

DEC 8 6 1974

Docket No. 50-301

Wisconsin Electric Power Company
Wisconsin Michigan Power Company
ATTN: Mr. Sol Burstein
Senior Vice President
231 West Michigan Street
Milwaukee, Wisconsin 53201

Gentlemen:

The Commission has issued the enclosed Amendment No. 7 to Facility License No. DPR-27 for the Point Beach Nuclear Plant Unit No. 2. This Amendment includes Change No. 13 to the Technical Specifications, and is in response to your request dated August 30, 1974 as revised by letter dated October 7, 1974.

The amendment permits the Point Beach Nuclear Plant Unit No. 2 to operate core cycle 2 for 20,000 effective full power hours in Region 2.

Copies of the Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

Original Signed

George Lear, Chief
Operating Reactors Branch #3
Directorate of Licensing

Enclosures:

1. Amendment No. 7
2. Safety Evaluation
3. Federal Register Notice

cc: See next page

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Mr. Sol Brustein

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DEC 26 1974

cc: w/enclosure

Bruce W. Churchill, Esquire
Shaw, Pittman, Potts & Trowbridge
910 - 17th Street, N. W.
Washington, D. C. 20006

Mr. Arthur M. Fish
Document Department
University of Wisconsin - Stevens
Point Library
Stevens Point, Wisconsin 54481

Mr. William F. Eich, Chairman
Public Service Commission of
Wisconsin
Hill Farms State Office Building
Madison, Wisconsin 53702

Mr. Gary Williams
Federal Activities Branch
Environmental Protection Agency
Region V Office
1 N. Wacker Drive, Room 822
Chicago, Illinois 60606

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WISCONSIN ELECTRIC POWER COMPANY
AND
WISCONSIN MICHIGAN POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 7
License No. DPR-27

1. The Atomic Energy Commission (the Commission) having found that:
 - A. The application for amendment by Wisconsin Electric Power Company and Wisconsin Michigan Power Company (the licensees) dated August 30, 1974, as amended on October 7, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. No request for a hearing or petition for leave to intervene was filed following notice of the proposed action.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B of Facility License No. DPR-27 is hereby amended to read as follows:

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"B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 13."

3. This license amendment is effective as of the date of its issuance.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed

A. Giambusso, Deputy Director
for Reactor Projects
Directorate of Licensing

Attachment:
Change No. 13 to the
Technical Specifications

Date of Issuance: DEC 26 1974

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ATTACHMENT TO LICENSE AMENDMENT NO. 7

CHANGE NO. 13 TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-27

The Technical Specifications are changed as follows:

Replace Page ⁵ 15.2.1-1 and 15.2.1-3 with the attached pages. ✓

Replace Figure 15.3.10-1 with the attached figure.

Delete Figure 15.3.10-3.

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15.2.0 SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

15.2.1 SAFETY LIMIT, REACTOR CORE

Applicability:

Applies to the limiting combinations of thermal power, reactor coolant system pressure, and coolant temperature during operation.

Objective:

To maintain the integrity of the fuel cladding.

Specification:

1. The combination of thermal power level, coolant pressure, and coolant temperature shall not exceed the limits shown in Figure 15.2.1-1. The safety limit is exceeded if the point defined by the combination of reactor coolant system average temperature and power level is at any time above the appropriate pressure line.
2. The cumulative fuel residence time for fuel region 2 of core cycle 2 for Unit No. 2 shall be limited to 20,000 effective full power hours (EFPH) under design operating conditions, with a primary system pressure of 2,000 psia.

This combination of hot channel factors is higher than that calculated at full power for the range from all control rods fully withdrawn to maximum allowable control rod insertion. The control rod insertion limits are covered by Specification 15.3.10-1. Somewhat worse hot channel factors could occur at lower power levels because additional control rods are in the core. However, the control rod insertion limits dictated by Figure 15.3.10-1 insure that the DNB ratio is always greater at part power than at full power. Additional peaking factors to account for local peaking due to fuel rod axial gaps and reduction in fuel pellet stack length have been included in the calculation of the curves shown in Figure 15.2.1-1.

Figure 15.2.1-1 also includes an allowance for an increase in the enthalpy rise hot channel factor at reduced power based on the expression:

$$F_{\Delta H}^N = 1.58 [1 + 0.2 (1-P)] \text{ where } P \text{ is the fraction of rated power.}$$

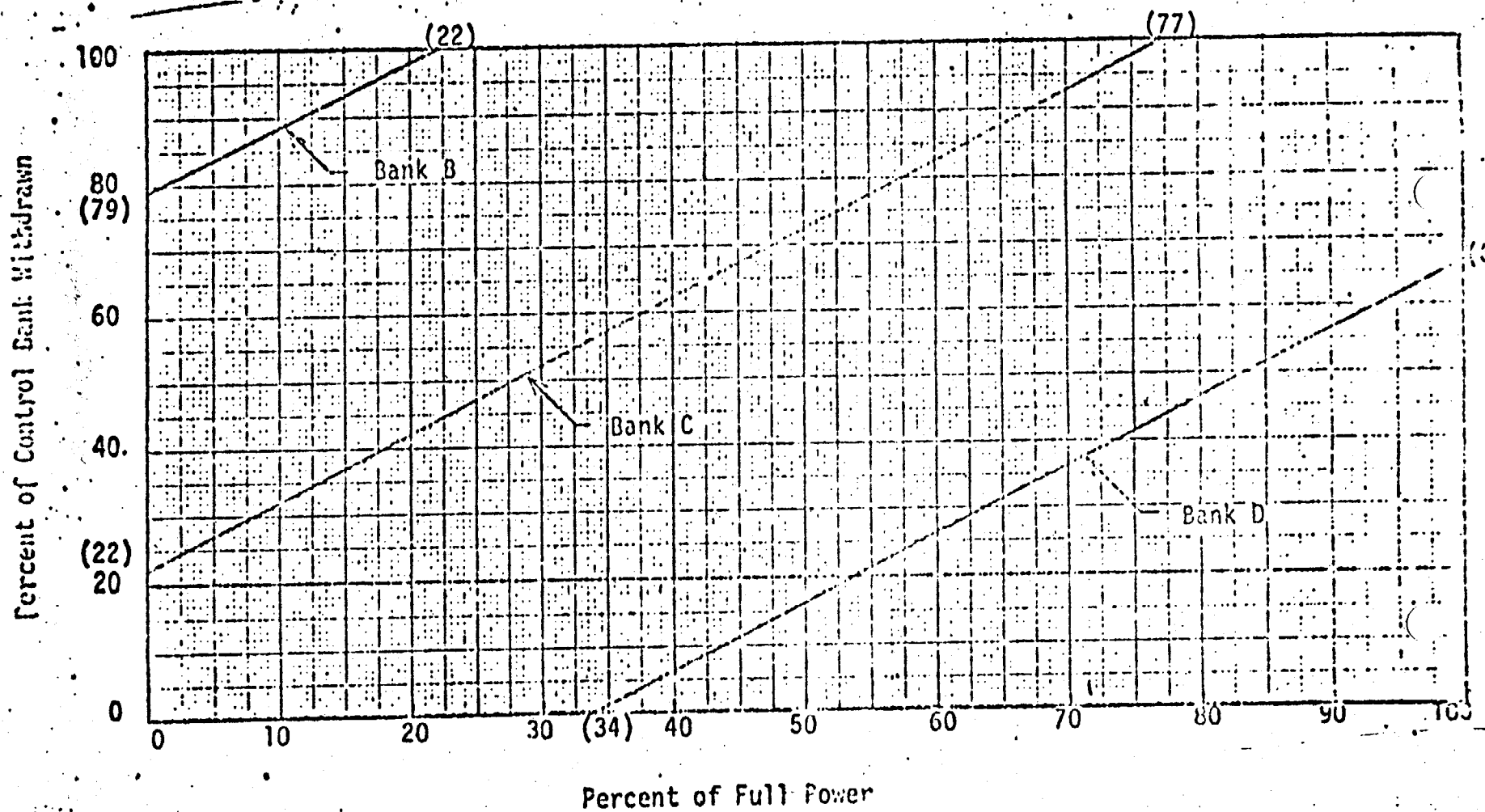
The hot channel factors are also sufficiently large to account for the degree of malpositioning of part-length rods that is allowed before the reactor trip set points are reduced and rod withdrawal block and load runback may be required. Rod withdrawal block and load runback occur ~~before~~ reactor trip setpoints are reached.

The Reactor Control and Protective System is designed to prevent any anticipated combination of transient conditions that would result in a DNB ratio of less than 1.30.

The cumulative fuel residence time for fuel region 2 of core cycle 2 is limited to 20,000 EFPH to assure no fuel clad flattening without prior review by the Regulatory staff. The residence time of 20,000 EFPH is based on predicted minimum time to clad flattening for an operating pressure of 2000 psi. Beyond the residence time of 20,000 EFPH for the region 2 fuel of core Cycle 2, an assumption of clad flattening is presently required. Prior to attaining 20,000 EFPH, the licensee will provide the additional analysis required for operation beyond 20,000 EFPH.

FIGURE 15.3.10-1

Full Length Rod Insertion Limits
Point Beach Unit 2 - Cycle 2



UNITED STATES ATOMIC ENERGY COMMISSION

DOCKET NO. 50-301

WISCONSIN ELECTRIC POWER COMPANY

AND

WISCONSIN MICHIGAN POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

No request for a hearing or petition for leave to intervene having been filed following publication of the notice of proposed action in the FEDERAL REGISTER on November 13, 1974 (39 FR 40062), notice is hereby given that the Atomic Energy Commission (the Commission) has issued Amendment No. 7 to Facility Operating License No. DPR-27, issued to the Wisconsin Electric Power Company and the Wisconsin Michigan Power Company, revising Technical Specifications for operation of the Point Beach Nuclear Power Plant Unit No. 2 located in the Town of Two Creeks, Manitowoc County, Wisconsin. The amendment is effective as of the date of issuance.

The amendment permits operation of Point Beach Nuclear Plant Unit No. 2, in Cycle 2 for 20,000 Effective Full Power Hours, following refueling using pre-pressurized fuel that is of a higher pressure than the present fuel. Amendment No. 6 to Facility Operating License No. DPR-27, issued on December 13, 1974, authorized the Wisconsin Electric Power Company and the Wisconsin Michigan Power Company to operate Point Beach Nuclear Power Plant Unit No. 2 in Cycle 2 for 14,000 Effective Full Power Hours.

The Commission has found that the application for amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The

Commission has made appropriate findings as required by the Act and the

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Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

For further details with respect to this license amendment see (1) Amendment No. 7 to Facility License No. DPR-27 with any attachment. (2) the related Safety Evaluation, and (3) additional information submitted by the licensee in a letter dated October 7, 1974, which are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the University of Wisconsin - Stevens Point Library, Stevens Point, Wisconsin. A copy of items (1) and (2) may be obtained upon request addressed to the U. S. Atomic Energy Commission, Washington, D. C. 20545, Attention: Deputy Director for Reactor Projects, Directorate of Licensing - Regulation.

Dated at Bethesda, Maryland, this 26th day of December, 1974.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed

George Lear, Chief
Operating Reactors Branch #3
Directorate of Licensing

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