

March 18, 1996

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Mr. Donald C. Shelton  
 Acting Vice President Nuclear - Perry  
 Centerior Service Company  
 P. O. Box 97, A200  
 Perry, OH 44081

SUBJECT: AMENDMENT NO. 83 TO FACILITY OPERATING LICENSE NO. NPF-58 - PERRY  
 NUCLEAR POWER PLANT, UNIT NO. 1 (TAC NO. M94784)

Dear Mr. Shelton:

The Commission has issued the enclosed Amendment No. 83 to Facility Operating License No. NPF-58 for the Perry Nuclear Power Plant, Unit No. 1. This amendment revises the Technical Specifications in response to your application dated February 17, 1996.

This amendment allows one main steam line's leakage rate to be as high as 35 standard cubic feet per hour (scfh) as long as the total leakage through all four main steam lines does not exceed 100 scfh until the end of Operating Cycle 6.

A copy of the Safety Evaluation is also enclosed. Notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original signed by:

Jon B. Hopkins, Sr. Project Manager  
 Project Directorate III-3  
 Division of Reactor Projects III/IV  
 Office of Nuclear Reactor Regulation

Docket No. 50-440

- Enclosures: 1. Amendment No. 83 to License No. NPF-58  
 2. Safety Evaluation

cc w/encls: See next page

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NAME	DFoster-Curseen		JHopkins\sw		EHOLLER		GMarcus		CBerlinger	
DATE	03/11/96		03/11/96		03/13/96		03/15/96		03/11/96	

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

March 18, 1996

Mr. Donald C. Shelton  
Acting Vice President Nuclear - Perry  
Centerior Service Company  
P. O. Box 97, A200  
Perry, OH 44081

SUBJECT: AMENDMENT NO. 83 TO FACILITY OPERATING LICENSE NO. NPF-58 - PERRY  
NUCLEAR POWER PLANT, UNIT NO. 1 (TAC NO. M94784)

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Sincerely,

A handwritten signature in black ink, appearing to read "Jon B. Hopkins, Sr.", written over a horizontal line.

Jon B. Hopkins, Sr. Project Manager  
Project Directorate III-3  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-440

Enclosures: 1. Amendment No. 83 to  
License No. NPF-58  
2. Safety Evaluation

cc w/encls: See next page

Mr. Donald C. Shelton  
Centerior Service Company

Perry Nuclear Power Plant  
Unit Nos. 1 and 2

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U. S. Nuclear Regulatory Commission  
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DERR--Compliance Unit  
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Public Utilities Commission  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY, ET AL.

DOCKET NO. 50-440

PERRY NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 83  
License No. NPF-58

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by The Cleveland Electric Illuminating Company (CEICO), Centerior Service Company, Duquesne Light Company, Ohio Edison Company, OES Nuclear, Inc., Pennsylvania Power Company, and Toledo Edison Company (the licensees) dated February 17, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-58 is hereby amended to read as follows:

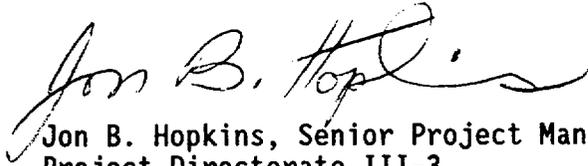
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P PDR

**(2) Technical Specifications**

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 83 are hereby incorporated into this license. The Cleveland Electric Illuminating Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented not later than 30 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Jon B. Hopkins, Senior Project Manager  
Project Directorate III-3  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of issuance: March 18, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 83

FACILITY OPERATING LICENSE NO. NPF-58

DOCKET NO. 50-440

Replace the following pages of the Appendix "A" Technical Specifications including the issued but not yet implemented Improved Technical Specifications (ITS) with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3/4 6-3

3/4 6-4

ITS 3.6-18

Insert

3/4 6-3

3/4 6-4

ITS 3.6-18

## CONTAINMENT SYSTEMS

### PRIMARY CONTAINMENT LEAKAGE

#### LIMITING CONDITION FOR OPERATION

---

3.6.1.2 Primary containment leakage rates shall be limited to:

- a. An overall integrated leakage rate, except for the main steam line isolation valves#, of less than or equal to  $0.75 L_a$ , where  $L_a$  is 0.20 percent by weight of the primary containment air per 24 hours at  $P_a$ .
- b. A combined leakage rate of less than or equal to  $0.60 L_a$  for all penetrations and all valves, except for main steam line isolation valves# and valves which are hydrostatically leak tested, subject to Type B and C tests when pressurized to  $P_a$ .
- c. Less than or equal to 25 scf per hour\* for any one main steam line through the isolation valves when tested at  $P_a$ .
- d. A combined leakage rate of less than or equal to  $0.0504 L_a$  for all penetrations that are secondary containment bypass leakage paths when pressurized to the required test pressure.
- e. A combined leakage rate of less than or equal to 1 gpm times the total number of containment isolation valves in hydrostatically tested lines which penetrate the primary containment, when tested at greater than or equal to  $1.10 P_a$ .

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, AND 3, with the reactor coolant system temperature greater than 200 °F.

#### ACTION:

- With:
- a. The measured overall integrated primary containment leakage rate, except for the main steam line isolation valves#, exceeding  $0.75 L_a$ , or
  - b. The measured combined leakage rate for all penetrations and all valves except for main steam line isolation valves# and valves which are hydrostatically leak tested, subject to Type B and C tests exceeding  $0.60 L_a$ , or
  - c. The measured leakage rate exceeding 25 scf per hour\* for any one main steam line through the isolation valves, or

---

# Exemption to Appendix J of 10 CFR 50.

\* One main steam line may be less than or equal to 35 scf per hour as long as total leakage rate through all four main steam lines is less than or equal to 100 scf per hour. This will remain in effect from the effective date of this amendment until the completion of Operating Cycle 6.

## CONTAINMENT SYSTEMS

### PRIMARY CONTAINMENT LEAKAGE

#### LIMITING CONDITION FOR OPERATION (Continued)

- d. The combined leakage rate for all penetrations that are secondary containment bypass leakage paths exceeding  $0.0504 L_a$ , or
- e. The measured combined leakage rate for all containment isolation valves in hydrostatically tested lines which penetrate the primary containment exceeding 1 gpm times the total number of such valves.

Restore the leakage rate to less than or equal to the above limit(s) within 1 hour or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

#### SURVEILLANCE REQUIREMENTS

4.6.1.2 The primary containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR Part 50, except that the provisions of Bechtel Topical Report BN-TOP-1 may be used for Type A tests having a duration less than 24 hours:

- a. Three Type A Overall Integrated Containment Leakage Rate tests shall be conducted at  $40 \pm 10$  month intervals during shutdown at  $P_a$  during each 10-year service period.
- b. If any periodic Type A test fails to meet  $0.75 L_a$ , the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet  $0.75 L_a$ , a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet  $0.75 L_a$ , at which time the above test schedule may be resumed.
- c. The accuracy of each Type A test shall be verified by a supplemental test which:

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.6.1.3.9 -----NOTE----- Only required to be met in MODES 1, 2, and 3. -----</p> <p>Verify the combined leakage rate for all secondary containment bypass leakage paths is <math>\leq 0.0504 L_a</math> when pressurized to <math>\geq P_a</math>.</p>	<p>-----NOTE----- SR 3.0.2 is not applicable -----</p> <p>In accordance with 10 CFR 50, Appendix J, as modified by approved exemptions</p>
<p>SR 3.6.1.3.10 -----NOTE----- Only required to be met in MODES 1, 2, and 3. -----</p> <p>Verify leakage rate through each main steam line is <math>\leq 25</math> scfh when tested at <math>\geq P_a</math>. Until the end of Operating Cycle 6, the leakage rate through one main steam line is limited to <math>\leq 35</math> scfh when tested at <math>\geq P_a</math>, as long as the total leakage rate through all four main steam lines is <math>\leq 100</math> scfh.</p>	<p>-----NOTE----- SR 3.0.2 is not applicable -----</p> <p>In accordance with 10 CFR 50, Appendix J, as modified by approved exemptions</p>

(continued)



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 83 TO FACILITY OPERATING LICENSE NO. NPF-58

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY, ET AL.

PERRY NUCLEAR POWER PLANT, UNIT NO. 1

DOCKET NO. 50-440

1.0 INTRODUCTION

By letter dated February 17, 1996, The Cleveland Electric Illuminating Company (CEI or the licensee) requested changes to the Technical Specifications (TSs) for the Perry Nuclear Power Plant, Unit 1 (PNPP). The proposed amendment would allow one main steam line's leakage rate to be as high as 35 standard cubic feet per hour (scfh) as long as the total leakage rate through all four main steam lines does not exceed 100 scfh until the end of Operating Cycle 6.

2.0 EVALUATION

The current TSs require that the primary containment leakage rates shall be limited to less than or equal to 25 standard cubic feet per hour (scfh) for any one of the four main steam lines through their isolation valves. During the current refueling outage, it was determined that the "C" main steam line had a leakage rate of 28.1 scfh. However, the combined leakage through all four main steam line isolation valves is less than 75 scfh. Therefore, the total leakage is less than the value of 100 scfh which is the effective combined limit when 25 scfh is considered for all four main steam lines.

The NRC staff has reviewed the licensee's request and finds that the effective combined limit of 100 scfh for all four main steam lines is maintained by the proposed amendment. Therefore, the staff finds the proposed amendment to be acceptable.

3.0 EXIGENT CIRCUMSTANCES

The need for a change to main steam line leakage rate limits became apparent on February 11, 1996, during surveillance testing of the main steam lines. Repair of the responsible valve to reduce the leakage below 25 scfh would provide no significant benefit to safety, while involving an estimated 2 person-REM of radiation exposure and an estimated 2200 person-hours of work. Plant startup from the current refueling outage is scheduled for March 27, 1996. The license amendment is needed prior to that date to avoid delaying plant startup, therefore, the NRC staff finds that exigent circumstances exist in that the Commission and licensee must act quickly and that time does not allow publication of a notice allowing 30 days for prior public comment.

The NRC staff has reviewed the circumstances surrounding the amendment request, and finds that the circumstances could not have been avoided and the licensee made a timely request for amendment. Therefore, the staff finds that the license amendment may be issued in an exigent manner.

#### 4.0 BASIS FOR FINAL NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration. The NRC staff has reviewed the licensee's analysis against the standards of 10 CFR 50.92(c). The staff's review is presented below.

The amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated because the change does not increase the probability of a steam line rupture, and the effective combined leakage limit for all four main steam lines is not changed.

The amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated because no physical alterations to the plant are being made.

The amendment does not involve a significant reduction in a margin of safety because the effective combined leakage limit for all four main steam lines is not changed.

Based on this review, the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff has determined that the amendment request involves no significant hazards consideration.

#### 5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Ohio State official was notified of the proposed issuance of the amendment. The State official had no comments.

#### 6.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or a change to a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluent that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission made a final no significant hazards consideration determination with respect to this amendment. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

**7.0 CONCLUSION**

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

**Principal Contributor:** J. Hopkins

**Date:** March 18, 1995