May 7, 1990

Docket No. 50-440

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Mr. Alvin Kaplan, Vice President
Nuclear Group
The Cleveland Electric Illuminating Company
10 Center Road
Perry, Ohio 44081

Dear Mr. Kaplan:

SUBJECT: AMENDMENT NO. 29 TO FACILITY OPERATING LICENSE NO. NPF-58 (TAC NO. 68272)

The Commission has issued the enclosed Amendment No. 29 to Facility Operating License No. NPF-58 for the Perry Nuclear Power Plant, Unit No. 1. This amendment revises the Technical Specifications in response to your application dated May 20, 1988, as supplemented November 27, 1989.

This amendment revises the turbine first stage pressure bypass setpoints and allowable values in notes (h) and (b) of Tables 3.3.1-1 and 3.3.4.2-1, respectively, of the Technical Specifications to reflect test data rather than calculational data. The related bases are also changed.

A copy of the Safety Evaluation and of the notice of issuance are also enclosed. The notice has been forwarded to the Office of the Federal Register for publication.

Sincerely,

/s/

Timothy G. Colburn, Sr. Project Manager
Project Directorate III-3
Division of Reactor Projects - III, IV, V & Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 29 to License No. NPF-58
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:

See next page

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Date:	4/19/90	4/19/90	4/19/90	4/24/90

sub. to prior publn of EA

DOCUMENT NAME: 68272 AMD

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 7, 1990

Docket No. 50-440

Mr. Alvin Kaplan, Vice President
Nuclear Group
The Cleveland Electric Illuminating
Company
10 Center Road
Perry, Ohio 44081

Dear Mr. Kaplan:

SUBJECT: AMENDMENT NO. 29 TO FACILITY OPERATING LICENSE NO. NPF-58
(TAC NO. 68272)

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This amendment revises the turbine first stage pressure bypass setpoints and allowable values in notes (h) and (b) of Tables 3.3.1-1 and 3.3.4.2-1, respectively, of the Technical Specifications to reflect test data rather than calculational data. The related bases are also changed.

A copy of the Safety Evaluation and of the notice of issuance are also enclosed. The notice has been forwarded to the Office of the Federal Register for publication.

Sincerely,

Timothy G. Colburn

Timothy G. Colburn, Sr. Project Manager
Project Directorate III-3
Division of Reactor Projects - III, IV, V
& Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 29 to
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2. Safety Evaluation
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cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY, ET AL.

DOCKET NO. 50-440

PERRY NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 29
License No. NPF-58

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by The Cleveland Electric Illuminating Company, Duquesne Light Company, Ohio Edison Company, Pennsylvania Power Company, and Toledo Edison Company (the licensees) dated May 20, 1988 as supplemented November 27, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

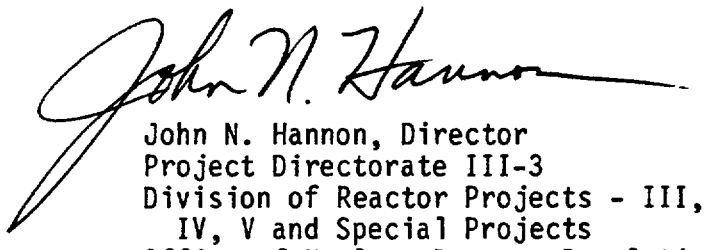
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-58 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 29 are hereby incorporated into this license. The Cleveland Electric Illuminating Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John N. Hannon, Director
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 7, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 29

FACILITY OPERATING LICENSE NO. NPF-58

DOCKET NO. 50-440

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. Overleaf pages are provided to maintain document completeness.

Remove	Insert
3/4 3-5	3/4 3-5
3/4 3-46	3/4 3-46
B 2-9	B 2-9
B 2-10	B 2-10

TABLE 3.3.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION

TABLE NOTATIONS

(a) A channel may be placed in an inoperable status for up to 2 hours for required surveillance without placing the trip system in the tripped condition provided at least one OPERABLE channel in the same trip system is monitoring that parameter.

(b) Unless adequate shutdown margin has been demonstrated per Specification 3.1.1 and the "one-rod-out" Refuel position interlock has been demonstrated OPERABLE per Specification 3.9.1, the shorting links shall be removed from the RPS circuitry prior to and during the time any control rod is withdrawn.*

(c) An APRM channel is inoperable if there are less than 2 LPRM inputs per level or less than 14 LPRM inputs to an APRM channel.

(d) This function is not required to be OPERABLE when the reactor pressure vessel head is removed per Specification 3.10.1.

(e) This function shall be automatically bypassed when the reactor mode switch is not in the Run position.

(f) This function is not required to be OPERABLE when DRYWELL INTEGRITY is not required.

(g) With any control rod withdrawn. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.

(h) This function is automatically bypassed when turbine first stage pressure is less than the value of turbine first stage pressure corresponding to 40%** of RATED THERMAL POWER.

---

*Not required for control rods removed per Specification 3.9.10.1 or 3.9.10.2.

**The Turbine First Stage Pressure Bypass Setpoints and corresponding Allowable Values are adjusted based on Feedwater temperatures (see 3/4.2.2 for definition of ΔT). The Setpoints and Allowable Values for various ΔTs are as follows:

T(°F)	Setpoint (psig)	Allowable Value (psig)
$0 = T$	≤ 212	≤ 218
$0 < \Delta T \leq 50$	≤ 190	≤ 196
$50 < \Delta T \leq 100$	≤ 168	≤ 174
$100 < \Delta T \leq 170$	≤ 146	≤ 152

TABLE 3.3.1-2

REACTOR PROTECTION SYSTEM RESPONSE TIMES

FUNCTIONAL UNIT	RESPONSE TIME (Seconds)
1. Intermediate Range Monitors:	
a. Neutron Flux - High	NA
b. Inoperative	NA
2. Average Power Range Monitor*:	
a. Neutron Flux - High, Setdown	NA
b. Flow Biased Simulated Thermal Power - High	$\leq$ 0.09**
c. Neutron Flux - High	$\leq$ 0.09
d. Inoperative	NA
3. Reactor Vessel Steam Dome Pressure - High	$\leq$ 0.35
4. Reactor Vessel Water Level - Low, Level 3	$\geq$ 1.05
5. Reactor Vessel Water Level - High, Level 8	$\leq$ 1.05
6. Main Steam Line Isolation Valve - Closure	$\leq$ 0.06
7. Main Steam Line Radiation - High	NA
8. Drywell Pressure - High	NA
9. Scram Discharge Volume Water Level - High	NA
10. Turbine Stop Valve - Closure	$\leq$ 0.06
11. Turbine Control Valve Fast Closure, Valve Trip System Oil Pressure - Low	$\leq$ 0.07#
12. Reactor Mode Switch Shutdown Position	NA
13. Manual Scram	NA

*Neutron detectors are exempt from response time testing. Response time shall be measured from the detector output or from the input of the first electronic component in the channel.

**Not including simulated thermal power time constant, 6 ± 0.6 seconds.

#Measured from start of turbine control valve fast closure.

SURVEILLANCE REQUIREMENTS

4.3.4.2.1 Each end-of-cycle recirculation pump trip system instrumentation channel shall be demonstrated OPERABLE by the performance of the CHANNEL FUNCTIONAL TEST and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3.4.2.1-1.

4.3.4.2.2 LOGIC SYSTEM FUNCTIONAL TESTS and simulated automatic operation of all channels shall be performed at least once per 18 months.

4.3.4.2.3 The END-OF-CYCLE RECIRCULATION PUMP TRIP SYSTEM RESPONSE TIME of each trip function shown in Table 3.3.4.2-3 shall be demonstrated to be within its limit at least once per 18 months. Each test shall include at least the logic of one type of channel input, turbine control valve fast closure or turbine stop valve closure, such that both types of channel inputs are tested at least once per 36 months. The measured time shall be added to the most recent breaker arc suppression time and the resulting END-OF-CYCLE RECIRCULATION PUMP TRIP SYSTEM RESPONSE TIME shall be verified to be within its limits.

4.3.4.2.4 The time interval necessary for breaker arc suppression from energization of the recirculation pump circuit breaker trip coil shall be measured at least once per 60 months.

TABLE 3.3.4.2-1

END-OF-CYCLE RECIRCULATION PUMP TRIP SYSTEM INSTRUMENTATION

TRIP FUNCTION	MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM(a)
1. Turbine Stop Valve - Closure	2(b)
2. Turbine Control Valve - Fast Closure	2(b)

(a) A trip system may be placed in an inoperable status for up to 2 hours for required surveillance provided that the other trip system is OPERABLE.

(b) This function is automatically bypassed when turbine first stage pressure is less than the value of turbine first stage pressure corresponding to 40%* of RATED THERMAL POWER.

*The Turbine First Stage Pressure Bypass Setpoints and corresponding Allowable Values are adjusted based on Feedwater temperatures (see 3/4.2.2 for definition of ΔT). The Setpoints and Allowable Values for various ΔTs are as follows:

T(°F)	Setpoint (psig)	Allowable Value (psig)
$0 = T$	$\leq$ 212	$\leq$ 218
$0 < \Delta T \leq 50$	$\leq$ 190	$\leq$ 196
$50 < \Delta T \leq 100$	$\leq$ 168	$\leq$ 174
$100 < \Delta T \leq 170$	$\leq$ 146	$\leq$ 152

REACTOR PROTECTION SYSTEM INSTRUMENTATION SETPOINTS (Continued)

8. Drywell Pressure-High

High pressure in the drywell could indicate a break in the primary pressure boundary systems. The reactor is tripped in order to minimize the possibility of fuel damage and reduce the amount of energy being added to the coolant and to the primary containment. The trip setting was selected as low as possible without causing spurious trips.

9. Scram Discharge Volume Water Level-High

The scram discharge volume receives the water displaced by the motion of the control rod drive pistons during a reactor scram. Should this volume fill up to a point where there is insufficient volume to accept the displaced water at pressures below 65 psig, control rod insertion would be hindered. The reactor is therefore tripped when the water level has reached a point high enough to indicate that it is indeed filling up, but the volume is still great enough to accommodate the water from the movement of the rods at pressures below 65 psig when they are tripped. The trip setpoint for each scram discharge volume is equivalent to a contained volume of approximately 24 gallons of water.

10. Turbine Stop Valve-Closure

The turbine stop valve closure trip anticipates the pressure, neutron flux, and heat flux increases that would result from closure of the stop valves. With a trip setting of 5% of valve closure from full open, the resultant increase in heat flux is such that adequate thermal margins are maintained during the worst case transient. As indicated in Table 3.3.1-1, this function is automatically bypassed below the turbine first stage pressure value equivalent to thermal power less than 40% of RATED THERMAL POWER.

The automatic bypass setpoint is feedwater temperature dependent due to the subcooling changes that affect the turbine first stage pressure - reactor power relationship. For RATED THERMAL POWER operation with feedwater temperature greater than or equal to 420°F, an allowable value of 218 psig turbine first stage pressure is provided for the bypass function. This setpoint is also applicable to operation at less than RATED THERMAL POWER with the correspondingly lower feedwater temperature. The allowable value is reduced as defined in Table 3.3.1-1 for RATED THERMAL POWER operation with a feedwater temperature between 370°F and 420°F; 370°F and 320°F; and 320°F and 250°F, respectively. Similarly, the reduced setpoint is applicable to operation at less than RATED THERMAL POWER with the correspondingly lower feedwater temperature.

11. Turbine Control Valve Fast Closure, Trip Oil Pressure-Low

The turbine control valve fast closure trip anticipates the pressure, neutron flux, and heat flux increase that could result from fast closure of the

REACTOR PROTECTION SYSTEM INSTRUMENTATION SETPOINTS (Continued)

turbine control valves due to load rejection with or without coincident failure of the turbine bypass valves. The Reactor Protection System initiates a trip when fast closure of the control valves is initiated by the fast acting solenoid valves and in less than 20 milliseconds after the start of control valve fast closure. This is achieved by the action of the fast acting solenoid valves in rapidly reducing hydraulic trip oil pressure at the main turbine control valve actuator disc dump valves. This loss of pressure is sensed by pressure switches whose contacts form the one-out-of-two twice logic input to the Reactor Protection System. This trip setting, a slower closure time, and a different valve characteristic from that of the turbine stop valve, combine to produce transients which are very similar to that for the stop valve. Relevant transient analyses are discussed in Section 15.2.2 of the Final Safety Analysis Report. As with the Turbine Stop Valve-Closure, this function is also bypassed below 40% of RATED THERMAL POWER. The basis for the bypass setpoint and reduction of the setpoint due to reduced feedwater temperatures is identical to that described for the Turbine Stop Valve-Closure.

12. Reactor Mode Switch Shutdown Position

The reactor mode switch Shutdown position provides additional manual reactor trip capability.

13. Manual Scram

The Manual Scram provides manual reactor trip capability. The manual scram function is composed of four push button switches in a one-out-of-two taken twice logic.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 29 TO FACILITY OPERATING LICENSE NO. NPF-58

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY, ET AL.

PERRY NUCLEAR POWER PLANT, UNIT NO. 1

DOCKET NO. 50-440

1.0 INTRODUCTION

The Cleveland Electric Illuminating Company (CEI) requested a license amendment to Technical Specification Tables (3.3.1-1 and 3.3.4.2-1) and the related Bases (Sections 2.2.1.10 and 2.2.1.11) for the Perry Nuclear Power Plant, Unit 1 by letter dated May 20, 1988 as supplemented November 27, 1989. The proposed amendment would revise notes to Technical Specifications Tables 3.3.1-1 and 3.3.4.2-1 and the Bases Section 2.2.1-10 and 2.2.1-11 to revise the First Stage Turbine Pressure Setpoints based on test data. The existing values were based on turbine thermal heat balance calculations, and the licensees determined through testing that these values have been shown to be overly conservative. The current requirement that the turbine valve closure scrams should be bypassed below the turbine first stage pressure equivalent to 40% of rated thermal power (RTP) remains unchanged. The proposed Setpoints and Allowable Values were established based on data determined by startup testing. The bases for reducing the setpoints due to reduced feedwater temperature also remains unchanged. In the proposed change the Setpoints and Allowable Values would be depicted in terms of actual turbine first stage pressure, rather than as a percentage of the Turbine Control Valve wide-open position. The actual pressure is more accurately meaningful to measure RTP.

2.0 EVALUATION

Turbine First Stage Pressure has been historically used as the parameter to approximate reactor power and effect the actual trip bypass. The Reactor Protection System (RPS) design purposely chooses this parameter, as opposed to the more direct measurement of power such as neutron flux, in order to assure diversity between the Turbine Stop Valve Closure (TSVC) and Turbine Control Valve Fast Closure (TCVFC) scram functions and the neutron flux scram function.

Four pressure sensors are connected to the high pressure turbine via instrument lines to monitor either turbine bowl pressure or turbine first stage shell pressure. The choice is dependent upon characteristics of turbine operation. The pressure being monitored varies essentially linearly with turbine throttle flow over the full range of turbine operation.

The turbine first stage pressure sensors are used for an indirect measurement of reactor power. They provide an automatic bypass of the TSVC and TCVFC reactor scram and End-of-Cycle Recirculation Pumps Trip functions at low reactor power levels where an immediate scram from closed or closing turbine valves is not required. The trip bypass is in effect whenever turbine first stage pressure is below a specified value, and thus permits continued reactor operation below that setpoint with the turbine stop valves and/or turbine control valve closed. In the event of inadequate turbine bypass capacity or failure of the turbine bypass system, adequate reactor protection is provided by the diverse high reactor dome pressure or high neutron flux scram functions.

The proposed change is based on test data which provided the actual value of Turbine First Stage Pressure equivalent to 40% rated thermal power. The Setpoints and Allowable Values were calculated for normal operation and for reduced feedwater heating operation. The purpose of this setpoint is to establish when a turbine trip will cause an anticipatory reactor trip, and an end-of-cycle recirculation pump trip as described in various sections of the Updated Safety Analysis Report. Turbine first stage pressure is used to sense power since this pressure increases approximately linearly proportional with power. The existing technical specifications have established the Setpoint and Allowable Value as a percentage of calibration span of the existing instruments. The proposed Setpoint and Allowable Value are in units of pressure. In both cases the Setpoint and Allowable Value are based on 40% of rated thermal power. The licensee performed testing during the Startup Testing Program which provided actual data for First Stage Pressure at 40% of rated thermal power. The proposed setpoint of 212 psig (corresponding to 29.5%) for normal feedwater heating conditions, versus the current setpoint of 25.4% of span, provides additional margin against inadvertent scrams at low power, while still providing for initiation of the turbine stop and control valve closure anticipatory reactor scrams and the end-of-cycle recirculation pump trip functions prior to exceeding 40% of rated power.

The basis for establishing the Trip Setpoint and Allowable Values (40% of rated thermal power) has not changed. Test data has accurately determined the value of turbine first stage pressure equivalent to 40% of rated thermal power. The Trip Setpoint and Allowable Value are being changed based on this test data. The staff has determined that the proposed change more accurately represents the Setpoint and Allowable Value for turbine first stage pressure and, therefore, it is acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.21, 51.32, and 51.35 an Environmental Assessment and Finding of No Significant Impact has been prepared and published in the FEDERAL REGISTER on May 7, 1990 (55 FR 18990). Accordingly, based on the environmental assessment, the Commission has determined that the issuance of the amendment will not have a significant effect on the quality of the human environment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Rhow, NRR/SICB

Dated: May 7, 1990

7590-01

UNITED STATES NUCLEAR REGULATORY COMMISSION

THE CLEVELAND ELECTRICAL ILLUMINATING COMPANY, ET Al.

DOCKET NO. 50-440

NOTICE OF ISSUANCE OF AMENDMENTS TO

FACILITY OPERATING LICENSES

The U.S. Nuclear Regulatory Commission (Commission) has issued Amendment No. to Facility Operating License No. NPF-58, issued to the Cleveland Electric Illuminating Company, Duquesne Light Company, Ohio Edison Company, Pennsylvania Power Company, and Toledo Edison Company (the licensees), which revised the Technical Specifications for operation of the Perry Nuclear Power Plant Unit No. 1, located in Lake County, Ohio. The amendment was effective as of the date of issuance.

The amendment modified the Technical Specifications (TS) to revise the Setpoint and Allowable Values for the Turbine First Stage Pressure Trip in notes (h) and (b) of Tables 3.3.1-1 and 3.3.4.2-1 of the TS.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

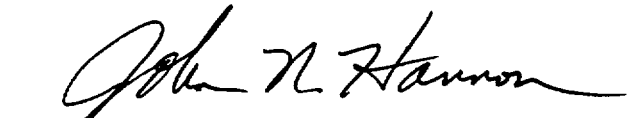
Notice of Consideration of Issuance of Amendment and Opportunity for Hearing in connection with this action was published in the FEDERAL REGISTER on March 30, 1990 (55 FR 12075). No request for a hearing or petition for leave to intervene was filed following this notice.

The Commission has prepared an Environmental Assessment related to the action and has determined not to prepare an environmental impact statement. Based upon the environmental assessment, the Commission has concluded that the issuance of this amendment will not have a significant effect on the quality of the human environment.

For further details with respect to the action see (1) the application for amendment dated May 20, 1988, as supplemented November 27, 1989, (2) Amendment No. 29 to License No. NPF-58, (3) the Commission's related Safety Evaluation dated May 7, 1990, and (4) the Environmental Assessment dated April 27, 1990 (55 FR 18990). All of these items are available for public inspection at the Commission's Public Document Room, Gelman Building, 2120 L Street N.W., and at the Perry Public Library, 3753 Main Street, Perry, Ohio 44081. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Reactor Projects III, IV, V and Special Projects.

Dated at Rockville, Maryland this 7th day of May 1990.

FOR THE NUCLEAR REGULATORY COMMISSION



John N. Hannon, Director
Project Directorate III-3
Division of Reactor Projects - III,
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Office of Nuclear Reactor Regulation