



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

June 10, 1987

Docket No.: 50-440

Mr. Murray R. Edelman
Sr. Vice President
Nuclear Group
The Cleveland Electric Illuminating
Company
P. O. Box 5000
Cleveland, Ohio 44101

Dear Mr. Edelman:

SUBJECT: EMERGENCY CHANGE TO TECHNICAL SPECIFICATIONS CONCERNING MAIN STEAM
LINE HIGH TEMPERATURE

RE: Perry Nuclear Power Plant, Unit No. 1

The Commission has issued the enclosed Amendment No. 7 to Facility Operating License No. NPF-58 for the Perry Nuclear Power Plant, Unit No. 1. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated May 29, 1987.

This amendment changes the main steam line high temperature trip setpoints and allowable values for the turbine building and steam tunnel, items 2.f, 2.g, 2.h, 4.f, 4.g, 5.f and 5.g of Table 3.3.2-2 of the TSs. These changes were authorized by telephone on May 29, 1987, and confirmed by letter dated June 1, 1987.

We have reviewed the circumstances associated with your request for an emergency TS change and concluded that the relief provided by this amendment is necessary to avoid effective derating of Perry Unit 1 and to permit continued startup testing and operation in a timely manner. We have also concluded that these circumstances could not have been avoided. Therefore, we concluded that a valid emergency existed, as defined in 10 CFR 50.91(a)(5), and your application has been processed accordingly.

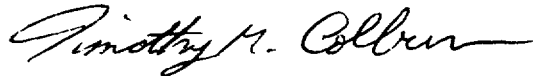
8706240424 870610
PDR ADDCK 05000440
P PDR

June 10, 1987

- 2 -

A copy of our Safety Evaluation is also enclosed. Notice of Issuance and Final Determination of No Significant Hazards Consideration and Opportunity for Hearing will be included in the Commission's biweekly Federal Register Notice.

Sincerely,



Timothy G. Colburn, Project Manager
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

cc w/enclosures:
See next page

Enclosures:

1. Amendment No. 7 to
License No. NPF-58
2. Safety Evaluation

Mr. Murray R. Edelman
The Cleveland Electric
Illuminating Company

Perry Nuclear Power Plant
Unit 1

cc:

Shaw, Pittman, Potts & Trowbridge
2300 N Street, N.W.
Washington, D.C. 20037

Donald H. Hauser, Esq.
The Cleveland Electric
Illuminating Company
P.O. Box 5000
Cleveland, Ohio 44101

Resident Inspector's Office
U.S. Nuclear Regulatory Commission
Parmlly at Center Road
Perry, Ohio 44081

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Frank P. Weiss, Esq.
Assistant Prosecuting Attorney
105 Main Street
Lake County Administration Center
Painesville, Ohio 44077

Ms. Sue Hiatt
OCRE Interim Representative
8275 Munson
Mentor, Ohio 44060

Terry J. Lodge, Esq.
618 N. Michigan Street
Suite 105
Toledo, Ohio 43624

John G. Cardinal, Esq.
Prosecuting Attorney
Ashtabula County Courthouse
Jefferson, Ohio 44047

Eileen M. Buzzelli
The Cleveland Electric
Illuminating Company
P. O. Box 97 E-210
Perry, Ohio 44081

Mr. James W. Harris, Director
Division of Power Generation
Ohio Department of Industrial
Relations
P.O. Box 825
Columbus, Ohio 43216

The Honorable Lawrence Logan
Mayor, Village of Perry
4203 Harper Street
Perry, Ohio 44081

The Honorable Robert V. Orosz
Mayor, Village of North Perry
North Perry Village Hall
4778 Lockwood Road
North Perry Village, Ohio 44081

Attorney General
Department of Attorney General
30 East Broad Street
Columbus, Ohio 43216

Radiological Health Program
Ohio Department of Health
1224 Kinnear Road
Columbus, Ohio 43212

Ohio Environmental Protection
Agency
361 East Broad Street
Columbus, Ohio 43266-0558

Mr. James R. Secor, Chairman
Perry Township Board of Trustees
Box 65
4171 Main Street
Perry, Ohio 44081

State of Ohio
Public Utilities Commission
180 East Broad Street
Columbus, Ohio 43266-0573

Distribution/Concurrence for Letter to Edelman dated - 6/10/87Distribution

Docket File

NRC PDR

Local PDR

G. Holahan

R. Ingram

T. Colburn

OGC-Bethesda

D. Hagan

E. Jordan

J. Partlow

T. Barnhart (4)

Wanda Jones

E. Butcher

A. Thadani

ACRS (10)

GPA/PA

ARM/LFMB

PD31 Plant Gray File

cc: Plant Service List

TGC for
 LA: PD31: DRSP
 RINGRAM
 6/3/87

TGC
 PM: PD31: DRSP
 TCOLBURN: dlm
 6/3/87

JVM
 ACTINGD: PD31: DRSP
 MJVIRGILIO
 6/3/87

gmj
 AD/DRSP
 GMHOLAHAN
 6/4/87

w/changes noted
AT
 OGC
 AD/S: DEST
 ATHADANI
 6/8/87 6/3/87
Chgs. made
6/10/87 - m



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CLEVELAND ELECTRIC ILLUMINATING COMPANY, ET AL

DOCKET NO. 50-440

PERRY NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 7
License No. NPF-58

1. The Nuclear Regulatory Commission (the Commission) has found that
 - A. The application for amendment by Cleveland Electric Illuminating Company, Duquesne Light Company, Ohio Edison Company, Pennsylvania Power Company, and Toledo Edison Company (the licensees) dated May 29, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-58 is hereby amended to read as follows:

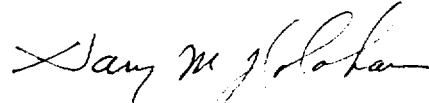
Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. are hereby incorporated into this license. Cleveland Electric Illuminating Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

8706240427 870610
PDR ADDCK 05000440
P PDR

3. This license amendment is effective as of May 29, 1987.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in dark ink, appearing to read "Gary M. Holahan". The signature is fluid and cursive, with the first name "Gary" being the most prominent.

Gary M. Holahan, Assistant Director
for Region III and V Reactors
Division of Reactor Projects III, IV, V
& Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 10, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 7

FACILITY OPERATING LICENSE NO. NPF-58

DOCKET NO. 50-440

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Overleaf page(s) provided to maintain document completeness.

Remove

Pages 3/4 3-17
3/4 3-18
3/4 3-19

Insert

Pages 3/4 3-17
3/4 3-18
3/4 3-19

TABLE 3.3.2-2
ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
1. <u>PRIMARY CONTAINMENT ISOLATION</u>		
a. Reactor Vessel Water Level - Low, Level 2	≥ 129.8 inches*	≥ 127.6 inches
b. Drywell Pressure - High	≤ 1.68 psig	≤ 1.88 psig
c. Containment and Drywell Purge Exhaust Plenum Radiation - High	≤ 2 mR/hr** above background	≤ 4 mR/hr** above background
d. Reactor Vessel Water Level - Low, Level 1	≥ 16.5 inches*	≥ 14.3 inches
e. Manual Initiation	NA	NA
2. <u>MAIN STEAM LINE ISOLATION</u>		
a. Reactor Vessel Water Level - Low, Level 1	≥ 16.5 inches*	≥ 14.3 inches
b. Main Steam Line Radiation - High	≤ 3.0 x full power background	< 3.6 x full power background
c. Main Steam Line Pressure - Low	≥ 807.0 psig	≥ 795.0 psig
d. Main Steam Line Flow - High	≤ 183 psid	≤ 191 psid
e. Condenser Vacuum - Low	≥ 8.5 inches Hg. vacuum	≥ 7.6 inches Hg. vacuum
f. Main Steam Line Tunnel Temperature - High	$\leq 154.4^{\circ}\text{F}^{**}$	$\leq 158.9^{\circ}\text{F}^{**}$
g. Main Steam Line Tunnel Δ Temperature - High	$\leq 103.6^{\circ}\text{F}^{**}$	$\leq 107.4^{\circ}\text{F}^{**}$
h. Turbine Building Main Steam Line Temperature - High	$\leq 134.4^{\circ}\text{F}^{**}$	$\leq 138.9^{\circ}\text{F}^{**}$
i. Manual Initiation	NA	NA

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
3. <u>SECONDARY CONTAINMENT ISOLATION</u>		
a. Reactor Vessel Water Level - Low, Level 2	≥ 129.8 inches*	≥ 127.6 inches
b. Drywell Pressure - High	≤ 1.68 psig	≤ 1.88 psig
c. Manual Initiation	NA	NA
4. <u>REACTOR WATER CLEANUP SYSTEM ISOLATION</u>		
a. Δ Flow - High	≤ 68 gpm**	≤ 77.1 gpm**
b. Δ Flow Timer	≤ 45 seconds	≤ 47 seconds
c. Equipment Area Temperature - High		
1. RWCU Hx Room	$< 136.4^{\circ}\text{F}$	$< 138.9^{\circ}\text{F}$
2. Pump Rooms, Valve Nest Room	$\leq 135.4^{\circ}\text{F}$	$\leq 137.9^{\circ}\text{F}$
d. Equipment Area Δ Temperature - High		
1. RWCU Hx Room	$< 76.65^{\circ}\text{F}$	$< 78.4^{\circ}\text{F}$
2. RWCU Pump Rooms, Valve Nest Room	$\leq 28.65^{\circ}\text{F}$	$\leq 30.4^{\circ}\text{F}$
e. Reactor Vessel Water Level - Low, Level 2	≥ 129.8 inches*	≥ 127.6 inches
f. Main Steam Line Tunnel Ambient Temperature - High	$\leq 154.4^{\circ}\text{F}^{**}$	$\leq 158.9^{\circ}\text{F}^{**}$
g. Main Steam Line Tunnel Δ Temperature - High	$\leq 103.6^{\circ}\text{F}^{**}$	$\leq 107.4^{\circ}\text{F}^{**}$
h. SLCS Initiation	NA	NA
i. Manual Initiation	NA	NA

PERRY - UNIT 1

3/4 3-18

Amendment No. 7

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
5. <u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION</u>		
a. RCIC Steam Line Flow - High	$\leq 290'' \text{ H}_2\text{O}^{**}$	$\leq 298.5'' \text{ H}_2\text{O}^{**}$
b. RCIC Steam Supply Pressure - Low	$\geq 60 \text{ psig}$	$\geq 55 \text{ psig}$
c. RCIC Turbine Exhaust Diaphragm Pressure - High	$\leq 10 \text{ psig}$	$\leq 20 \text{ psig}$
d. RCIC Equipment Room Ambient Temperature - High	$\leq 143.4^\circ\text{F}$	$\leq 145.9^\circ\text{F}$
e. RCIC Equipment Room Δ Temperature - High	$\leq 37.25^\circ\text{F}$	$\leq 39^\circ\text{F}$
f. Main Steam Line Tunnel Ambient Temperature - High	$\leq 154.4^\circ\text{F}^{**}$	$\leq 158.9^\circ\text{F}^{**}$
g. Main Steam Line Tunnel Δ Temperature - High	$\leq 103.6^\circ\text{F}^{**}$	$\leq 107.4^\circ\text{F}^{**}$
h. Main Steam Line Tunnel Temperature Timer	$\leq 29 \text{ minutes}$	$\leq 30 \text{ minutes}$
i. RHR Equipment Room Ambient Temperature - High	$\leq 157.4^\circ\text{F}$	$\leq 159.9^\circ\text{F}$
j. RHR Equipment Room Δ Temperature - High	$\leq 50.65^\circ\text{F}$	$\leq 52.4^\circ\text{F}$
k. RCIC Steam Flow High Timer	$3 \text{ seconds} \leq t \leq 13'' \text{ seconds}$	$3 \text{ seconds} \leq t \leq 13 \text{ seconds}$
l. Drywell Pressure - High	$\leq 1.68 \text{ psig}$	$\leq 1.88 \text{ psig}$
m. Manual Initiation	NA	NA

TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
6. <u>RHR SYSTEM ISOLATION</u>		
a. RHR Equipment Area Ambient Temperature - High	$\leq 157.4^{\circ}\text{F}$	$\leq 159.9^{\circ}\text{F}$
b. RHR Equipment Area Δ Temperature - High	$\leq 50.65^{\circ}\text{F}$	$\leq 52.4^{\circ}\text{F}$
c. RHR/RCIC Steam Line Flow - High	$\leq 105'' \text{ H}_2\text{O}^{**}$	$\leq 114'' \text{ H}_2\text{O}^{**}$
d. Reactor Vessel Water Level - Low, Level 3	$\geq 177.7 \text{ inches}^*$	$\geq 177.1 \text{ inches}$
e. Reactor Vessel (RHR Cut-in Permissive) Pressure - High	$\leq 135 \text{ psig}$	$\leq 150 \text{ psig}$
f. Drywell Pressure - High	$\leq 1.68 \text{ psig}$	$\leq 1.88 \text{ psig}$
g. Manual Initiation	NA	NA

*See Bases Figure B 3/4 3-1.

**Initial setpoint. Final setpoint to be determined during startup test program. Any required change to this setpoint shall be submitted to the Commission within 90 days of test completion.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 7 TO FACILITY OPERATING LICENSE NO. NPF-58

CLEVELAND ELECTRIC ILLUMINATING COMPANY, ET AL

PERRY NUCLEAR POWER PLANT, UNIT NO. 1

DOCKET NO. 50-440

1.0 INTRODUCTION

By letter dated May 29, 1987, Cleveland Electric Illuminating Company, Duquesne Light Company, Ohio Edison Company, Pennsylvania Power Company, and Toledo Edison Company (the licensees) requested an amendment to Facility Operating License No. NPF-58 for Perry Nuclear Power Plant, Unit No. 1, pursuant to 10 CFR 50.90. The licensee also requested this amendment pursuant to emergency circumstances as described in 10 CFR 50.91(a)(5). The change would modify the isolation setpoints for the following isolation signals:

1. Main steam line tunnel temperature-high - Technical Specification 3.3.2, Table 3.3.2-2, Items 2.f, 4.f, and 5.f.
2. Main steam line tunnel delta temperature-high - Technical Specification 3.3.2, Table 3.3.2-2, Items 2.g, 4.g. and 5.g.
3. Turbine building main steam line temperature-high - Technical Specification 3.3.2, Table 3.3.2-2, Item 2.h.

The trip setpoint would be modified for: (1) main steam line tunnel temperature from $\leq 131.4^{\circ}\text{F}$ with an allowable value of 133.9°F to $< 154.4^{\circ}\text{F}$ with an allowable value of $< 158.9^{\circ}\text{F}$; (2) main steam line tunnel delta temperature-high from $\leq 80.65^{\circ}\text{F}$ with an allowable value of $< 82.4^{\circ}\text{F}$ to $\leq 103.6^{\circ}\text{F}$ with an allowable value of $\leq 107.4^{\circ}\text{F}$; and (3) turbine building main steam line temperature-high from $\leq 116.4^{\circ}\text{F}$ with an allowable value of $\leq 118.9^{\circ}\text{F}$ to $\leq 135.4^{\circ}\text{F}$ with an allowable value of $\leq 138.9^{\circ}\text{F}$.

These changes are needed because actual operating temperatures have exceeded anticipated operating temperatures due to conservative initial setpoints and subsequent sudden increases in outside ambient temperature coupled with increased power levels experienced in the power ascension test program.

8706240429 870610
PDR ADOCK 05000440
P PDR

2.0 EVALUATION

The purpose of these temperature trip setpoints is to detect a 25 gpm or greater steam leak in the area of the instruments. In the main steam line tunnel, the trip setpoint would isolate main steam isolation valves (MSIVs), reactor water cleanup system (RWCU), and the reactor core isolation cooling (RCIC) system, and for the turbine building instruments, would isolate the MSIVs. Isolations of the MSIVs would automatically initiate a reactor scram. The requested changes to the setpoints are based on existing operating analysis to date, and may require a further increase as the plant approaches 100% power and experiences peak summertime ambient air temperatures.

The automatic closure of the associated containment isolation valves for RCIC, MSIV and RWCU prevents excessive loss of reactor coolant and release of a significant amount of radioactive material if a leak were to occur in a main steam line outside containment. Such a leak would result in increased ambient air temperatures in the location of the temperature sensors. As indicated in the Final Safety Analysis Report, Section 15.6.4, however, high steam line flow is the initiating condition for the main steam isolation following a main steam line break outside containment.

Original leak detection setpoints were based on an assumed delta temperature above theoretical normal operating temperatures. The existing setpoints did not account for temperature stratification, hot spots due to radiant heat from components and reduced ventilation. The licensee has performed an analysis calculating new setpoints based upon new "normal" operating temperatures and the same assumed delta to account for the 25 gpm leak.

The licensee has verified that the higher temperature setpoints would not penetrate the temperature envelope for environmental qualification (EQ) of safety-related electrical equipment given the peak temperature transient resulting from the leak. While service life may be shortened slightly at the higher temperatures, the licensee has committed to evaluate the changes to service life, if any, and adjust the EQ maintenance program accordingly. The results of this evaluation will be provided to the NRC staff.

Because the change in setpoints will not affect the isolation setpoints for the steam line break accident outside containment (high steam line flow) and because the licensee has demonstrated that the new setpoints would perform their intended purpose of detecting a 25 gpm leak in the vicinity of main steam lines outside containment, the staff finds that the proposed changes are acceptable.

3.0 EMERGENCY CIRCUMSTANCES

The continued operation of the Perry Nuclear Power Plant, Unit No. 1, with the current containment isolation temperature setpoints would effectively amount to a derating of the unit, limiting it to a power level less than allowed by Test Condition 3 (75%) which it is currently in. Further, the licensee has provided information supporting the need for the changes which could not have been anticipated far enough in advance to support a normal amendment application process. While some increase in ambient temperature

was seen, sufficient margin appeared to exist between ambient temperatures and the trip setpoints so as not to require an emergency change. In addition, the licensee had improved local ventilation and made insulation modifications which were expected to mitigate the ambient temperature rise. Only recently CEI discovered that unseasonably high outside ambient air temperatures coupled with the higher power levels allowed by Test Condition 3, would cause local ambient air temperatures to rise to within one degree of the trip setpoints. Therefore, no further margin existed despite the licensee's efforts to mitigate the ambient air temperature rise. The licensee was forced to reduce power to avoid exceeding the trip setpoints, effectively derating the unit.

The NRC staff has reviewed the circumstances associated with the licensee's request and agrees that the amendment is necessary for Perry 1 to avoid deration of the unit and continue startup testing and that failure to act in a timely manner would result in deration of the plant. The staff also concludes that this situation could not reasonably have been avoided. The staff finds that a valid emergency exists, as defined by 10 CFR 50.91(a)(5).

3.1 FINAL NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

The Commission has provided standards for determining whether a significant hazards consideration exists as stated in 10 CFR 50.92. A proposed amendment to an operating license for a facility involves no significant hazards considerations if operation of the facility in accordance with a proposed amendment would not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

The licensee has provided a discussion concerning a finding relative to whether a significant hazards consideration exists.

The proposed amendment to the Technical Specifications would not involve a significant increase in the probability or consequences of an accident previously evaluated because there is no change in the design or performance of plant systems or components from those evaluated in the Final Safety Analysis Report (FSAR). The FSAR accident analysis used different isolation instrumentation for line breaks outside containment (i.e., high steam flow). No accident analyses depend upon this leak detection instrumentation which is a diverse method of leak detection. As discussed in FSAR Section 5.2.5.2.2, this isolation is provided following a leak to prevent excessive loss of reactor coolant and release of a significant amount of radioactive material from the reactor coolant pressure boundary. This function is maintained by the Technical Specification revision. Since there is no change in the methods of leak detection, no additional reactor coolant would be lost nor radioactive material released beyond that previously assumed for the leak event. Therefore, there is no increase in the probability or the consequences of an accident by this amendment. There is no increase in the probability of an accident by this amendment since the instrumentation affected is not part of the reactor protection system.

Since the proposed amendment does not change any previously reviewed and approved description or analysis described in the FSAR, the proposed amendment does not create the possibility of a new or different kind of accident. As discussed above, this isolation is provided following a leak to prevent excessive loss of reactor coolant and release of a significant amount of radioactive material from the reactor coolant pressure boundary. This function is maintained by the Technical Specification revision. Since there is no change in the size of the leak detected, no additional reactor coolant would be lost nor radioactive material released beyond that previously assumed for the leak event. Therefore, the change request does not create the possibility of a new or different kind of accident from any accident previously evaluated.

There would not be a significant reduction in a margin of safety because there will be no detrimental changes in the availability, operability or reliability of the systems. As discussed in FSAR Section 5.2.5.2.2, the setpoint is designed to identify a 25 gpm leak in these areas. The setpoints requested are established to meet this criteria. Therefore, no margin of safety has been affected.

The staff has concluded, based on the above, that the amendment does not increase the probability or consequences of an accident previously evaluated, does not create the possibility of a new or different kind of accident from any previously evaluated, and does not involve a significant reduction in a margin of safety. Accordingly, the amendment does not involve a significant hazards consideration.

3.2 STATE CONSULTATION

A copy of this amendment request was transmitted by the licensee to the State of Ohio via express carrier. Both the staff and the licensee made good faith efforts to contact the State but were unable to do so.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has made a final no significant hazards consideration finding with respect to this amendment. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 50.22(b), no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: T. Colburn

Dated: May 29, 1987