

March 8, 1996

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Mr. Donald C. Shelton
 Acting Vice President Nuclear - Perry
 Centerior Service Company
 P. O. Box 97, A200
 Perry, OH 44081

SUBJECT: CORRECTION LETTER FOR AMENDMENT NO. 79 TO FACILITY OPERATING LICENSE
 NO. NPF-58 - PERRY NUCLEAR POWER PLANT, UNIT NO. 1 (TAC NO. M88283)

Dear Mr. Shelton:

Portions of Amendment No. 79 to Facility Operating License No. NPF-58 for the Perry Nuclear Power Plant, Unit No. 1, are being reissued to provide a page that was inadvertently deleted during reproduction of the document and to replace four Technical Specification (TS) pages to include changes from Amendment No. 75 as well as Amendment No. 79.

Amendment No. 79 addressed replacement of analog temperature instrumentation with digital equipment for the leak detection system based on your submittal which requested review of the unreviewed safety question and revision of the TS.

Sincerely,

Original Signed By:

Jon B. Hopkins, Sr. Project Manager
 Project Directorate III-3
 Division of Reactor Projects III/IV
 Office of Nuclear Reactor Regulation

Docket No. 50-440

- Enclosures: 1. Page 1 of Facility Operating License No. NPF-58
 2. Revised TS pages

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DOCUMENT NAME:G:\PERRY\PER88283.LTR

*See Previous Concurrence

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 8, 1996

Mr. Donald C. Shelton
Acting Vice President Nuclear - Perry
Centerior Service Company
P. O. Box 97, A200
Perry, OH 44081

SUBJECT: CORRECTION LETTER FOR AMENDMENT NO. 79 TO FACILITY OPERATING LICENSE
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Dear Mr. Shelton:

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Amendment No. 79 addressed replacement of analog temperature instrumentation with digital equipment for the leak detection system based on your submittal which requested review of the unreviewed safety question and revision of the TS.

Sincerely,

A handwritten signature in cursive script, appearing to read "Jon B. Hopkins".

Jon B. Hopkins, Sr. Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-440

Enclosures: 1. Page 1 of Facility Operating
License No. NPF-58
2. Revised TS pages

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY, ET AL.

DOCKET NO. 50-440

PERRY NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.
License No. NPF-58

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by The Cleveland Electric Illuminating Company, Centerior Service Company, Duquesne Light Company, Ohio Edison Company, Pennsylvania Power Company, and Toledo Edison Company (the licensees) dated November 22, 1993, supplemented May 5 and December 20, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-58 is hereby amended to read as follows:

ATTACHMENT TO LICENSE AMENDMENT NO. 79

FACILITY OPERATING LICENSE NO. NPF-58

DOCKET NO. 50-440

Replace the following pages of the Appendix "A" Technical Specifications including the issued but not yet implemented Improved Technical Specifications (ITS) with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3/4 3-23
3/4 3-24
3/4 3-25
3/4 3-26

Insert

3/4 3-23
3/4 3-24
3/4 3-25
3/4 3-26

TABLE 4.3.2.1-1

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
1. <u>PRIMARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level - Low, Level 2	S	Q	R ^{(b)(c)(d)}	1, 2, 3 and #
b. Drywell Pressure - High ##	S	Q	R ^(b)	1, 2, 3
c. Containment and Drywell Purge Exhaust Plenum Radiation - High	S	Q	R ^(d)	1, 2, 3 and *
d. Reactor Vessel Water Level - Low, Level 1	S	Q	R ^{(b)(c)(d)}	1, 2, 3 and #
e. Manual Initiation	NA	R ^(d)	NA	1, 2, 3 and *
2. <u>MAIN STEAM LINE ISOLATION</u>				
a. Reactor Vessel Water Level - Low, Level 1	S	Q	R ^{(b)(c)(d)}	1, 2, 3
b. Main Steam Line Radiation - High	S	Q	R ^{(c)(d)}	***
c. Main Steam Line Pressure - Low	S	Q	R ^(b)	1
d. Main Steam Line Flow - High	S	Q	R ^(b)	1, 2, 3
e. Condenser Vacuum - Low	S	Q	R ^(b)	1, 2**, 3**
f. Main Steam Line Tunnel Temperature - High				
1. Division 1 and 2	S	SA	R	1, 2, 3
2. Division 3 and 4	S	Q	R	1, 2, 3
g. Main Steam Line Tunnel Δ Temperature - High				
1. Division 1 and 2	S	SA	R	1, 2, 3
2. Division 3 and 4	S	Q	R	1, 2, 3
h. Turbine Building Main Steam Line Temperature - High	S	Q	R	1, 2, 3
i. Manual Initiation	NA	R	NA	1, 2, 3

TABLE 4.3.2.1-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
<u>3. SECONDARY CONTAINMENT ISOLATION</u>				
a. Reactor Vessel Water Level - Low, Level 2	S	Q	R ^{(b)(c)(d)}	1, 2, 3 and #
b. Drywell Pressure - High ##	S	Q	R ^(b)	1, 2, 3
c. Manual Initiation	NA	R	NA	1, 2, 3 and *
<u>4. REACTOR WATER CLEANUP SYSTEM ISOLATION</u>				
a. Δ Flow - High	S	Q	R	1, 2, 3
b. Δ Flow Timer	NA	Q	R	1, 2, 3
c. Equipment Area Temperature - High	S	SA	R	1, 2, 3
d. Equipment Area Ventilation Δ Temperature - High	S	SA	R	1, 2, 3
e. Reactor Vessel Water Level - Low, Level 2	S	Q	R ^{(b)(c)(d)}	1, 2, 3
f. Main Steam Line Tunnel Ambient Temperature - High	S	SA	R	1, 2, 3
g. Main Steam Line Tunnel Δ Temperature - High	S	SA	R	1, 2, 3
h. SLCS Initiation	NA	Q ^(a)	NA	1, 2, 3
i. Manual Initiation	NA	R	NA	1, 2, 3

3/4 3-24

Amendment No. 67, 75, 79

TABLE 4.3.2.1-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
5. <u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u>				
a. RCIC Steam Line Flow - High	S	Q	R ^(b)	1, 2, 3
b. RCIC Steam Supply Pressure - Low	S	Q	R ^(b)	1, 2, 3
c. RCIC Turbine Exhaust Diaphragm Pressure - High	S	Q	R ^(b)	1, 2, 3
d. RCIC Equipment Room Ambient Temperature - High	S	SA	R	1, 2, 3
e. Deleted				
f. Main Steam Line Tunnel Ambient Temperature - High	S	SA	R	1, 2, 3
g. Main Steam Line Tunnel Δ Temperature - High	S	SA	R	1, 2, 3
h. Main Steam Line Tunnel Temperature Timer	NA	SA	R	1, 2, 3
i. RHR Equipment Room Ambient Temperature - High	S	SA	R	1, 2, 3
j. RHR Equipment Room Δ Temperature - High	S	SA	R	1, 2, 3
k. RCIC Steam Line Flow High Timer	NA	Q	R	1, 2, 3
l. Drywell Pressure - High	S	Q	R ^(b)	1, 2, 3
m. Manual Initiation	NA	R ^(d)	NA	1, 2, 3

TABLE 4.3.2.1-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>TRIP FUNCTION</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>CHANNEL CALIBRATION</u>	<u>OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED</u>
6. <u>RHR SYSTEM ISOLATION</u>				
a. RHR Equipment Area Ambient Temperature - High	S	SA	R	1, 2, 3
b. RHR Equipment Area Δ Temperature - High	S	SA	R	1, 2, 3
c. RHR/RCIC Steam Line Flow - High	S	Q	R ^(b)	1, 2, 3
d. Reactor Vessel Water Level - Low, Level 3 ##	S	Q	R ^{(b)(c)(d)}	1, 2, 3
e. Reactor Vessel (RHR Cut-in Permissive) Pressure - High	S	Q	R ^{(b)(c)(d)}	1, 2, 3
f. Drywell Pressure - High ##	S	Q	R ^{(b)(d)}	1, 2, 3
g. Manual Initiation	NA	R ^(d)	NA	1, 2, 3

* When handling irradiated fuel in the primary containment and during CORE ALTERATIONS and operations with a potential for draining the reactor vessel.

** When any turbine stop valve is greater than 90% open and/or the key locked bypass switch is in the normal position.

*** OPERATIONAL CONDITION 1 or 2 when the mechanical vacuum pump lines are not isolated.

During CORE ALTERATION and operations with a potential for draining the reactor vessel.

(a) Each train or logic channel shall be tested at least every other 92 days.

(b) Calibrate trip unit setpoint at least once per 92 days.

These Trip Functions (1b, 3b, 6d, and 6f) utilize instruments which are common to RPS instrumentation.

(c) CHANNEL CALIBRATION may be extended to be performed during the fifth refueling outage.

(d) LOGIC SYSTEM FUNCTIONAL TEST may be extended to be performed during the fifth refueling outage.

Mr. Donald C. Shelton
Centerior Service Company

cc:

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