

MAY 11 1987

Docket No. 50-440

Mr. Murray R. Edelman, Vice President
Nuclear Operations Group
The Cleveland Electric Illuminating Company
P.O. Box 5000
Cleveland, Ohio 44101

Dear Mr. Edelman:

SUBJECT: PERRY NUCLEAR POWER PLANT, UNIT NO. 1 (TAC 65289)

Enclosed is an individual Notice of Consideration of Issuance of Amendment to Facility Operating License and Proposed No Significant Hazards Consideration Determination. This amendment was requested by your letter dated May 4, 1987 (PY-CEI/NRR-0643L). This Notice was forwarded to the Office of the Federal Register for publication.

Sincerely,

TGC

Timothy G. Colburn, Project Manager
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

Enclosure:
As stated

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Mr. Murray R. Edelman
The Cleveland Electric
Illuminating Company

Perry Nuclear Power Plant
Unit 1

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UNITED STATES NUCLEAR REGULATORY COMMISSION
CLEVELAND ELECTRIC ILLUMINATING COMPANY, ET AL.

DOCKET NO. 50-440

NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO
FACILITY OPERATING LICENSE AND PROPOSED NO SIGNIFICANT HAZARDS
CONSIDERATION DETERMINATION

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-58 issued to the Cleveland Electric Illuminating Company (CEI), the Duquesne Light Company, the Ohio Edison Company, the Pennsylvania Power Company, and the Toledo Edison Company (the licensees) for operation of the Perry Nuclear Power Plant, Unit No. 1, located in Lake County, Ohio.

The amendment would grant the licensees a one-time exception to Technical Specification 4.6.1.2.f to extend the leak testing interval of the Main Steamline Isolation Valves (MSIVs) in steamline "A" until July 12, 1987. Technical Specification 4.6.1.2.f requires that MSIVs be leak tested at least once per 18 months, and the MSIVs in steamline "A" are due to be tested by May 31, 1987.

CEI was granted an earlier Technical Specification exception for the operability of the Reactor Core Isolation Cooling (RCIC) system, which expires on May 31, 1987. The basis for the May 31st date was that modifications to the water level instruments which had proved successful during RCIC system

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testing were to be installed on all reference legs. These modifications were to be installed during a planned May-June 1987 maintenance outage; and it was planned that the steamline "A" MSIVs would be leak tested in accordance with the Technical Specification 4.6.1.2.f test time interval at that time. However, during a recent unplanned forced outage, the RCIC system instrument modifications were completed. Therefore, the only item which would force the plant to enter into a planned outage on May 31, 1987, is the leak testing of the MSIVs in steamline "A". Since CEI only recently determined that the post-modification testing on all of the reactor vessel water level instrumentation was performed successfully prior to the planned maintenance outage, it would serve no safety function to require the plant to be shut down on May 31, 1987 to conduct the steamline "A" MSIV tests.

The one-time change to extend the time interval for leak testing of the steamline "A" MSIVs is in response to the licensees' application for amendment dated May 4, 1987, which requested that the amendment be processed under exigent circumstances in accordance with 10 CFR 50.91(a)(6) in light of the short time remaining to perform the required MSIV leak tests.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant

increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

The proposed change does not increase the probability or consequences of an accident previously evaluated. CEI is proposing to perform leak testing of the MSIVs in steamline "A" six weeks later than presently required in the Plant Technical Specifications. The Commission's regulations in 10 CFR 50, Appendix J, "Primary Containment Leakage Testing for Water-Cooled Power Reactors," requires Type C tests to be conducted at intervals no greater than two years. The MSIVs in steamline "A" were last leak tested on July 13, 1985. Therefore, by postponing the leak tests until July 12, 1987, CEI will still be within the testing frequency interval required in 10 CFR 50, Appendix J.

The proposed change does not create a new or different kind of accident from any accident previously evaluated. CEI is proposing to postpone leak testing of the two MSIVs in steamline "A" from May 31, 1987, to July 12, 1987, and that by no later than July 12, 1987, CEI will perform all required testing on the MSIVs in accordance with the Commission's regulations. In addition, the change does not result in any modification to the plant design or systems operation.

The proposed change does not involve a significant reduction in a margin of safety. As is stated above, the two MSIVs in steamline "A" were leak tested on July 13, 1985, and the results of those tests were found to be well within the acceptance criteria established for each MSIV. CEI was issued a low power

operating license (No. NPF-45) for Unit 1 on March 18, 1986, and commenced nuclear heatup in August 1986. As such, the MSIVs in question have only seen approximately nine months of service; and the MSIVs would only have been in service for one year since last being tested, with approval of the test interval extension proposed.

Therefore, based on these considerations and the three criteria given above, the Commission has made a proposed determination that the amendment request, which will be effective only up to July 12, 1987, involves no significant hazards consideration.

The Commission has determined that failure to act in a timely way would result in an untimely and unnecessary shutdown of Unit 1 on May 31, 1987, with no enhancement of safety. Therefore, the Commission has insufficient time to issue its usual 30-day notice of the proposed action for public comment.

If the proposed determination becomes final, an opportunity for a hearing will be published in the FEDERAL REGISTER at a later date and any hearing request will not delay the effective date of the amendment.

If the Commission decides in its final determination that the amendment does involve a significant hazards consideration, a notice of opportunity for a prior hearing will be published in the FEDERAL REGISTER and, if a hearing is granted, it will be held before any amendment is issued.

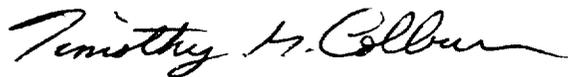
The Commission is seeking public comments on this proposed determination of no significant hazards consideration. Comments on the proposed determination may

be telephoned to Martin J. Virgilio, Acting Director, Project Directorate III-1, by collect call to (301) 492-4553, or submitted in writing to the Rules and Procedures Branch, Division of Rules and Records, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, and should cite the publication date and page number of this FEDERAL REGISTER notice. Written comments may also be delivered to Room 4000, Maryland National Bank Building, 7735 Old Georgetown Road, Bethesda, Maryland, from 8:15 A.M. to 5:00 P.M.

All comments received by May 29, 1987, will be considered in reaching a final determination. A copy of the application may be examined at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Perry Public Library, 3753 Main Street, Perry, Ohio 44081.

Dated at Bethesda, Maryland, this 11th day of May, 1987.

FOR THE NUCLEAR REGULATORY COMMISSION



Timothy G. Colburn, Project Manager
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects