

August 11, 1993

Docket No. 50-440

Mr. Robert A. Stratman
Vice President Nuclear - Perry
Centerior Severice Company
Post Office Box 97, S270
Perry, Ohio 44081

DISTRIBUTION

<u>Docket File</u>	GHill(2)
NRC & Local PDRs	Wanda/Almya
PD3-3 Reading	CGrimes
JRoe	ACRS(10)
JZwolinski	OPA
JHannon	OC/LFDCB
MRushbrook	Region III, DRP
RStransky	PD3-3 Gray
OGC-WF	DHagan
SMazumdar, HICB	

Dear Mr. Stratman:

SUBJECT: AMENDMENT NO. 50 TO FACILITY OPERATING LICENSE NO. NPF-58
(TAC NO. M82092)

The Commission has issued the enclosed Amendment No. 50 to Facility Operating License No. NPF-58 for the Perry Nuclear Power Plant, Unit No. 1. This amendment revises the Technical Specifications in response to your application dated October 30, 1991.

This amendment revises Technical Specification Tables 3.3.2-1 and 3.3.3-1 to clarify the logic for instrumentation used to isolate the High Pressure Core Spray (HPCS) test return valve and to initiate the HPCS system.

A copy of the Safety Evaluation is also enclosed. Notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original signed by Robert J. Stransky

Robert J. Stransky, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 50 to License No. NPF-58
2. Safety Evaluation

cc w/enclosures:
See next page

LA/PD3-3/DRPW
MRushbrook

PM/PD8-3/DRPW
Rstransky/b.j

PD/PD3-3/DRPW
JHannon *SBH*

OGC-WF1

9308270017 930811
PDR ADDCK 05000440
P PDR

WAC FILE CENTER COPY

Mr. Robert A. Stratman
Centerior Service Company

cc:

Jay E. Silberg, Esq.
Shaw, Pittman, Potts & Trowbridge
2300 N Street, N.W.
Washington, D.C. 20037

Mary E. O'Reilly
Centerior Energy Corporation
300 Madison Avenue
Toledo, Ohio 43652

Resident Inspector's Office
U.S. Nuclear Regulatory Commission
Parmlly at Center Road
Perry, Ohio 44081

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Lake County Prosecutor
Lake County Administration Bldg.
105 Main Street
Painesville, Ohio 44077

Ms. Sue Hiatt
OCRE Interim Representative
8275 Munson
Mentor, Ohio 44060

Terry J. Lodge, Esq.
618 N. Michigan Street, Suite 105
Toledo, Ohio 43624

Ashtabula County Prosecutor
25 West Jefferson Street
Jefferson, Ohio 44047

Mr. Kevin P. Donovan
Cleveland Electric
Illuminating Company
Perry Nuclear Power Plant
P. O. Box 97, E-210
Perry, Ohio 44081

Perry Nuclear Power Plant
Unit Nos. 1 and 2

Mr. James W. Harris, Director
Division of Power Generation
Ohio Department of Industrial Relations
P. O. Box 825
Columbus, Ohio 43216

The Honorable Lawrence Logan
Mayor, Village of Perry
4203 Harper Street
Perry, Ohio 44081

The Honorable Robert V. Orosz
Mayor, Village of North Perry
North Perry Village Hall
4778 Lockwood Road
North Perry Village, Ohio 44081

Attorney General
Department of Attorney General
30 East Broad Street
Columbus, Ohio 43216

Radiological Health Program
Ohio Department of Health
Post Office Box 118
Columbus, Ohio 43266-0118

Ohio Environmental Protection Agency
DERR--Compliance Unit
ATTN: Zack A. Clayton
P. O. Box 1049
Columbus, Ohio 43266-0149

Mr. Thomas Haas, Chairman
Perry Township Board of Trustees
3750 Center Rd., Box 65
Perry, Ohio 44081

State of Ohio
Public Utilities Commission
East Broad Street
Columbus, Ohio 43266-0573

David P. Igyarto, General Manager
Cleveland Electric Illuminating Company
Perry Nuclear Power Plant
Post Office Box 97, SB306
Perry, Ohio 44081

Mr. Robert A. Stratman
Centerior Service Company

Perry Nuclear Power Plant
Unit Nos. 1 and 2

cc:
James R. Williams, Chief of Staff
Ohio Emergency Management Agency
2825 West Granville Road
Worthington, Ohio 43085



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY, ET AL.

DOCKET NO. 50-440

PERRY NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 50
License No. NPF-58

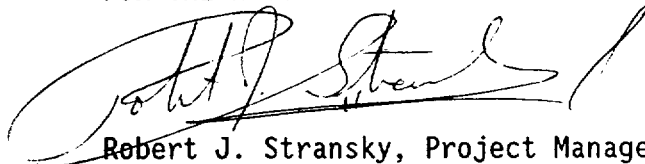
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by The Cleveland Electric Illuminating Company, Centerior Service Company, Duquesne Light Company, Ohio Edison Company, Pennsylvania Power Company, and Toledo Edison Company (the licensees) dated October 30, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-58 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 50 are hereby incorporated into this license. The Cleveland Electric Illuminating Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "Robert J. Stransky", is written over a horizontal line.

Robert J. Stransky, Project Manager
Project Directorate III-3
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of issuance: August 11, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 50

FACILITY OPERATING LICENSE NO. NPF-58

DOCKET NO. 50-440

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Overleaf pages are provided to maintain document completeness.

Remove

3/4 3-11

3/4 3-15

3/4 3-16

3/4 3-30

3/4 3-31

Insert

3/4 3-11

3/4 3-15

3/4 3-16

3/4 3-30

3/4 3-31

TABLE 3.3.2-1
ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM (a)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
1. <u>PRIMARY CONTAINMENT ISOLATION</u>			
a. Reactor Vessel Water Level - Low, Level 2 (Division 1 & 2)	2	1, 2, 3 and #	20
b. Reactor Vessel Water Level - Low, Level 2 (Division 3)	4 ^(d)	1, 2, 3 and #	28
c. Drywell Pressure - High (Division 1 & 2)	2	1, 2, 3	20
d. Drywell Pressure - High (Division 3)	4 ^(d)	1, 2, 3	28
e. Containment and Drywell Purge Exhaust Plenum Radiation - High	2 ^(b)	1, 2, 3 and *	21
f. Reactor Vessel Water Level - Low, Level 1	2	1, 2, 3 and #	20
g. Manual Initiation (Division 1 & 2)	2 ^(c)	1, 2, 3 and *	22
h. Manual Initiation (Division 3)	1 ^(e)	1, 2, 3 and *	28
2. <u>MAIN STEAM ISOLATION</u>			
a. Reactor Vessel Water Level - Low, Level 1	2	1, 2, 3	20
b. Main Steam Line Radiation - High	2	1, 2	23
c. Main Steam Line Pressure - Low	2	1	24
d. Main Steam Line Flow - High	2/line	1, 2, 3	23
e. Condenser Vacuum - Low	2	1, 2**, 3**	23
f. Main Steam Line Tunnel Temperature - High	2	1, 2, 3	23
g. Main Steam Line Tunnel Δ Temperature - High	2	1, 2, 3	23
h. Turbine Building Main Steam Line Temperature - High	2	1, 2, 3	23
i. Manual Initiation	2	1, 2, 3	22

PERRY - UNIT 1

3/4 3-11

Amendment No. 44,50

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM (a)</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
3. <u>SECONDARY CONTAINMENT ISOLATION</u>			
a. Reactor Vessel Water Level - Low, Level 2	2	1, 2, 3 and #	25
b. Drywell Pressure - High	2	1, 2, 3	25
c. Manual Initiation	2	1, 2, 3	22
	2	*	25
4. <u>REACTOR WATER CLEANUP SYSTEM ISOLATION</u>			
a. Δ Flow - High	1	1, 2, 3	27
b. Δ Flow Timer	1	1, 2, 3	27
c. Equipment Area Temperature - High	1	1, 2, 3	27
d. Equipment Area Δ Temperature - High	1	1, 2, 3	27
e. Reactor Vessel Water Level - Low, Level 2	2	1, 2, 3	27
f. Main Steam Line Tunnel Ambient Temperature - High	1	1, 2, 3	27
g. Main Steam Line Tunnel Δ Temperature - High	1	1, 2, 3	27
h. SLCS Initiation	1	1, 2, 3	27
i. Manual Initiation	2	1, 2, 3	26

TABLE 3.3.2-1 (Continued)
ISOLATION ACTUATION INSTRUMENTATION
ACTION

- ACTION 20 -** In OPERATIONAL CONDITION 1, 2 or 3, be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours. In OPERATIONAL CONDITION #, suspend CORE ALTERATIONS and operations with a potential for draining the reactor vessel.
- ACTION 21 -** Close the affected system isolation valve(s) within one hour or:
- a. In OPERATIONAL CONDITION 1, 2 or 3, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 - b. In Operational Condition *, suspend CORE ALTERATIONS, handling of irradiated fuel in the primary containment and operations with a potential for draining the reactor vessel.
- ACTION 22 -** Restore the manual initiation function to OPERABLE status within 48 hours or:
- a. In OPERATIONAL CONDITION 1, 2, or 3, be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 - b. In OPERATIONAL CONDITION *, suspend CORE ALTERATIONS, operations with a potential for draining the reactor vessel, and handling of irradiated fuel in the primary containment.
- ACTION 23 -** Be in at least STARTUP with the associated isolation valves closed within 6 hours or be in at least HOT SHUTDOWN within 12 hours and in COLD SHUTDOWN within the next 24 hours.
- ACTION 24 -** Be in at least STARTUP within 6 hours.
- ACTION 25 -** Verify SECONDARY CONTAINMENT INTEGRITY with the annulus exhaust gas treatment system operating within one hour.
- ACTION 26 -** Restore the manual initiation function to OPERABLE status within 8 hours or close the affected system isolation valves within 1 hour and declare the affected system inoperable.
- ACTION 27 -** Close the affected system isolation valves within one hour and declare the affected system inoperable.
- ACTION 28 -** Within one hour lock the affected system isolation valves closed, or verify, by remote indication, that the valve(s) is closed and electrically disarmed, or isolate the penetration(s) and declare the affected system inoperable.

NOTES

- * When handling irradiated fuel in the primary containment and during CORE ALTERATIONS and operations with a potential for draining the reactor vessel.
- ** When any turbine stop valve is greater than 90% open and/or the key locked Condenser Low Vacuum Bypass Switch is in the normal position.
- # During CORE ALTERATIONS and operations with a potential for draining the reactor vessel.

TABLE 3.3.2-1 (Continued)

ISOLATION ACTUATION INSTRUMENTATION
ACTION

NOTES (Continued)

- (a) A channel may be placed in an inoperable status for up to 2 hours for required surveillance without placing the trip system in the tripped condition provided at least one other OPERABLE channel in the same trip system is monitoring that parameter.
- (b) Containment and Drywell Purge System inboard and outboard isolation valves each use a separate two out of two isolation logic.
- (c) There is only one (1) RCIC manual initiation channel for RCIC system containment isolation valves.
- (d) Division 3 has only one trip system consisting of four channels logically combined in a one-out-of-two-twice configuration which only closes the HPCS Suppression Pool Test Return Valve (1E22-F023).
- (e) Division 3 Manual Initiation consists of a single channel in a single trip system.

TABLE 3.3.3-1 (Continued)

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP FUNCTION</u> ^(a)	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>ACTION</u>
B. <u>DIVISION 2 TRIP SYSTEM</u>			
1. <u>RHR B AND C (LPCI MODE)</u>			
a. Reactor Vessel Water Level - Low, Level 1	2 ^(b)	1, 2, 3, 4*, 5*	30
b. Drywell Pressure - High	2 ^(b)	1, 2, 3	30
c. Reactor Vessel Pressure - Low (LPCI Injection Valve Permissive)	1	1, 2, 3 4*, 5*	31 32
d. LPCI Pump B Start Time Delay Relay	1	1, 2, 3, 4*, 5*	31
e. LPCI Pump Discharge Flow - Low (Bypass)	1/pump	1, 2, 3, 4*, 5*	39
f. Manual Initiation	1	1, 2, 3, 4*, 5*	33
2. <u>AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "B"</u>[#]			
a. Reactor Vessel Water Level - Low, Level 1	2 ^(b)	1, 2, 3	30
b. Manual Inhibit	1	1, 2, 3	33
c. ADS Timer	1	1, 2, 3	31
d. Reactor Vessel Water Level - Low, Level 3 (Permissive)	1	1, 2, 3	31
e. LPCI Pump B and C Discharge Pressure - High (Permissive)	2	1, 2, 3	31
f. Manual Initiation	2	1, 2, 3	33

TABLE 3.3.3-1 (Continued)
EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION

ACTION

- ACTION 30 -** With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement:
- a. With one channel inoperable, place the inoperable channel in the tripped condition within one hour or declare the associated system inoperable.
 - b. With more than one channel inoperable, declare the associated system inoperable.
- ACTION 31 -** With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement, declare the associated ADS trip system or ECCS inoperable.
- ACTION 32 -** With the number of OPERABLE channels less than the Minimum OPERABLE Channels per Trip Function requirement, place the inoperable channel in the tripped condition within one hour.
- ACTION 33 -** With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement, restore the inoperable channel to OPERABLE status within 8 hours or declare the associated ADS valve or ECCS inoperable.
- ACTION 34 -** With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement, place the inoperable channel(s) in the tripped condition within one hour or declare the HPCS system inoperable.
- ACTION 35 -** With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement, place at least one inoperable channel in the tripped condition within one hour, or align the HPCS system to take suction from the suppression pool, or declare the HPCS system inoperable.
- ACTION 36** With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement, place at least one inoperable channel in the tripped condition within one hour or declare the HPCS system inoperable.
- ACTION 37 -** With the number of OPERABLE channels less than the Total Number of Channels, declare the associated emergency diesel generator inoperable and take the ACTION required by Specification 3.8.1.1 or 3.8.1.2, as appropriate.
- ACTION 38 -** With the number of OPERABLE channels less than the Total Number of Channels, place the inoperable channel in the tripped condition within 1 hour; operation may then continue until performance of the next required CHANNEL FUNCTIONAL TEST.
- ACTION 39** With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement, place the inoperable channel in the tripped condition within one hour. Restore the inoperable channel to OPERABLE status within 7 days or declare the associated system inoperable.

TABLE 3.3.3-2

EMERGENCY CORE COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
A. <u>DIVISION 1 TRIP SYSTEM</u>		
1. <u>RHR-A (LPCI MODE) AND LPCS SYSTEM</u>		
a. Reactor Vessel Water Level - Low, Level 1	≥ 16.5 inches*	≥ 14.3 inches
b. Drywell Pressure - High	≤ 1.68 psig	≤ 1.88 psig
c. LPCS Pump Discharge Flow - Low (Bypass)	> 1350 gpm	> 1200 gpm
d. Reactor Vessel Pressure - Low (LPCS Injection Valve Permissive)	577.7 ± 15 psig	$577.7 + 30, -95$ psig
e. Reactor Vessel Pressure - Low (LPCI Injection Valve Permissive)	$502.5 + 5, -10$ psig	$502.5 + 10, -40$ psig
f. LPCI Pump A Start Time Delay Relay	≤ 5 seconds	≤ 5.25 seconds
g. LPCI Pump A Discharge Flow - Low (Bypass)	> 1650 gpm	> 1450 gpm
h. Manual Initiation	NA	NA
2. <u>AUTOMATIC DEPRESSURIZATION SYSTEM TRIP SYSTEM "A"</u>		
a. Reactor Vessel Water Level - Low, Level 1	≥ 16.5 inches*	≥ 14.3 inches
b. Manual Inhibit	NA	NA
c. ADS Timer	≤ 105 seconds	≤ 117 seconds
d. Reactor Vessel Water Level - Low, Level 3 (Permissive)	≥ 177.7 inches*	≥ 177.1 inches
e. LPCS Pump Discharge Pressure - High (Permissive)	≥ 145 psig, increasing	≥ 125 psig, increasing
f. LPCI Pump A Discharge Pressure - High (Permissive)	≥ 125 psig, increasing	≥ 115 psig, increasing
g. Manual Initiation	NA	NA



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 50 TO FACILITY OPERATING LICENSE NO. NPF-58
THE CLEVELAND ELECTRIC ILLUMINATING COMPANY, ET AL.
PERRY NUCLEAR POWER PLANT, UNIT NO. 1
DOCKET NO. 50-440

1.0 INTRODUCTION

By letter dated October 30, 1991, the Cleveland Electric Illuminating Company and Centerior Service Company submitted a request to amend the Technical Specifications (TSs) issued to the Perry Nuclear Power Plant, Unit 1. The proposed amendment consisted of changes to TS Table 3.3.2-1, "Isolation Actuation Instrumentation," and Table 3.3.3-1, "Emergency Core Cooling System Actuation Instrumentation." The changes proposed by the licensee would clarify the requirements for instrumentation used to isolate the High Pressure Core Spray (HPCS) test return line and to actuate the HPCS system. The licensee also proposed to identify specific actions to be taken if one of the HPCS reactor vessel water level or drywell pressure instruments becomes inoperable.

2.0 EVALUATION

Section 1 of TS Table 3.3.2-1 is applicable for actuation of the primary containment isolation valves. These valves belong to Division 1, 2, and 3. The Division 1 and 2 isolation logic employs double trip systems and the Division 3 isolation logic employs a single trip system. The current wording of Item 1.a (Reactor Vessel Water level-low, level 2), Item 1.b (Drywell pressure-high), and Item 1.e (Manual initiation) are based on double trip systems. These items are also applicable to HPCS test return line isolation valve 1E22-F023 which is a Division 3 valve with a single trip system using a one-out-of-two-taken-twice logic. The current TS permits a minimum of two operable channels per trip system for all items listed in Section 1 of TS Table 3.3.2-1. However, the instrumentation logic used for valve 1E22-F023 requires all four operable channels in its trip system to avoid loss of this valve under all combinations of inoperable instruments without resorting to any TS action. To overcome these limitations, the licensee proposed to add references stating that the current Items 1.a, 1.b, and 1.e belong to Division 1 and 2 (redesignating them as Items 1.a, 1.c, and 1.g, respectively) and add three new corresponding items (Items 1.b, 1.d, and 1.h) for Division 3 instrumentation including addition of Notes (d) and (e).

Note (d) is applicable to new Items 1.b (Reactor vessel water level-low, level 2 (Division 3)), and Item 1.d (Drywell pressure-high (Division 3)). Note (d) states, "Division 3 has only one trip system consisting of four channels logically combined in a one-out-of-two-twice configuration which only closes the HPCS Suppression Pool Test Return Valve (1E22-F023). Note (e) is

applicable to new Item 1.h (manual a initiation (Division 3)). Note (e) states, "Division 3 Manual Initiation consists of a single channel in a single trip system."

The licensee proposed Action 28 for new Items 1.b, 1.d, and 1.h. The current Action 28 states, "Within one hour lock the affected system isolation valves closed, or verify, by remote indication, that the valve(s) is closed and electrically disarmed, or isolate the penetration(s) and declare the affected system inoperable." The licensee proposed to add a comma between "isolate the penetration(s)" and "and declare the affected system inoperable." Without this comma, the current Action 28 may be interpreted to mean that "declaration of the affected system inoperable" is applicable only after isolation of the penetration(s). It is the intent of Action 28 that the affected system be declared inoperable no matter which of the isolation actions is taken within Action 28. This clarification is achieved by adding a comma between the two statements.

The TS Table 3.3.3-1 specifies trip logic requirements for emergency core cooling system actuation instrumentation. The licensee proposed to modify Action 34 and replace Action 31 by Action 34, for Item C.1.c of Table 3.3.3-1.

Item C.1.a of Table 3.3.3-1 covers instrumentation of reactor vessel level-low, level 2. Item C.1.b covers instrumentation of drywell pressure-high, and Item C.1.c covers instrumentation of reactor vessel water level-high, level 8. Current Items C.1.a and C.1.b refer to Action 34 which is based on a two trip system and states:

"With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement:

- a. For one trip system, place that trip system in the tripped condition within one hour or declare the HPCS system inoperable.
- b. For both trip systems, declare the HPCS system inoperable."

The existing instrumentation logic of Items C.1.a and C.1.b employs only one trip system belonging to Division 3. Therefore, the current Action 34 is not appropriate for Items C.1.a and C.1.b. To overcome this inconsistency, the licensee proposed to reword Action 34 to make it appropriate for a single trip system.

The current Item C.1.c calls for Action 31 which reads, "With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement, declare the associated ADS trip system or ECCS inoperable." The licensee proposed to use Action 34 for Item C.1.c because the Items C.1.a, C.1.b, and C.1.c employ the same type of instrumentation logic.

To a large extent the proposed changes agree with the current TS for Grand Gulf, Unit 1 and Clinton, Unit 1.

We have reviewed the justification provided by the licensee and agree that the current wording of the TS Table 3.3.2-1, Section 1, and Table 3.3.3-1, Section C.1 is inappropriate, can cause compliance problems, and loss of containment isolation during an accident condition. The proposed TS changes would better represent the existing instrumentation logic, clarifying TS requirements, and improving plant safety, and therefore they are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Ohio State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or changes a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding (56 FR 64651). Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Mazumdar

Date: August 11, 1993