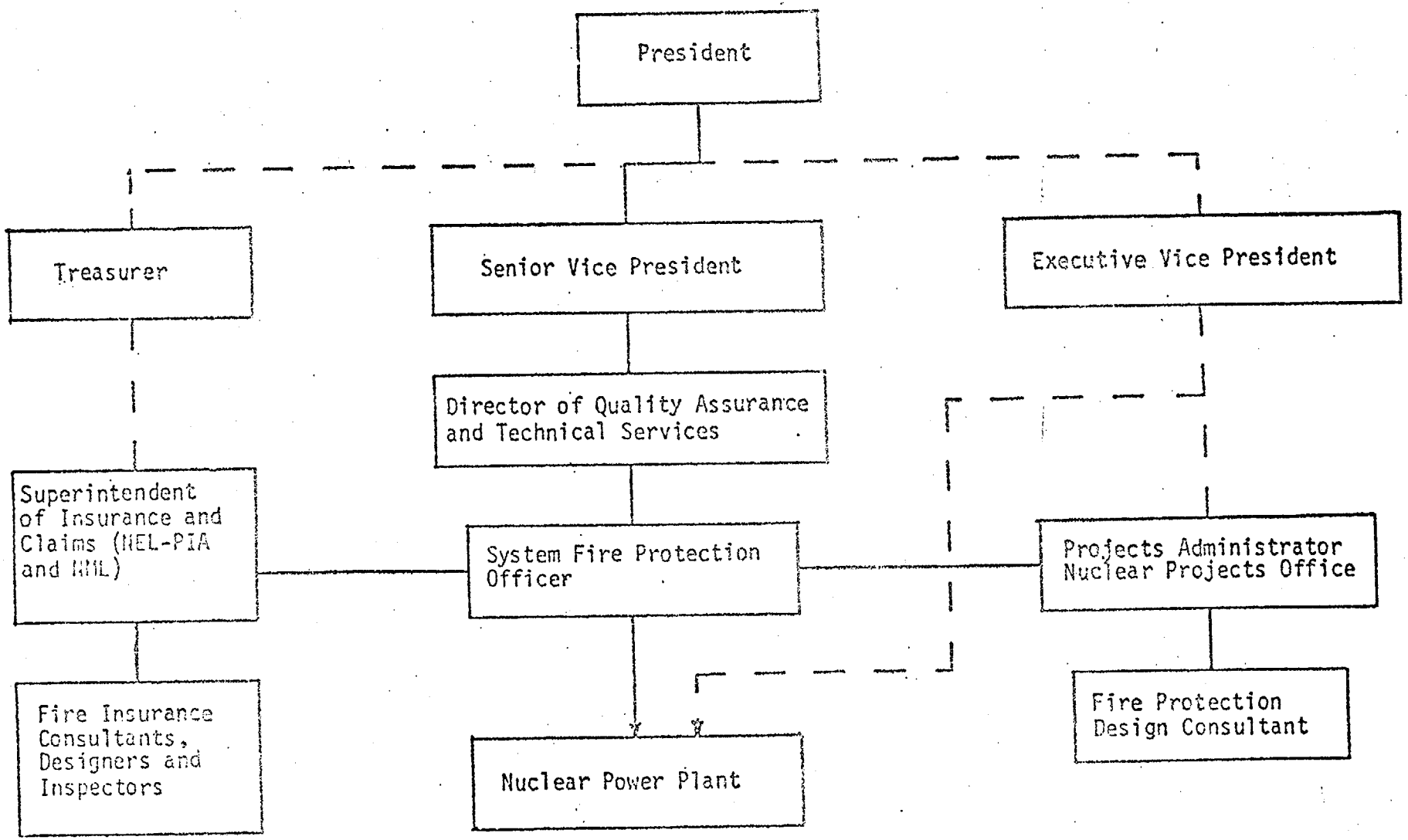
15.6.2.2.f. A Fire Brigade of at least 4 members shall be maintained onsite at all times. This excludes 3 members of the minimum shift crew necessary for safe shutdown of the plant and any personnel required for other essential functions during a fire emergency.



Notes:

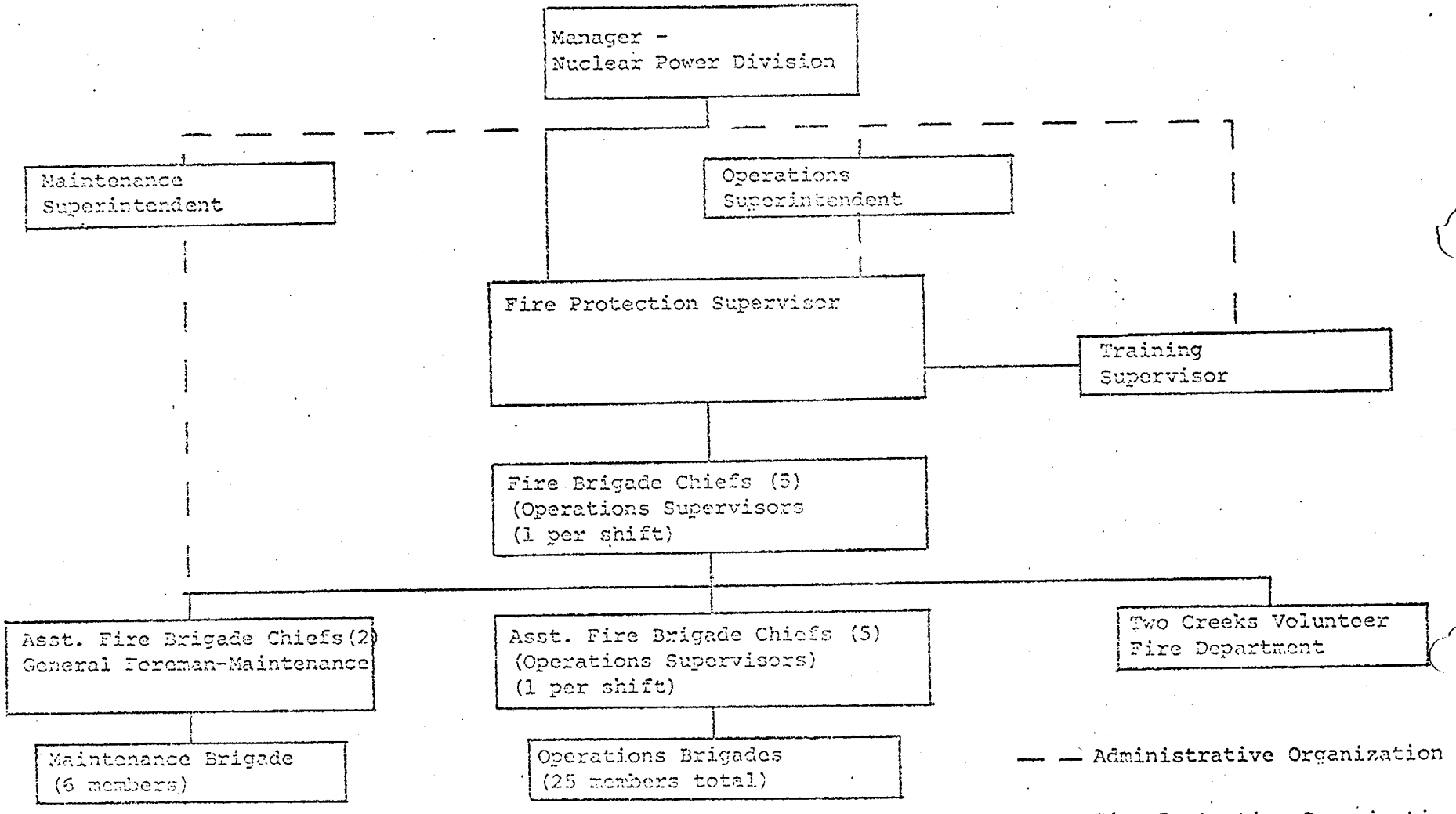
1. The Operations Group shift makeup is the minimum size for operation in all modes except cold shutdown of Point Beach Units Nos. 1 and 2. The Operations Group shift makeup may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members, provided immediate action is taken to restore the shift makeup to within the minimum requirements.
2. Health Physicist has optional reporting route to Manager on radiological health and safety matters.
3. SRO - NRC Senior Reactor Operator License
   RO - NRC Reactor Operator License

CONDUCT OF PLANT OPERATIONS CHART
Figure 15.6.2-2



WISCONSIN ELECTRIC POWER COMPANY
OFF-SITE MANAGEMENT
FIRE PROTECTION ORGANIZATION

Firure 15.6.2-3



POINT BEACH NUCLEAR PLANT
FIRE PROTECTION ORGANIZATION

Figure 15.6.2-4

15.6.3 FACILITY STAFF QUALIFICATIONS

15.6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions.

15.6.4 TRAINING

15.6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

15.6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Fire Protection Supervisor and shall meet or exceed the requirements of Section 27 of the NFPA Code-1976, except that the meeting frequency may by quarterly.

15.6.5 REVIEW AND AUDIT

15.6.5.1 Duty and Call Superintendents

a. To assist and counsel the Shift Supervisor in case of Significant Operating Events, a Duty and Call Superintendent Group has been established. The Duty and Call Superintendent Group shall consist of any qualified person designated by the Manager - Nuclear Power Division.

b. In the event of a reportable occurrence, the Shift Supervisor shall communicate with at least one Duty and Call Superintendent before taking other than the immediate on-the-spot action required. One Duty and Call Superintendent will be assigned to be "on call" at all times. The Duty and Call Superintendent provides continuously available counsel, call out backups, and review to the Shift Supervisor.

15.6.5.2 Manager's Supervisory Staff

FUNCTION

15.6.5.2.1 The Manager's Supervisory Staff (MSS) shall function to advise the Manager - Nuclear Power Division on all matters related to nuclear safety.

b) Review all proposed tests and experiments related to Safety and the results thereof when applicable.

c) Review all proposed changes to Technical Specifications.

d) Review all proposed changes or modifications to plant systems or equipment where changes would require a change in operating or emergency procedures or that affect nuclear safety.

e) Periodically review plant operations for industrial and nuclear safety hazards.

f) Investigate violations or suspected violations of Technical Specifications, such investigations to include reports, evaluations, and recommendations to prevent recurrence, to the Vice President - Nuclear Plant and to the Chairman of the Off-Site Review Committee.

g) Perform special reviews and investigations and prepare reports thereon as requested by the Chairman of the Off-Site Review Committee.

h) Investigate, review, and report on all reportable occurrences.

i) Cause to be conducted periodic drills on emergency procedures, including evacuation (partial or complete) of the site and check adequacy of communications with off-site support groups.

j) Review the Facility Fire Protection Program and implementing procedures at least once per 24 months.

AUTHORITY

15.6.5.2.7 a) The Supervisory Staff shall serve as advisory to the Manager - Nuclear Power Division.

b) The Supervisory Staff shall recommend to the Manager approval or disapproval of proposals under items a) through d) above. In the event of disagreement between a majority of the

15.6.5.4 Fire Protection Audits

a) An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.

b) An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years.

15.6.8 PLANT OPERATING PROCEDURES

15.6.8.1 The plant shall be operated and maintained in accordance with approved procedures. Major procedures, supported by appropriate minor procedures (such as checkoff lists, operating instructions, data sheets, alarm responses, chemistry analytical procedures, etc.) shall be provided for the following operations where these operations involve nuclear safety of the plant:

1. Normal sequences of startup, operation and shutdown of components, systems and overall plant.
2. Refueling.
3. Specific and foreseen potential malfunctions of systems or components including abnormal reactivity changes.
4. Security Plan Implementation
5. Emergencies which could involve release of radioactivity.
6. Nuclear core testing.
7. Surveillance and Testing of safety related equipment.
8. Fire Protection Implimentation (to be provided by March 31, 1978).

15.6.8.2 Approval of Procedures

A. All major procedures of the categories listed in 15.6.8.1 (except 15.6.8.1.4) and 15.6.11.1, and modifications to the intent thereof, shall be reviewed by the Manager's Supervisory Staff and approved by the Manager - Nuclear Power Division prior to implementation.

B. Minor procedures (checkoff lists, operating instructions, data sheets, alarm responses, chemistry analytical procedures, technical instructions, special and routine maintenance procedures, laboratory manuals, etc.) shall, prior to initial use, be reviewed by the Manager's Supervisory Staff and approved by the Manager - Nuclear Power Division.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY
WISCONSIN MICHIGAN POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 36
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Wisconsin Electric Power Company and Wisconsin Michigan Power Company (the licensees) dated July 28, 1977 and revised by letters dated October 5 and December 12, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the heatlh and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-27 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 36, are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 23, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 36

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NO. 50-301

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised page is identified by Amendment number and contains vertical lines indicating the area of change.

Remove	Replace
15i	15i
-	15ii
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-	15.3.14-2
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-	Figure 15.6.2-3
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15.6.5-3	15.6.5-3
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TABLE OF CONTENTS

TABLE OF CONTENTS (CONT'D)

l. Degree of Redundancy

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The reactor is said to be critical when the neutron chain reaction is self-sustaining and $k_{eff} = 1.0$.

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The reactor is in the low power operating condition when the reactor is critical and the average neutron flux of the power range instrumentation indicates less than or equal to 2% of rated power.

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15.3.14 Fire Protection System

Applicability

Applies to the operability of the fire protection system components which provide fire suppression capability for equipment required for safe plant shutdown at all times when those systems are required to be operable.

Objective

To specify the functional requirements for fire protection system components which would be employed to mitigate the consequences of fires which could affect equipment required for safe plant shutdown.

Specification

A. Fire Suppression Water System

One of the following conditions shall be provided:

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3. With both fire pumps inoperable,
   a. Establish a backup FIRE SUPPRESSION WATER SYSTEM within 24 hours.
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   c. If a. above cannot be fulfilled, place the reactor in Hot Standby within the next 6 hours and in Cold Shutdown within the following thirty (30) hours.

B. Spray and/or Sprinkler Systems

The following systems shall be operable.

Diesel Generator 3D Dry Pipe Sprinkler System

Diesel Generator 4D Dry Pipe Sprinkler System

1. With an above listed system inoperable, local hose station fire suppression equipment for the affected area shall be verified operable within 1 hour.
2. Additional portable fire suppression equipment shall be provided for the affected area.
3. A fire watch inspection shall be performed in the affected area twice per shift. Activity within the affected area shall be restricted to that which is necessary for continued operation.

C. Fire Hose Stations

The following hose stations shall be operable:

HR-13 South Wall Control Building Elev. 8'-0"

HR-15 South Wall Control Building Elev. 44'-0"

HR-16 North Wall Control Building Elev. 8'-0"

HR-18 North Wall Control Building Elev. 44'-0"

HR-31 West Wall Auxiliary Building Elev. 8'-0"

1. With a hose station inoperable, backup portable fire suppression equipment for the affected area shall be verified operable within 1 hour.
2. Appropriate backup portable fire suppression equipment shall be provided for the affected area.

D. Fire Detection

The fire detection instrumentation for each fire detection zone shown in Table 15.3.14-1 shall be operable.

1. With a fire detection instrument inoperable, the affected area shall be inspected to assure that potential fire hazards are minimized.

2. A fire watch inspection shall be performed in the affected area once per hour. Activity in the affected area shall be restricted to that which is necessary for continued operation.

E. Fire Barrier Penetration Fire Seals

All penetration fire barriers protecting safety related areas shall be functional.

1. In the event of a penetration fire barrier impairment a continuous fire watch shall be established on at least one side of the affected penetration within 1 hour.

Basis

The overall fire protection program at Point Beach Nuclear Plant utilizes the principles of defense in depth. This includes minimization of combustibles, early warning fire detection, primary and backup fire suppression capability and priority maintenance procedures to restore inoperable equipment to operable status as soon as possible. Collectively these measures provide adequate capability to minimize potential damage to safety related equipment and to allow for safe plant shutdown in the event of a potential fire occurrence.

Should a portion or component of the fire protection system be inoperable, these specifications provide assurance that alternate methods of fire protection are strengthened and that the capability to mitigate the consequences of a potential fire is maintained.

TABLE 15.3.14-1

	Area	Quantity	Elevation	Type
1.	Cable Spreading Room	2	26'-0"	Smoke
2.	Switchgear Room	1	8'-0"	Smoke
3.	Diesel Generator 3D Room	1	8'-0"	Smoke
4.	Diesel Generator 4D Room	1	8'-0"	Smoke
5.	Fuel Oil Pumphouse	1	25'-7"	Smoke
6.	Unit 1 Electrical Equip. Room	1	46'-0"	Smoke
7.	Unit 2 Electrical Equip. Room	1	46'-0"	Smoke
8.	Circulating Water Pumphouse	6	7'-0"	Smoke

TABLE 15.4.1-2 (CONTINUED)

		Test	Frequency	FSAR Section Reference
(14)	Refueling System Interlocks	Functioning	Each refueling shutdown	9.4.5
(15)	Service Water System	Functioning	Each refueling shutdown	9.5.5
(16)	Primary System Leakage	Evaluate	Monthly(6)	4
(17)	Diesel Fuel Supply	Fuel inventory	Daily	8.2.3
(18)	Turbine Stop and Governor Valves	Functioning	Monthly(6) (9)	10
(19)	Low Pressure Turbine Rotor Inspection(5)	Visual and magnetic particle or liquid penetrant	Every five years	10
(20)	Boric Acid System	Storage Tank Temperature	Daily	
(21)	Boric Acid System	Visual observation of piping temperatures (all ≥145°F)	Daily	
(22)	Boric Acid Piping Heat Tracing	Electrical circuit operability	Monthly	

(1) A radiochemical analysis for this purpose shall consist of a quantitative measurement of each radionuclide with half life of >30 minutes such that at least 95% of total activity of primary coolant is accounted for.
(2) $\bar{E}$ determination will be started when the gross activity analysis of a filtered sample indicates ≥10 μc/cc and will be redetermined if the primary coolant gross radioactivity of a filtered sample increases by more than 10 μc/cc.
(3) Drop tests shall be conducted at rated reactor coolant flow. Rods shall be dropped under both cold and hot conditions, but cold drop tests need not be timed.
(4) Drop tests will be conducted in the hot condition for rods on which maintenance was performed.
(5) As accessible without disassembly of rotor.
(6) Not required during periods of refueling shutdown.
(7) At least once per week during periods of refueling shutdown.
(8) At least three times per week (with maximum time of 72 hours between samples) during periods of refueling shutdown.
(9) a. The monthly functional test interval for Point Beach Unit No. 2 for February 1976 is extended seven days to February 26, 1976.
b. The requirement of the monthly functional test for Point Beach Unit No. 1 is waived for the one month period of September, 1977.

15.4.15 Fire Protection System

Applicability

Applies to the periodic inspection and testing requirements of fire protection equipment.

Objective

To verify the operability of fire protection equipment.

Specification

Testing of fire protection system equipment, as a minimum, shall be done as follows:

A. Fire Suppression Water System

	Test	Frequency
1.	Verify valves in the Flow Path (automatic and manual) in the correct position.	Monthly
2.	Fire Pump Functional Test	Monthly
3.	Fire Pump Capacity Test	Yearly
4.	Automatic system and valve actuation and flowpath valve cycle tests	Yearly
5.	System flow Test (In accordance with Ch. 5, Sect. 11 F.P. Handbook)	3 Years

B. Spray and/or Sprinkler Systems

	Test	Frequency
1.	Complete Cycle of Each Testable Valve	Yearly
2.	Simulated System Functional Test	Yearly
3.	Visually inspect headers and nozzles	18 mo.
4.	Air flow test to verify open head nozzles unobstructed	3 years

C. Fire Hose Stations

	Test	Frequency
1.	Visual Inspection	Monthly
2.	Hose Hydro-Test	Yearly
3.	Partially open each hose station valve to verify operability and no blockage	3 years

D. Fire Detection

	Test	Frequency
1.	Channel Functional Test	2 mo.

E. Fire Barrier Penetration Fire Seals

	Test	Frequency
1.	Visual Inspection	18 mo. and following repairs or maintenance

F. Fire Pump Diesel Engine

		Test	Frequency
1.	a.	Verify 200 gallons of fuel in fuel storage tank	Monthly
	b.	Verify diesel starts from ambient conditions and operates for at least 20 minutes.	Monthly
2.		Sample diesel fuel per ASTM-D270-65 and verify acceptable per Table 1 of ASTM-D975-74 with respect to viscosity, water content and sediment.	Quarterly
3.	a.	Inspect diesel per procedures prepared in conjunction with its manufacturer's recommendations	18 months
	b.	Verify diesel starts from ambient conditions and operates for $\geq$20 minutes while loaded with the fire pump	18 months

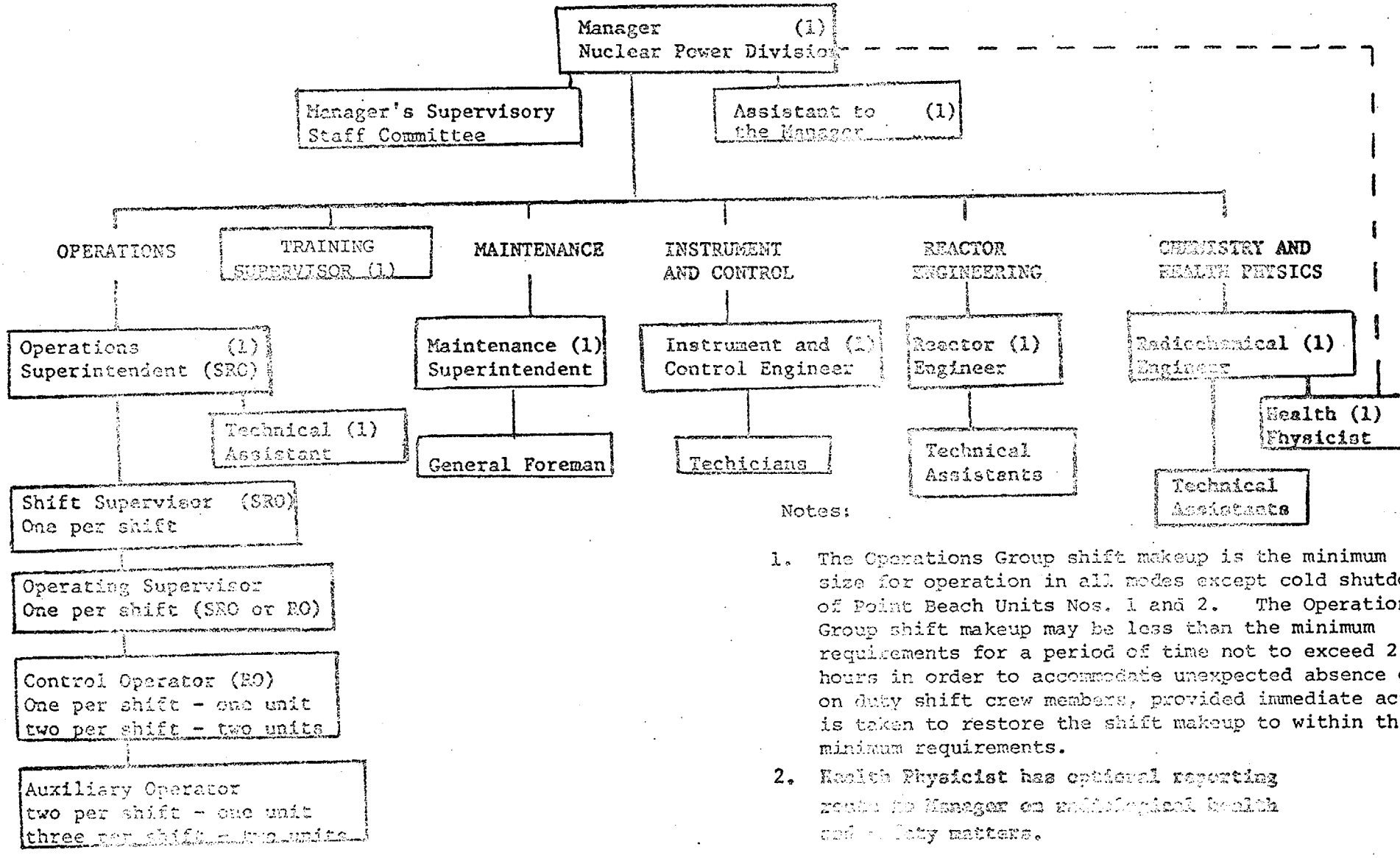
G. Fire Pump Diesel Battery and Charger

	Test	Frequency
1.	a. Verify electrolite level above the plates	Weekly
	b. Verify that the overall battery voltage is $\geq$ 24 volts	Weekly
2.	Verify the specific gravity is appropriate for continued service of the battery	Quarterly
3.	a. Verify that the battery, cell plates and battery racks show no visual indication of physical damage or abnormal deterioration	18 months
	b. Verify that the battery to battery and terminal connections are clean, tight, free of corrosion and coated with anti-corrosion material	18 months

Basis

Normally, the fire protection system is not in use. However, the system components are required to perform as designed in the event of a fire emergency. The National Fire Protection Association and the plant insurance carrier have specified periodic tests and inspections to demonstrate fire protection equipment operability. The listed tests and inspections include and exceed the requirements of these organizations. Testing more frequently than that listed is not considered necessary to insure operability and performance.

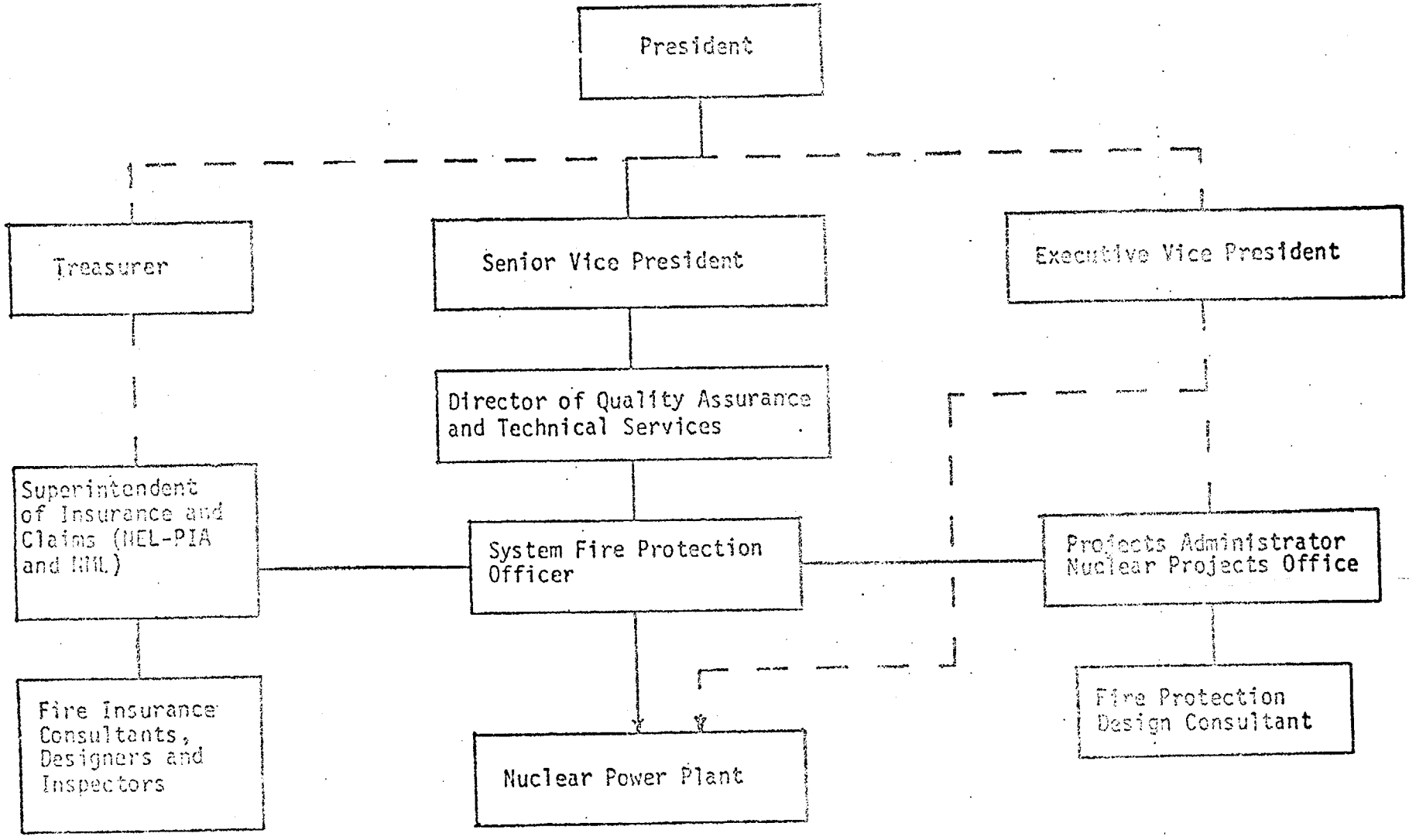
15.6.2.2.f. A Fire Brigade of at least 4 members shall be maintained onsite at all times. This excludes 3 members of the minimum shift crew necessary for safe shutdown of the plant and any personnel required for other essential functions during a fire emergency.



Notes:

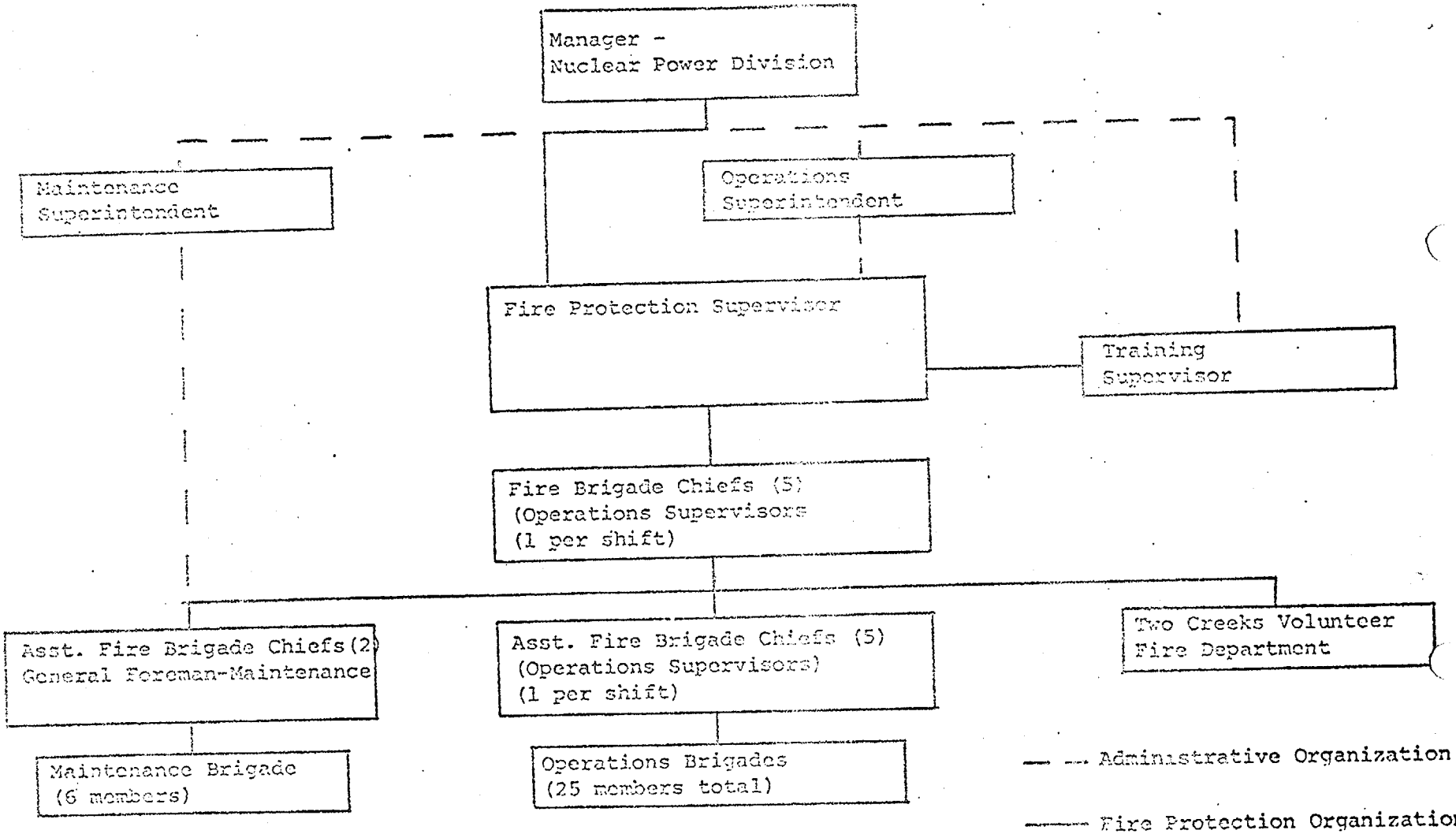
1. The Operations Group shift makeup is the minimum size for operation in all modes except cold shutdown of Point Beach Units Nos. 1 and 2. The Operations Group shift makeup may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members, provided immediate action is taken to restore the shift makeup to within the minimum requirements.
2. Health Physicist has optional reporting route to Manager on radiological health and safety matters.
3. SRO - NRC Senior Reactor Operator License
   RO - NRC Reactor Operator License

CONDUCT OF PLANT OPERATIONS
Figure 15.6.2-2



WISCONSIN ELECTRIC POWER COMPANY
OFF-SITE MANAGEMENT
FIRE PROTECTION ORGANIZATION

Firure 15.6.2-3



POINT BEACH NUCLEAR PLANT
FIRE PROTECTION ORGANIZATION

Figure 15.6.2-1

15.6.3 FACILITY STAFF QUALIFICATIONS

15.6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions.

15.6.4 TRAINING

15.6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

15.6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Fire Protection Supervisor and shall meet or exceed the requirements of Section 27 of the NFPA Code-1976, except that the meeting frequency may by quarterly.

15.6.5 REVIEW AND AUDIT

15.6.5.1 Duty and Call Superintendents

a. To assist and counsel the Shift Supervisor in case of Significant Operating Events, a Duty and Call Superintendent Group has been established. The Duty and Call Superintendent Group shall consist of any qualified person designated by the Manager - Nuclear Power Division.

b. In the event of a reportable occurrence, the Shift Supervisor shall communicate with at least one Duty and Call Superintendent before taking other than the immediate on-the-spot action required. One Duty and Call Superintendent will be assigned to be "on call" at all times. The Duty and Call Superintendent provides continuously available counsel, call out backups, and review to the Shift Supervisor.

15.6.5.2 Manager's Supervisory Staff

FUNCTION

15.6.5.2.1 The Manager's Supervisory Staff (MSS) shall function to advise the Manager - Nuclear Power Division on all matters related to nuclear safety.

b) Review all proposed tests and experiments related to Safety and the results thereof when applicable.

c) Review all proposed changes to Technical Specifications.

d) Review all proposed changes or modifications to plant systems or equipment where changes would require a change in operating or emergency procedures or that affect nuclear safety.

e) Periodically review plant operations for industrial and nuclear safety hazards.

f) Investigate violations or suspected violations of Technical Specifications, such investigations to include reports, evaluations, and recommendations to prevent recurrence, to the Vice President - Nuclear Plant and to the Chairman of the Off-Site Review Committee.

g) Perform special reviews and investigations and prepare reports thereon as requested by the Chairman of the Off-Site Review Committee.

h) Investigate, review, and report on all reportable occurrences.

i) Cause to be conducted periodic drills on emergency procedures, including evacuation (partial or complete) of the site and check adequacy of communications with off-site support groups.

j) Review the Facility Fire Protection Program and implementing procedures at least once per 24 months.

AUTHORITY

15.6.5.2.7 a) The Supervisory Staff shall serve as advisory to the Manager - Nuclear Power Division.

b) The Supervisory Staff shall recommend to the Manager approval or disapproval of proposals under items a) through d) above. In the event of disagreement between a majority of the

15.6.5.4 Fire Protection Audits

a) An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.

b) An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years.

15.6.8 PLANT OPERATING PROCEDURES

15.6.8.1 The plant shall be operated and maintained in accordance with approved procedures. Major procedures, supported by appropriate minor procedures (such as checkoff lists, operating instructions, data sheets, alarm responses, chemistry analytical procedures, etc.) shall be provided for the following operations where those operations involve nuclear safety of the plant:

1. Normal sequences of startup, operation and shutdown of components, systems and overall plant.
2. Refueling.
3. Specific and foreseen potential malfunctions of systems or components including abnormal reactivity changes.
4. Security Plan Implementation
5. Emergencies which could involve release of radioactivity.
6. Nuclear core testing.
7. Surveillance and Testing of safety related equipment.
8. Fire Protection Implimentation (to be provided by March 31, 1978).

15.6.8.2 Approval of Procedures

A. All major procedures of the categories listed in 15.6.8.1 (except 15.6.8.1.4) and 15.6.11.1, and modifications to the intent thereof, shall be reviewed by the Manager's Supervisory Staff and approved by the Manager - Nuclear Power Division prior to implementation.

B. Minor procedures (checkoff lists, operating instructions, data sheets, alarm responses, chemistry analytical procedures, technical instructions, special and routine maintenance procedures, laboratory manuals, etc.) shall, prior to initial use, be reviewed by the Manager's Supervisory Staff and approved by the Manager - Nuclear Power Division.

7590-01

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-266 AND 50-301

WISCONSIN ELECTRIC POWER COMPANY
WISCONSIN MICHIGAN POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 32 and 36 to Facility Operating Licenses Nos. DPR-24 and DPR-27 issued to Wisconsin Electric Power Company and Wisconsin Michigan Power Company, which revised Technical Specifications for operation of the Point Beach Nuclear Plant Units Nos. 1 and 2, located in the town of Two Creeks, Manitowac County, Wisconsin. The amendments are effective as of the date of issuance.

These amendments consist of changes in the Technical Specifications that incorporate the Fire Protection System into the Limitng Conditions for Operation, Surveillance Requirements and Administrative Controls.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chpater I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative

declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated July 28, 1977 as revised by letters dated October 5 and December 12, 1977, (2) Amendment No. 32 to License DPR-24, (3) Amendment No. 36 to License DPR-27, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the University of Wisconsin - Stevens Point Library, ATTN: Mr. Arthur M. Fish, Stevens Point, Wisconsin 54481. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 23rd day of January 1978.

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors