Docket Nos. 50-266 and 50-301.

> Wisconsin Michigan and Wisconsin Electric Power Company ATTN: Mr. Sol Burstein 231 West Michigan Street Milwaukee, Wisconsin 53201

Gentlemen:

Change No. 7 License No. DFR-24 Change No. 10 License No. DFR-27

By letter dated April 1, 1974, you proposed a change to the Technical Specifications of Facility Operating Licenses Nos. DPR-24 and DPR-27 to allow inspection of fuel in the spent fuel storage rack location and to redefine requirements for the Duty and Call Superintendent. As discussed with your staff, we have modified your proposed change to meet Regulatory requirements.

Based on our evaluation of the proposed change, as modified, we conclude that it does not involve a significant hazards consideration and that there is reasonable assurance that the health and safety of the public will not be endangered by operation of the reactor in the manner proposed.

Accordingly, pursuant to Section 50.59 of 10 CFR Part 50, the Technical Specifications of Licenses Nos. DPR-24 and DPR-27 are changed as shown in Attachment A.

Sincerely,

161

Karl R. Goller Assistant Director for Operating Reactors Directorate of Licensing

Enclosures:

- 1. Attachment A Change No. 7 to License No. DPR-24 and Change No. 10 to License No. DPR-27
- 2. Safety Evaluation

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cc w/enclosures: Mr. Bruce W. Churchill, Esquire Shaw, Pittman, Potts, Trowbridge and Madden 910 - 17th Street, N. W. Washington, D. C. 20006

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ATTACHMENT A

CHANGE NO. 7 (DFR-24) AND CHANGE NO. 10 (DFR-27)

TO THE TECHNICAL SPECIFICATIONS

POINT BEACH NUCLEAR PLANTS UNITS NO. 1 AND NO. 2

WISCONSIN MICHIGAN AND WISCONSIN ELECTRIC POWER COMPANY

DOCKET NOS. 50-266 AND 50-301

Change Technical Specification 15.5.4.2 and 3 to read:

- "2. The new and spent fuel storage racks are designed so that it is impossible to store assemblies in other than the prescribed storage locations. The fuel is stored vertically in an array with sufficient center-to-center distance between assemblies to assure $K_{\mbox{eff}} \leq 0.90$ even if unborated water were used to fill the pit. Two inspection locations, one in the north spent fuel pit (at Q-3) and one in the south spent fuel pit (at Q-26), allow rotation of a fuel assembly for visual inspection, but shall not be used for storage.
- 3. The spent fuel storage pit shall be filled with borated water at a concentration of at least 1800 ppm boron whenever there is fuel in the pit."

Change Technical Specification 15.6.1.C.1 to read:

"1. Duty and Call Superintendent

To assist and counsel the Shift Supervisor in the event of abnormal occurrences, à Duty and Call Superintendent Group has been established. The Duty and Call Superintendent Group shall consist of any qualified person designated by the Manager - Nuclear Power Division. In the event of an abnormal occurrence, the Shift Supervisor shall communicate with at least one Duty and Call Superintendent before taking other than the immediate on-the-spot action required. One Duty and Call Superintendent will be assigned to be "on call" at all times. The Duty and Call Superintendent provides continuously available counsel, call out backup, and review to the Shift Supervisor.

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UNITED STATES ATOMIC ENERGY COMMISSION

SAFETY EVALUATION BY THE DIRECTORATE OF LICENSING

WISCONSIN MICHIGAN AND WISCONSIN ELECTRIC POWER COMPANY

CHANGE NO. 7 (DPR-24) AND CHANGE NO. 10 (DPR-27)

TO THE TECHNICAL SPECIFICATIONS

BOCKET NOS. 50-266 AND 50-301

By letter dated April 1, 1974, Wisconsin Michigan and Wisconsin Electric Power Company proposed a change to the Technical Specifications of Facility Operating Licenses Nos. DPR-24 and DPR-27 to allow inspection of fuel in a spent fuel storage rack location and to redefine requirements for the Duty and Call Superintendent.

Our concern relative to the proposed change in the use of the fuel storage racks is to assure that the change will not compromise the safety of fuel storage with respect to criticality.

The modification to the storage racks involves removal of the fuel assembly support grid in one assembly location in each storage rack. This allows an assembly to be rotated, raised, and lowered while visually inspected by periscope and/or recorded on video tape. Storage of fuel assemblies in the inspection locations is prohibited (Technical Specifications).

The proposed Technical Specifications have been modified to clearly include the requirement that the spent fuel pit be filled with borated water with a concentration of at least 1800 ppm boron whenever there is fuel in the pit. A concentration of 1800 ppm boron is sufficient to assure that criticality can not be caused in the storage pit by any position of an assembly in either inspection port. Required boron concentration is the same concentration that is required in the primary coolant system during refueling (Specification 15.3.8-è) which has been determined to be sufficient to maintain the reactor in a subcritical condition even with all rods withdrawn. The spent fuel pit has a fixed spacing for assemblies in storage such as to assure a $K_{\mbox{eff}}$ of ≤ 0.90 with pit filled with unborated water. We have therefore determined that the fuel in the storage pit with borated water will remain adequately subcritical during fuel inspection because the stored fuel is significantly more subcritical than the reactor core for both borated and unborated conditions.

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The proposed change in requirements for the Duty and Call Superintendent Group will allow the Manager - Nuclear Power Division to designate members of the group instead of designating by title the management personnel authorized for the group. Qualifications for the Duty and Call Superintendent are documented in their "Quality Assurance Manual Volume 1, Section 3.13."

We have reviewed the proposed change in requirements for members of the Duty and Call Superintendent Group and have determined that requirements as described in the Quality Assurance Manual are adequate. A senior operator's license or the equivalent experience and training of a senior operator is required for members of the Duty and Call Superintendent Croup.

Based on our evaluation of the proposed change, we have concluded that it does not involve a significant hazards consideration and that there is reasonable assurance that the health and safety of the public will not be endangered by operations in the manner proposed.



Peter B. Erickson Operating Reactors Branch #1 Directorate of Licensing



Robert A. Purple, Chief Operating Reactors Branch #1 Directorate of Licensing

Date: APR 1 9 1974

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