

3/17/76

Dockets Nos. 50-266
and 50-301

Wisconsin Electric Power Company
Wisconsin Michigan Power Company
ATTN: Mr. Sol Burstein
Executive Vice President
231 West Michigan Street
Milwaukee, Wisconsin 53201

Gentlemen:

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In response to your requests dated September 8, 1975 and January 30, 1976, the Commission has issued the enclosed Amendments Nos. 15 and 20 to Facility Operating Licenses Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant, Units Nos. 1 and 2.

The amendment consists of a license amendment and Technical Specification change relating to the receipt, possession, and use of byproduct, source, and special nuclear material.

Our current procedure for the licensing of byproduct, source, and special nuclear materials included in reactor licenses is not to specify quantity limits. Therefore, we have issued this amendment consistent with that procedure.

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosures:

1. Amendment No. 15 to License DPR-24
2. Amendment No. 20 to License DPR-27
3. Safety Evaluation
4. Federal Register Notice

cc w/encs:				
See next page	ORB#3	OELD	ORB#3	
SURNAME >	CParrish ^{up}	JWetmore ^{acr}	KGlear ⁶	
DATE >	2/24/76	2/24/76	3/12/76	

Wisconsin Michigan Power Company
Wisconsin Electric Power Company

- 2 -

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY
WISCONSIN MICHIGAN POWER COMPANY

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company and Wisconsin Michigan Power Company (the licensees) dated September 8, 1975, and Supplement dated January 30, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
 - E. The receipt, possession and use of the byproduct, source and special nuclear material as authorized by this license, as amended, will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40 and 70, including Sections 30.33, 40.32, 70.23 and 70.31.
 - F. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Facility Operating License No. DPR-24, as amended, is hereby further amended by replacing in their entirety paragraphs 2, 2.A., 2.B., 2.C., and 2.D. thereof with the following:

"2. Subject to the conditions and requirements incorporated herein,

the Commission hereby licenses

- A. Pursuant to Section 104b of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," to possess, use, and operate the facility at the designated location on the Point Beach site in accordance with the procedures and limitations set forth in this license;
 - B. Pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in Final Facility Description and Safety Analysis Report, as supplemented and amended as of March 17, 1976;
 - C. Pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - D. Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
 - E. Pursuant to the Act and 10 CFR Parts 30 and 70, to possess such byproduct and special nuclear materials as may be produced by the operation of the facility, but not to separate such materials retained within the fuel cladding."
3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the
Technical Specifications

Date of Issuance: March 17, 1976



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY
WISCONSIN MICHIGAN POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 20
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Wisconsin Electric Power Company and Wisconsin Michigan Power Company (the licensees) dated September 8, 1975, and Supplement dated January 30, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
 - E. The receipt, possession and use of the byproduct, source and special nuclear material as authorized by this license, as amended, will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40 and 70, including Sections 30.33, 40.32, 70.23 and 70.31.
 - F. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Facility Operating License No. DPR-27, as amended, is hereby further amended by replacing in their entirety paragraphs 2, 2.A., 2.B., 2.C., and 2.D. thereof with the following:

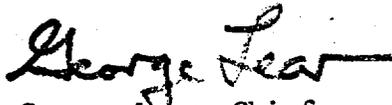
"2. Subject to the conditions and requirements incorporated herein,

the Commission hereby licenses

- A. Pursuant to Section 104b of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," to possess, use, and operate the facility at the designated location on the Point Beach site in accordance with the procedures and limitations set forth in this license;
- B. Pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in Final Facility Description and Safety Analysis Report, as supplemented and amended as of March 17, 1976;
- C. Pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- D. Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
- E. Pursuant to the Act and 10 CFR Parts 30 and 70, to possess such byproduct and special nuclear materials as may be produced by the operation of the facility, but not to separate such materials retained within the fuel cladding."

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the
Technical Specifications

Date of Issuance: March 17, 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 15

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-24

DOCKET NO. 50-266

Replace pages 15-i, 15.4.12-1, 15.4.12-2 and 15.6.4-2 of the Technical Specifications with the attached revised pages. Add page 15.6.5-11.

<u>Section</u>	<u>Title</u>	<u>Page</u>
15	TECHNICAL SPECIFICATIONS AND BASES	
15.1	Definitions	15.1-1
15.2.0	Safety Limits and Limiting Safety System Settings	15.2.1-1
15.2.1	Safety Limit, Reactor Core	15.2.1-1
15.2.2	Safety Limit, Reactor Coolant System Pressure	15.2.2-1
15.2.3	Limiting Safety System Settings, Protective Instrumentation	15.2.3-1
15.3	Limiting Conditions for Operation	15.3-1
15.3.1	Reactor Coolant System	15.3.1-1
15.3.2	Chemical and Volume Control System	15.3.2-1
15.3.3	Emergency Core Cooling System, Auxiliary Cooling Systems, Air Recirculation Fan Coolers, and Containment Spray	15.3.3-1
15.3.4	Steam and Power Conversion System	15.3.4-1
15.3.5	Instrumentation System	15.3.5-1
15.3.6	Containment System	15.3.6-1
15.3.7	Auxiliary Electrical Systems	15.3.7-1
15.3.8	Refueling	15.3.8-1
15.3.9	Effluent Releases	15.3.9-1
15.3.10	Control Rod and Power Distribution Limits	15.3.10-1
15.3.11	Movable In-Core Instrumentation	15.3.11-1
15.3.12	Control Room Emergency Filtration	15.3.12-1
15.4	Surveillance Requirements	15.4-1
15.4.1	Operational Safety Review	15.4.1-1
15.4.2	In-service Inspection of Primary System Components	15.4.2-1
15.4.3	Primary System Testing Following Opening	15.4.3-1
15.4.4	Containment Tests	15.4.4-1
15.4.5	Emergency Core Cooling System and Containment Cooling System Tests	15.4.5-1
15.4.6	Emergency Power System Periodic Tests	15.4.6-1
15.4.7	Main Steam Stop Valves	15.4.7-1
15.4.8	Auxiliary Feedwater System	15.4.8-1
15.4.9	Reactivity Anomalies	15.4.9-1
15.4.10	Operational Environmental Monitoring	15.4.10-1
15.4.11	Control Room Emergency Filtration	15.4.11-1
15.4.12	Miscellaneous Radioactive Material Sources	15.4.12-1
15.5	Design Features	15.5.1-1
15.5.1	Site	15.5.1-1
15.5.2	Containment	15.5.2-1
15.5.3	Reactor	15.5.3-1
15.5.4	Fuel Storage	15.5.4-1
15.6	Administrative Controls	15.6.1-1
15.6.1	Organization, Review and Audit	15.6.1-1
15.6.2	Action to be Taken in the event of an Abnormal Occurrence in Plant Operation	15.6.2-1
15.6.3	Action to be Taken if a Safety Limit is Exceeded	15.6.3-1
15.6.4	Plant Operating Procedures	15.6.4-1
15.6.5	Plant Operating Records	15.6.5-1
15.6.6	Plant Reporting Requirements	15.6.6-1
15.6.7	Respiratory Protection for Airborne Radioactivity	15.6.7-1

15.4.12 MISCELLANEOUS RADIOACTIVE MATERIALS SOURCES

Applicability

Applies to periodic testing requirements of miscellaneous radioactive materials sealed sources.

Objective

To verify that potential leakage of sealed radioactive sources is maintained within acceptable limits.

Specification

A. Source Leakage Test

1. Sealed radioactive sources as described below shall be leak tested for contamination. The leakage test shall be capable of detecting the presence of 0.005 microcurie of radioactive material on the test sample. If the test reveals the presence of 0.005 microcurie or more of removable contamination, it shall immediately be withdrawn from use, decontaminated, and repaired, or be disposed of in accordance with Commission regulations.
2. Sealed sources containing quantities of by-product material that exceed the quantities listed in 10 CFR 30.71 Schedule B shall be leak tested in accordance with the Surveillance Requirements below. Sealed alpha sources containing greater than 0.1 microcuries are also to be leak tested in accordance with the Surveillance Requirements.

B. Surveillance Requirements

Tests for leakage and/or contamination shall be performed by the licensee or by other persons specifically authorized by the Commission or an agreement State, as follows:

1. Each sealed source, except startup sources and sources in fission detectors subject to core flux, containing radioactive material, other than Hydrogen-3, with a half-life greater than thirty days and in any form other than gas shall be tested for leakage and/or contamination at intervals not to exceed six months.
2. The periodic leak test required does not apply to sealed sources that are stored and not being used. Such stored sources shall be tested for leakage prior to any use or transfer to another user unless they have been leak tested within six months prior to the date of use or transfer.
3. Startup sources shall be leak tested prior to and following any repair or maintenance and before being subjected to core flux.

Basis

Ingestion or inhalation of source material may give rise to total body or organ irradiation. This specification assures that leakage from radioactive material sources does not exceed allowable limits. In the unlikely event that those quantities of radioactive by-product materials of interest to this specification which are exempt from leakage testing are ingested or inhaled, they represent less than one maximum permissible body burden for total body irradiation. The limits for all other sources (including alpha emitters) are based upon 10 CFR 70.39 limits for plutonium.

- K. Records of special reactor tests or experiments.
- L. Records of changes made in the Operating Procedures.
- M. Test results, in units of microcuries, for leak test performed pursuant to Specification 15.4.12.
- N. Record of annual physical inventory verifying accountability of sources subject to Specification 15.4.12.

* Starred items will be permanently retained.

15.6.4-2

4. Leak Testing of Sources

Results of required leak tests performed on sealed sources if the tests reveal the presence of 0.005 microcurie or more of removable contamination.

15.6.5-11

ATTACHMENT TO LICENSE AMENDMENT NO. 20

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-27

DOCKET NO. 50-301

Replace pages 15-i, 15.4.12-1, 15.4.12-2, and 15.6.4-2 of the Technical Specifications with the attached revised pages. Add page 15.6.5-11.

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15.3.7	Auxiliary Electrical Systems	15.3.7-1
15.3.8	Refueling	15.3.8-1
15.3.9	Effluent Releases	15.3.9-1
15.3.10	Control Rod and Power Distribution Limits	15.3.10-1
15.3.11	Movable In-Core Instrumentation	15.3.11-1
15.3.12	Control Room Emergency Filtration	15.3.12-1
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15.4.8	Auxiliary Feedwater System	15.4.8-1
15.4.9	Reactivity Anomalies	15.4.9-1
15.4.10	Operational Environmental Monitoring	15.4.10-1
15.4.11	Control Room Emergency Filtration	15.4.11-1
15.4.12	Miscellaneous Radioactive Material Sources	15.4.12-1
15.5	Design Features	15.5.1-1
15.5.1	Site	15.5.1-1
15.5.2	Containment	15.5.2-1
15.5.3	Reactor	15.5.3-1
15.5.4	Fuel Storage	15.5.4-1
15.6	Administrative Controls	15.6.1-1
15.6.1	Organization, Review and Audit	15.6.1-1
15.6.2	Action to be Taken in the event of an Abnormal Occurrence in Plant Operation	15.6.2-1
15.6.3	Action to be Taken if a Safety Limit is Exceeded	15.6.3-1
15.6.4	Plant Operating Procedures	15.6.4-1
15.6.5	Plant Operating Records	15.6.5-1
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15.6.7	Respiratory Protection for Airborne Radioactivity	15.6.7-1

15.4.12 MISCELLANEOUS RADIOACTIVE MATERIALS SOURCES

Applicability

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Objective

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Specification

A. Source Leakage Test

1. Sealed radioactive sources as described below shall be leak tested for contamination. The leakage test shall be capable of detecting the presence of 0.005 microcurie of radioactive material on the test sample. If the test reveals the presence of 0.005 microcurie or more of removable contamination, it shall immediately be withdrawn from use, decontaminated, and repaired, or be disposed of in accordance with Commission regulations.
2. Sealed sources containing quantities of by-product material that exceed the quantities listed in 10 CFR 30.71 Schedule B shall be leak tested in accordance with the Surveillance Requirements below. Sealed alpha sources containing greater than 0.1 microcuries are also to be leak tested in accordance with the Surveillance Requirements.

B. Surveillance Requirements

Tests for leakage and/or contamination shall be performed by the licensee or by other persons specifically authorized by the Commission or an agreement State, as follows:

1. Each sealed source, except startup sources and sources in fission detectors subject to core flux, containing radioactive material, other than Hydrogen-3, with a half-life greater than thirty days and in any form other than gas shall be tested for leakage and/or contamination at intervals not to exceed six months.
2. The periodic leak test required does not apply to sealed sources that are stored and not being used. Such stored sources shall be tested for leakage prior to any use or transfer to another user unless they have been leak tested within six months prior to the date of use or transfer.
3. Startup sources shall be leak tested prior to and following any repair or maintenance and before being subjected to core flux.

Basis

Ingestion or inhalation of source material may give rise to total body or organ irradiation. This specification assures that leakage from radioactive material sources does not exceed allowable limits. In the unlikely event that those quantities of radioactive by-product materials of interest to this specification which are exempt from leakage testing are ingested or inhaled, they represent less than one maximum permissible body burden for total body irradiation. The limits for all other sources (including alpha emitters) are based upon 10 CFR 70.39 limits for plutonium.

15.4.12-2

- K. Records of special reactor tests or experiments.
- L. Records of changes made in the Operating Procedures.
- M. Test results, in units of microcuries, for leak test performed pursuant to Specification 15.4.12.
- N. Record of annual physical inventory verifying accountability of sources subject to Specification 15.4.12.

* Starred items will be permanently retained.

15.6.4-2

4. Leak Testing of Sources

Results of required leak tests performed on sealed sources if the tests reveal the presence of 0.005 microcurie or more of removable contamination.

15.6.5-11



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENTS NOS 15 AND 20 TO LICENSES DPR-24/27

WISCONSIN ELECTRIC POWER COMPANY
WISCONSIN MICHIGAN POWER COMPANY

POINT BEACH NUCLEAR PLANT, UNITS 1/2

DOCKETS NOS. 50-266/301

Introduction

By letter dated September 8, 1975, Wisconsin Electric Power Company (WEPCO) proposed license amendments to Facility Operating Licenses DPR-24 and DPR-27 for Point Beach Nuclear Plant, Units 1 and 2. In response to our December 19, 1975 request, WEPCO provided additional information to support their proposed amendments by letter dated January 30, 1976.

The proposed amendments involve the modification of those parts of the Facility Operating Licenses which relate to the receipt, possession, and use of byproduct, source, and special nuclear material.

In support of the proposed license amendments, WEPCO has:

- a. Proposed Technical Specification changes which (1) provide for leakage testing of miscellaneous radioactive materials sources, (2) establish surveillance requirements for the leakage tests, and (3) require retention of leakage test results.
- b. Updated the Radioactive Materials Safety portion of the Final Facility Description & Safety Analysis Report (FFDSAR) for Point Beach Nuclear Plant Units 1 and 2.

Discussion

By letter dated December 16, 1974, we requested a number of nuclear power facility licensees, to provide: (1) proposed amendments to the conditions of existing Facility Operating Licenses which relate to the receipt, possession, and use of byproduct, source, and special nuclear materials; (2) proposed Technical Specification changes which provide for leakage testing and the related surveillance and reporting requirements for miscellaneous radioactive material sources; (3) FSAR (FFDSAR in the case of Point Beach) revisions to include information described in Regulatory Guide 1.70.3, "Additional Information, Radioactive Materials Safety For Nuclear Power Plants", of February 1974. Our letter included standard formats and guidelines for the requested proposals.

The objective of the requests made in our letter of December 16, 1974 was to add flexibility to the operation of nuclear power plants by establishing a more generalized approach to the licensing of byproduct, source, and special nuclear materials. This objective would reduce the number of licensing actions required as a result of changes in possession limits of related materials. In order to assure that adequate safeguards be maintained within the framework of this more generalized approach, provisions for more stringent control, accountability, and leakage testing of byproduct, source and special nuclear materials were included.

Evaluation

The proposed Technical Specification changes and the FFDSAR revisions have been reviewed by the NRC staff with particular attention to the Radioactive Materials Safety program. We evaluated the personnel qualifications, facilities, equipment, and procedures for handling byproduct, source, and special nuclear material, as described in the revised FFDSAR and we conclude that they are consistent with the provisions of Regulatory Guide 1.70.3. We also conclude that the comprehensive testing and surveillance program, as established by the proposed Technical Specification changes, as modified by the staff and concurred in by the licensee, provides additional assurance that leakage from radioactive material sources will not exceed allowable limits.

We evaluated the amount of reactor fuel which can be received, used, and possessed by the licensee under provisions of the proposed license amendments by assuming that: (1) the new fuel storage area is filled with unused fuel at equilibrium concentration, (2) the reactor is filled with unused fuel at equilibrium concentration, and (3) the spent fuel storage pit is filled with reactor fuel which has been used to equilibrium burnup. We concluded by this evaluation that the provisions of the proposed license amendments will not significantly alter the amount of reactor fuel which can be received, used, and possessed by the licensee. The proposed amendments do not authorize the licensee to receive, use, or possess fuel significantly different from that currently described in the FFDSAR.

We have determined that the proposed license amendment, as supported by the proposed Technical Specification changes and the FFDSAR revisions, are acceptable in that they:

- a. Comply with the guidance and intent of our letter of December 16, 1974.
- b. Assure that the amount and type of reactor fuel which can be received, used, and possessed is limited by the onsite fuel storage capacity and the requirements for reactor operation which have been approved previously by the NRC staff and which are described in the FFDSAR as of this date.

- c. Provide reasonable assurance that byproduct, source, and special nuclear material will be stored, used, and accounted for in a manner which meets the applicable radiation protection provisions of 10 CFR Parts 20, 30, 40 and 70.

Furthermore, we have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Summary

The licensee's radiation protection program, as supplemented by the FFDSAR revisions and the proposed Technical Specifications additions, has been evaluated. We have concluded that the incorporation of flexible yet controlled licensing provisions for the receipt, possession, and use of byproduct, source, and special nuclear material into the Facility Operating Licenses for Point Beach Nuclear Plant Units 1 and 2 is acceptable.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 17, 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-266 AND 50-301

WISCONSIN ELECTRIC POWER COMPANY
WISCONSIN MICHIGAN POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 15 and 20 to Facility Operating Licenses Nos. DPR-24 and DPR-27 issued to Wisconsin Electric Power Company and Wisconsin Michigan Power Company, which revises certain license provisions and the Technical Specifications for operation of the Point Beach Nuclear Plant Units Nos. 1 and 2, located in the town of Two Creeks, Manitowoc County, Wisconsin. The amendment is effective as of its date of issuance.

The amendment consists of a license amendment and Technical Specifications change relating to the receipt, possession, and use of byproduct, source, and special nuclear material and incorporates surveillance requirements for leakage testing of sealed sources in the Technical Specifications.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to

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DATE >						

10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated September 8, 1975 and Supplement dated January 30, 1976, (2) Amendment No. 15 to License No. DPR-24, (3) Amendment No. 20 to License No. DPR-27 and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street N.W., Washington, D.C. and at the Stevens Point Library, University of Wisconsin, Stevens Point, Wisconsin 54481.

A copy of (2), (3) and (4) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 17 day of March 1976.

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

SEE PREVIOUS YELLOW FOR CONCURRENCE

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SURNAME >	CParrish <i>CP</i>	JWetmore <i>acr</i>	<i>Ketchum</i>	GLear		
DATE >	3/ 4 /76	3/ A /76	3/ 11 /76	3/ 12 /76		

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-266 AND 50-301

WISCONSIN ELECTRIC POWER COMPANY
WISCONSIN MICHIGAN POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 15 and 20 to Facility Operating Licenses Nos. DPR-24 and DPR-27 issued to Wisconsin Electric Power Company and Wisconsin Michigan Power Company, which revised Technical Specifications for operation of the Point Beach Nuclear Plant Units Nos. 1 and 2, located in the town of Two Creeks, Manitowoc County, Wisconsin.

The amendment consists of a license amendment and Technical Specifications change relating to the receipt, possession, and use of byproduct, source, and special nuclear material.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or

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environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated September 8, 1975 and Supplement dated January 30, 1976, (2) Amendment No. 15 to License No. DPR-24, (3) Amendment No. 20 to License No. DPR-27 and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street N.W., Washington, D.C. and at the Stevens Point Library, University of Wisconsin, Stevens Point, Wisconsin 54481.

A copy of items (2) and (4) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this day of

FOR THE NUCLEAR REGULATORY COMMISSION

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

OFFICE >	ORB#3	ORB#3 <i>WR</i>	OELD	ORB#3		
SURNAME >	CParrish <i>CP</i>	JWetmore <i>acr</i>		GLear		
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