

May 24, 1994

Docket Nos. STN 50-456

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E. Murphy	J. Strosnider

Mr. D. L. Farrar
 Manager, Nuclear Regulatory Services
 Commonwealth Edison Company
 Executive Towers West III, Suite 500
 1400 OPUS Place
 Downers Grove, Illinois 60515

Dear Mr. Farrar:

SUBJECT: CORRECTION TO ISSUANCE OF AMENDMENT (TAC NO. M89166)

On May 7, 1994, the Commission issued Amendment No. 50 to Facility Operating License No. NPF-72 for the Braidwood Station, Unit No. 1.

Four of the Technical Specifications (TS) pages included in the amendment contained an error. Specifically, TSs pages 3/4 4-13, 3/4 4-15, 3/4 4-18, and 3/4 4-19 for Braidwood, Unit 1, should not have referenced the amendment number. Because you had submitted these pages for clarification only, these pages did not require or contain any changes. Therefore, these pages were not amended and should not have referenced the amendment number. Please substitute the enclosed replacement pages for the corresponding pages previously issued. Additionally, a revised instruction page is enclosed to correctly identify the amended TS pages.

We regret any inconvenience this error may have created.

Sincerely,

Original Signed By:

Ramin R. Assa, Acting Project Manager
 Project Directorate III-2
 Division of Reactor Projects - III/IV
 Office of Nuclear Reactor Regulation

9406100218 940524
 PDR ADDCK 05000456
 P PDR

Enclosures:
 Instruction page
 TS pages 3/4 4-13, 3/4 4-15,
 3/4 4-18, and 3/4 4-19

cc w/enclosures:
 See next page

OFC	LA:PDIII-2	SPE:PDIII-2	PM:PDIII-2	D:PDIII-2		
NAME	CHAWES CMH	MBLYNCH	RASSA RA	JDYER JH		
DATE	5/24/94	05/24/94	5/24/94	5/24/94	1 / 94	1 / 94
COPY	(YES/NO)	(YES/NO)	(YES/NO)	(YES/NO)	YES/NO	YES/NO

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cc w/enclosures:
 See next page

OFC	LA:PDIII-2	SPE:PDIII-2	PM:PDIII-2	D:PDIII-2		
NAME	CHAWES <i>CMH</i>	MDLYNCH <i>MDL</i>	RASSA <i>RA</i>	JDYER <i>JM</i>		
DATE	5/24/94	05/24/94	5/24/94	5/24/94	1 / 94	1 / 94
COPY	(YES/NO)	(YES/NO)	(YES/NO)	YES/NO	YES/NO	YES/NO

Mr. D. L. Farrar
Commonwealth Edison Company

Braidwood Station
Unit 1

cc:

Mr. William P. Poirier
Westinghouse Electric Corporation
Energy Systems Business Unit
Post Office Box 355, Bay 236 West
Pittsburgh, Pennsylvania 15230

Chairman
Will County Board of Supervisors
Will County Board Courthouse
Joliet, Illinois 60434

Joseph Gallo
Gallo & Ross
1250 Eye St., N.W., Suite 302
Washington, D.C. 20005

Ms. Lorraine Creek
Rt. 1, Box 182
Manteno, Illinois 60950

Regional Administrator
U. S. NRC, Region III
801 Warrenville Road
Lisle, Illinois 60532-4351

Attorney General
500 South 2nd Street
Springfield, Illinois 62701

Ms. Bridget Little Rorem
Appleseed Coordinator
117 North Linden Street
Essex, Illinois 60935

Michael Miller, Esquire
Sidley and Austin
One First National Plaza
Chicago, Illinois 60690

Mr. Edward R. Crass
Nuclear Safeguards and Licensing
Division
Sargent & Lundy Engineers
55 East Monroe Street
Chicago, Illinois 60603

George L. Edgar
Newman & Holtzinger, P.C.
1615 L Street, N.W.
Washington, D.C. 20036

U. S. Nuclear Regulatory Commission
Resident Inspectors Office
Rural Route #1, Box 79
Braceville, Illinois 60407

Illinois Dept. of Nuclear Safety
Office of Nuclear Facility Safety
1035 Outer Park Drive
Springfield, Illinois 62704

Mr. Ron Stephens
Illinois Emergency Services
and Disaster Agency
110 East Adams Street
Springfield, Illinois 62706

Commonwealth Edison Company
Braidwood Station Manager
Rt. 1, Box 84
Braceville, Illinois 60407

Howard A. Learner
Environmental Law and Policy
Center of the Midwest
203 North LaSalle Street
Suite 1390
Chicago, Illinois 60601

EIS Review Coordinator
U.S. Environmental Protection Agency
77 W. Jackson Blvd.
Chicago, Illinois 60604-3590

ATTACHMENT TO LICENSE AMENDMENT NO. 50

FACILITY OPERATING LICENSE NO. NPF-72

DOCKET NO. STN 50-456

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

<u>Remove Pages</u>	<u>Insert Pages</u>
3/4 4-14	3/4 4-14
-	3/4 4-14a
3/4 4-16	3/4 4-16
3/4 4-17	3/4 4-17
-	3/4 4-17a
-	3/4 4-17b
3/4 4-21	3/4 4-21
3/4 4-27	3/4 4-27
B 3/4 4-3	B 3/4 4-3
-	B 3/4 4-3a
B 3/4 4-4	B 3/4 4-4

REACTOR COOLANT SYSTEM

3/4.4.5 STEAM GENERATORS

LIMITING CONDITION FOR OPERATION

3.4.5 Each steam generator shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one or more steam generators inoperable, restore the inoperable steam generator(s) to OPERABLE status prior to increasing T_{avg} above 200°F.

SURVEILLANCE REQUIREMENTS

4.4.5.0 Each steam generator shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

4.4.5.1 Steam Generator Sample Selection and Inspection - Each steam generator shall be determined OPERABLE during shutdown by selecting and inspecting at least the minimum number of steam generators specified in Table 4.4-1.

4.4.5.2 Steam Generator Tube* Sample Selection and Inspection - The steam generator tube minimum sample size, inspection result classification, and the corresponding action required shall be as specified in Table 4.4-2. The inservice inspection of steam generator tubes shall be performed at the frequencies specified in Specification 4.4.5.3 and the inspected tubes shall be verified acceptable per the acceptance criteria of Specification 4.4.5.4. When applying the expectations of 4.4.5.2.a through 4.4.5.2.c, previous defects or imperfections in the area repaired by the sleeve are not considered an area requiring reinspection. The tubes selected for each inservice inspection shall include at least 3% of the total number of tubes in all steam generators; the tubes selected for these inspections shall be selected on a random basis except:

- a. Where experience in similar plants with similar water chemistry indicates critical areas to be inspected, then at least 50% of the tubes inspected shall be from these critical areas;
- b. The first sample of tubes selected for each inservice inspection (subsequent to the preservice inspection) of each steam generator shall include:

*When referring to a steam generator tube, the sleeve shall be considered a part of the tube if the tube has been repaired per Specification 4.4.5.4.a.10.

SURVEILLANCE REQUIREMENTS (Continued)

4.4.5.3 Inspection Frequencies - The above required inservice inspections of steam generator tubes shall be performed at the following frequencies:

- a. The first inservice inspection shall be performed after 6 Effective Full Power Months but within 24 calendar months of initial criticality. Subsequent inservice inspections shall be performed at intervals of not less than 12 nor more than 24 calendar months after the previous inspection. If two consecutive inspections, not including the preservice inspection, result in all inspection results falling into the C-1 category or if two consecutive inspections demonstrate that previously observed degradation has not continued and no additional degradation has occurred, the inspection interval may be extended to a maximum of once per 40 months;
- b. If the results of the inservice inspection of a steam generator conducted in accordance with Table 4.4-2 at 40-month intervals fall in Category C-3, the inspection frequency shall be increased to at least once per 20 months. The increase in inspection frequency shall apply until the subsequent inspections satisfy the criteria of Specification 4.4.5.3a.; the interval may then be extended to a maximum of once per 40 months; and
- c. Additional, unscheduled inservice inspections shall be performed on each steam generator in accordance with the first sample inspection specified in Table 4.4-2 during the shutdown subsequent to any of the following conditions:
 - 1) Reactor-to-secondary tube leaks (not including leaks originating from tube-to-tube sheet welds) in excess of the limits of Specification 3.4.6.2c., or
 - 2) A seismic occurrence greater than the Operating Basis Earthquake, or
 - 3) A Condition IV loss-of-coolant accident requiring actuation of the Engineered Safety Features, or
 - 4) A Condition IV main steam line or feedwater line break.

TABLE 4.4-1
MINIMUM NUMBER OF STEAM GENERATORS TO BE
INSPECTED DURING INSERVICE INSPECTION

Preservice Inspection	Yes
No. of Steam Generators per Unit	Four
First Inservice Inspection	Two
Second & Subsequent Inservice Inspections	One ¹

TABLE NOTATION

1. The inservice inspection may be limited to one steam generator on a rotating schedule encompassing 3 N % of the tubes (where N is the number of steam generators in the plant) if the results of the first or previous inspections indicate that all steam generators are performing in a like manner. Note that under some circumstances, the operating conditions in one or more steam generators may be found to be more severe than those in other steam generators. Under such circumstances the sample sequence shall be modified to inspect the most severe conditions. Each of the other two steam generators not inspected during the first inservice inspections shall be inspected during the second and third inspections. The fourth and subsequent inspections shall follow the instructions described above.

TABLE 4.4.2

STEAM GENERATOR TUBE INSPECTION

1ST SAMPLE INSPECTION			2ND SAMPLE INSPECTION		3RD SAMPLE INSPECTION	
Sample Size	Result	Action Required	Result	Action Required	Result	Action Required
A minimum of S Tubes per S. G.	C-1	None	N.A.	N.A.	N.A.	N.A.
	C-2	Plug or repair defective tubes and inspect additional 2S tubes in this S. G.	C-1	None	N.A.	N.A.
			C-2	Plug or repair defective tubes and inspect additional 4S tubes in this S. G.	C-1	None
			C-3	Perform action for C-3 result of first sample	C-2	Plug or repair defective tubes
			All other S. G.s are C-1	None	C-3	Perform action for C-3 result of first sample
	C-3	Inspect all tubes in this S. G., plug or repair defective tubes and inspect 2S tubes in each other S. G. Notification to NRC pursuant to §50.72 (b)(2) of 10 CFR Part 50	Some S. G.s C-2 but no additional S. G. are C-3	Perform action for C-2 result of second sample	N.A.	N.A.
			Additional S. G. is C-3	Inspect all tubes in each S. G. and plug or repair defective tubes. Notification to NRC pursuant to §50.72(b)(2) of 10 CFR Part 50	N.A.	N.A.

S = $\frac{N}{3} \times$ Where N is the number of steam generators in the unit, and n is the number of steam

REFERENCES - UNITS 1 & 2

3/4 4-19

AMENDMENT NO. 50

S1