

August 3, 1999

Mr. C. Lance Terry  
Senior Vice President  
& Principal Nuclear Officer  
TU Electric  
Attn: Regulatory Affairs Department  
P. O. Box 1002  
Glen Rose, TX 76043

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION, UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: SURVEILLANCE REQUIREMENT CHANGES AND EDITORIAL CORRECTIONS (TAC NOS. MA5418 AND MA5419)

Dear Mr. Terry:

The Commission has issued the enclosed Amendment No. 66 to Facility Operating License No. NPF-87 and Amendment No. 66 to Facility Operating License No. NPF-89 for the Comanche Peak Steam Electric Station, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated May 4, 1999, as supplemented by letter dated June 4, 1999.

The amendments correct a number of editorial errors in the TSs that occurred with the issuance of License Amendment No. 64 regarding the improved Technical Specifications conversion. In addition, a Surveillance Requirement (SR) 3.8.4.7 is changed to allow the substitution of a modified performance discharge test, for a service test, for the 125 VDC batteries and SRs 3.8.1.7, 3.8.1.12, 3.8.1.15, and 3.8.1.20 are revised to separate the voltage and frequency acceptance criteria for the diesel generator start surveillances into two sets of criteria; those criteria required to be met within 10 seconds, and those criteria required to be met following achievement of steady state conditions.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

**REG FILE CENTER COPY**

ORIG. SIGNED BY  
David H. Jaffe, Senior Project Manager, Section 1  
Project Directorate IV & Decommissioning  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

cc w/encl: See next page

Enclosures:

- 1. Amendment No. 66 to NPF-87
- 2. Amendment No. 66 to NPF-89
- 3. Safety Evaluation

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*Handwritten initials: J, P, G*

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**\*SEE PREVIOUS CONCURRENCE**

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NAME	DJaffe, db	LBerry	WBeckner	JCalvo		Comanche	RGramm
DATE	7/15/99	7/15/99	7/16/99	7/9/99		7/26/99	8/2/99

DOCUMENT NAME: G:\PDIV-1\ComanchePeak\AMDA5418.WPD

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PDR ADDCK 05000445  
P PDR

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 Senior Vice President  
 & Principal Nuclear Officer  
 TU Electric  
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 P. O. Box 1002  
 Glen Rose, TX 76043

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The amendments correct a number of editorial errors in the TSs that occurred with the issuance of License Amendment No. 64 regarding the improved Technical Specifications conversion. In addition, Surveillance Requirement 3.8.4.7 is changed to allow the substitution of a modified performance discharge test, for a service test, for the 125 VDC batteries. The remaining issues associated with the May 4, 1999, application will be addressed in future correspondence.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

David H. Jaffe, Senior Project Manager, Section 1  
 Project Directorate IV & Decommissioning  
 Division of Licensing Project Management  
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Docket Nos. 50-445 and 50-446

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NAME	DJaffe:db	LBerry	WBeckner	JCalvo		RGramm	
DATE	7/8/99	7/17/99	7/16/99	7/9/99	1/99	1/99	

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

August 3, 1999

Mr. C. Lance Terry  
Senior Vice President  
& Principal Nuclear Officer  
TU Electric  
Attn: Regulatory Affairs Department  
P. O. Box 1002  
Glen Rose, TX 76043

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The amendments correct a number of editorial errors in the TSs that occurred with the issuance of License Amendment No. 64 regarding the improved Technical Specifications conversion. In addition, a Surveillance Requirement (SR) 3.8.4.7 is changed to allow the substitution of a modified performance discharge test, for a service test, for the 125 VDC batteries and SRs 3.8.1.7, 3.8.1.12, 3.8.1.15, and 3.8.1.20 are revised to separate the voltage and frequency acceptance criteria for the diesel generator start surveillances into two sets of criteria; those criteria required to be met within 10 seconds, and those criteria required to be met following achievement of steady state conditions.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink, appearing to read "D. Jaffe", with a long horizontal line extending to the right.

David H. Jaffe, Senior Project Manager, Section 1  
Project Directorate IV & Decommissioning  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosures: 1. Amendment No. 66 to NPF-87  
2. Amendment No. 66 to NPF-89  
3. Safety Evaluation

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Comanche Peak Steam Electric Plant

cc:

Senior Resident Inspector  
U.S. Nuclear Regulatory Commission  
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Glen Rose, TX 76403-2159

Regional Administrator, Region IV  
U.S. Nuclear Regulatory Commission  
611 Ryan Plaza Drive, Suite 400  
Arlington, TX 76011

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Citizens Association for Sound Energy  
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Dallas, TX 75224

Mr. Roger D. Walker  
Regulatory Affairs Manager  
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George L. Edgar, Esq.  
Morgan, Lewis & Bockius  
1800 M Street, N.W.  
Washington, DC 20036-5869

Honorable Dale McPherson  
County Judge  
P. O. Box 851  
Glen Rose, TX 76043

Office of the Governor  
ATTN: John Howard, Director  
Environmental and Natural  
Resources Policy  
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Austin, TX 78711

Arthur C. Tate, Director  
Division of Compliance & Inspection  
Bureau of Radiation Control  
Texas Department of Health  
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Jim Calloway  
Public Utility Commission of Texas  
Electric Industry Analysis  
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Austin, TX 78711-3326



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

TEXAS UTILITIES ELECTRIC COMPANY  
COMANCHE PEAK STEAM ELECTRIC STATION, UNIT 1

DOCKET NO. 50-445

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 66  
License No. NPF-87

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Texas Utilities Electric Company (TU Electric, the licensee) dated May 4, 1999, as supplemented by letter dated June 4, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-87 is hereby amended to read as follows:

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PDR ADOCK 05000445  
P PDR

2. Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 66 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Gramm, Chief, Section 1  
Project Directorate IV & Decommissioning  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: August 3, 1999



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

TEXAS UTILITIES ELECTRIC COMPANY  
COMANCHE PEAK STEAM ELECTRIC STATION, UNIT 2

DOCKET NO. 50-446

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 66  
License No. NPF-89

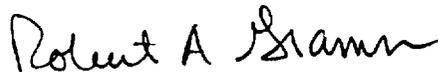
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Texas Utilities Electric Company (TU Electric, the licensee) dated May 4, 1999, as supplemented by letter dated June 4, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-89 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 66 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. TU Electric shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Gramm, Chief, Section 1  
Project Directorate IV & Decommissioning  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: August 3, 1999

ATTACHMENT TO LICENSE AMENDMENT NOS. 66 AND 66

FACILITY OPERATING LICENSE NOS. NPF-87 AND NPF-89

DOCKET NOS. 50-445 AND 50-446

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
3.0-5	3.0-5
3.1-19	3.1-19
3.2-6	3.2-6
3.2-13	3.2-13
3.3-11	3.3-11
3.3-45	3.3-45
3.4-24	3.4-24
3.4-25	3.4-25
3.4-38	3.4-38
3.7-23	3.7-23
3.7-27	3.7-27
3.7-30	3.7-30
3.8-8	3.8-8
3.8-11	3.8-11
3.8-12	3.8-12
3.8-16	3.8-16
3.8-21	3.8-21
3.8-25	3.8-25
3.8-26	3.8-26
3.8-34	3.8-34
3.9-9	3.9-9
3.9-10	3.9-10
4.0-2	4.0-2
5.0-27	5.0-27
5.0-29	5.0-29

3.1 REACTIVITY CONTROL SYSTEMS

3.1.8 PHYSICS TESTS Exceptions — MODE 2

LCO 3.1.8 During the performance of PHYSICS TESTS, the requirements of

LCO 3.1.3, "Moderator Temperature Coefficient (MTC)";  
LCO 3.1.4, "Rod Group Alignment Limits";  
LCO 3.1.5, "Shutdown Bank Insertion Limits";  
LCO 3.1.6, "Control Bank Insertion Limits"; and  
LCO 3.4.2, "RCS Minimum Temperature for Criticality"

may be suspended, provided:

- a. RCS lowest operating loop average temperature is  $\geq 541^{\circ}\text{F}$ ; and
- b. SDM is within the limits provided in the COLR; and
- c. THERMAL POWER is  $\leq 5\%$  RTP

APPLICABILITY: MODE 2 during PHYSICS TESTS.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. SDM not within limit.	A.1 Initiate boration to restore SDM to within limit.	15 minutes
	<u>AND</u> A.2 Suspend PHYSICS TESTS exceptions.	1 hour
B. THERMAL POWER not within limit.	B.1 Open reactor trip breakers.	Immediately

(continued)

3.0 SR APPLICABILITY (continued)

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SR 3.0.3

If it is discovered that a Surveillance was not performed within its specified Frequency, then compliance with the requirement to declare the LCO not met may be delayed, from the time of discovery, up to 24 hours or up to the limit of the specified Frequency, whichever is less. This delay period is permitted to allow performance of the Surveillance.

If the Surveillance is not performed within the delay period, the LCO must immediately be declared not met, and the applicable Condition(s) must be entered.

When the Surveillance is performed within the delay period and the Surveillance is not met, the LCO must immediately be declared not met, and the applicable Condition(s) must be entered.

---

SR 3.0.4

Entry into a MODE or other specified condition in the Applicability of an LCO shall not be made unless the LCO's Surveillances have been met within their specified Frequency. This provision shall not prevent entry into MODES or other specified conditions in the Applicability that are required to comply with ACTIONS or that are part of a shutdown of the unit.

SR 3.0.4 is only applicable for entry into a MODE or other specified condition in the Applicability in MODES 1, 2, 3, and 4.

---

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ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>A. (continued)</p>	<p>A.4     Reevaluate safety analyses and confirm results remain valid for duration of operation under this condition.</p> <p><u>AND</u></p> <p>A.5     -----NOTES-----            1. Perform Required Action A.5 only after Required Action A.4 is completed.             2. Required action A.6 shall be completed whenever Required Action A.5 is performed.</p> <p>-----</p> <p>Normalize excore detectors to restore QPTR to within limit.</p> <p><u>AND</u></p>	<p>Prior to increasing THERMAL POWER above the limit of Required Action A.1</p> <p>Prior to increasing THERMAL POWER above the limit of Required Action A.1</p> <p>(continued)</p>

**SURVEILLANCE REQUIREMENTS (continued)**

SURVEILLANCE	FREQUENCY
<p>SR 3.3.1.5 Perform ACTUATION LOGIC TEST.</p>	<p>31 days on a STAGGERED TEST BASIS</p>
<p>SR 3.3.1.6 -----NOTE-----            Not required to be performed until 72 hours after achieving equilibrium conditions with THERMAL POWER <math>\geq</math> 75 % RTP.            -----            Calibrate excore channels to agree with incore detector measurements.</p>	<p>92 EFPD</p>
<p>SR 3.3.1.7 -----NOTES-----            1. Not required to be performed for source range instrumentation prior to entering MODE 3 from MODE 2 until 4 hours after entry into MODE 3.            2. Source range instrumentation shall include verification that interlocks P-6 and P-10 are in their required state for existing unit conditions.            -----            Perform COT.</p>	<p>92 days</p>

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>D. Two channels per bus for the 6.9 kV bus undervoltage function inoperable.</p>	<p>D.1 Restore one channel per bus to OPERABLE status.</p> <p><u>OR</u></p> <p>D.2 Declare the affected A.C. emergency buses inoperable.</p>	<p>1 hour</p> <p>1 hour</p>
<p>E. Two channels per bus for one or more degraded voltage or low grid undervoltage function inoperable</p>	<p>E.1 Restore one channel per bus to OPERABLE status.</p> <p><u>OR</u></p> <p>E.2.1 Declare both offsite power source buses inoperable.</p> <p><u>AND</u></p> <p>E.2.2 Open offsite power source breakers to the associated buses.</p>	<p>1 hour</p> <p>1 hour</p> <p>6 hours</p>
<p>F. One or more Automatic Actuation Logic and Actuation Relays trains inoperable.</p>	<p>F.1 Restore train(s) to OPERABLE status.</p>	<p>1 hour</p>
<p>G. Required Action and associated Completion Time not met.</p>	<p>G.1 Enter applicable Condition(s) and Required Action(s) for the associated DG made inoperable by LOP DG start instrumentation.</p>	<p>Immediately</p>

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>C. One block valve inoperable.</p>	<p>-----NOTE----- Required Actions do not apply when block valve is inoperable solely as a result of complying with Required Actions B.2 or E.2. -----</p> <p>C.1 Place associated PORV in manual control.</p> <p><u>AND</u></p> <p>C.2 Restore block valve to OPERABLE status.</p>	<p>1 hour</p> <p>72 hours</p>
<p>D. Required Action and associated Completion Time of Condition A, B, or C not met.</p>	<p>D.1 Be in MODE 3.</p> <p><u>AND</u></p> <p>D.2 Be in MODE 4</p>	<p>6 hours</p> <p>12 hours</p>

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>E. Two PORVs inoperable and not capable of being manually cycled.</p>	<p>E.1 Close associated block valves.</p>	<p>1 hour</p>
	<p><u>AND</u></p>	
	<p>E.2 Remove power from associated block valves.</p>	<p>1 hour</p>
	<p><u>AND</u></p>	
	<p>E.3 Be in MODE 3</p>	<p>6 hours</p>
<p>F. More than one block valve inoperable.</p>	<p>-----NOTE----- Required Actions do not apply when block valve is inoperable solely as a result of complying with Required Actions B.2 or E.2. -----</p>	
	<p>F.1 Place associated PORVs in manual control.</p>	<p>1 hour</p>
	<p><u>AND</u> F.2 Restore one block valve to OPERABLE status</p>	<p>2 hours</p>

(continued)

**SURVEILLANCE REQUIREMENTS**

SURVEILLANCE	FREQUENCY
<p>SR 3.4.14.1</p> <p style="text-align: center;">-----NOTES-----</p> <ol style="list-style-type: none"> <li>1. Not required to be performed in MODES 3 and 4.</li> <li>2. Not required to be performed on the RCS PIVs located in the RHR flow path when in the shutdown cooling mode of operation.</li> <li>3. RCS PIVs actuated during the performance of this Surveillance are not required to be tested more than once if a repetitive testing loop cannot be avoided.</li> </ol> <hr/> <p>Verify leakage from each RCS PIV is equivalent to <math>\leq 0.5</math> gpm per nominal inch of valve size up to a maximum of 5 gpm at an RCS pressure <math>\geq 2215</math> psig and <math>\leq 2255</math> psig.</p>	<p>In accordance with the Inservice Testing Program, and 18 months</p> <p><u>AND</u></p> <p>Prior to entering MODE 2 whenever the unit has been in MODE 5 for 7 days or more, and if leakage testing has not been performed in the previous 9 months except for valves 8701A, 8701B, 8702A and 8702B</p> <p><u>AND</u></p> <p style="text-align: right;">(continued)</p>

3.7 PLANT SYSTEMS

3.7.10 Control Room Emergency Filtration/Pressurization System (CREFS)

LCO 3.7.10 Two CREFS trains shall be OPERABLE

-----NOTE-----  
The Control Room boundary may be opened intermittently under administrative controls.  
-----

APPLICABILITY: MODES 1, 2, 3, 4, 5, and 6,  
During movement of irradiated fuel assemblies

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One CREFS train inoperable.	A.1 Restore CREFS train to OPERABLE status.	7 days
B. Two CREFS Trains inoperable due to inoperable Control Room boundary in MODES 1, 2, 3, and 4.	B.1 Restore control room boundary to OPERABLE status.	24 hours
C. Required Action and associated Completion Time of Condition A or B not met in MODE 1, 2, 3, or 4.	C.1 Be in MODE 3.	6 hours
	<u>AND</u> C.2 Be in MODE 5.	36 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>D. Two CRACS trains inoperable in MODE 5 or 6, or during movement of irradiated fuel assemblies.</p>	<p>D.1.1 Verify at least 100% of the required heat removal capability equivalent to a single OPERABLE train available.</p> <p><u>AND</u></p> <p>D.1.2 Restore the CRACS trains to OPERABLE status.</p> <p><u>OR</u></p> <p>D.2.1 Suspend CORE ALTERATIONS.</p> <p><u>AND</u></p> <p>D.2.2 Suspend movement of irradiated fuel assemblies.</p>	<p>Immediately</p> <p>30 days</p> <p>Immediately</p> <p>Immediately</p>
<p>E. Two CRACS trains inoperable in MODE 1, 2, 3, or 4.</p>	<p>E.1.1 Verify at least 100% of the required heat removal capability equivalent to a single OPERABLE train available.</p> <p><u>AND</u></p> <p>E.1.2 Restore one CRACS train to OPERABLE status.</p> <p><u>OR</u></p> <p>E.2 Enter LCO 3.0.3.</p>	<p>Immediately</p> <p>30 days</p> <p>Immediately</p>

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. Required Actions and associated Completion Times not met.	D.1 Be in MODE 3.	6 hours
	<u>AND</u> D.2 Be in MODE 5.	36 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.7.12.1 Operate each ESF Filtration train for $\geq 10$ continuous hours with the heaters operating.	31 days
SR 3.7.12.2 Perform required ESF Filtration Unit filter testing in accordance with the Ventilation Filter Testing Program (VFTP).	In accordance with the VFTP
SR 3.7.12.3 Verify each PPVS train actuates on an actual or simulated actuation signal.	18 months
SR 3.7.12.4 Verify one PPVS train can maintain a pressure $\leq -0.05$ inches water gauge relative to atmospheric pressure during the post accident mode of operation.	18 months on a STAGGERED TEST BASIS

(continued)

**SURVEILLANCE REQUIREMENTS (continued)**

SURVEILLANCE	FREQUENCY
SR 3.8.1.4 Verify each day tank contains $\geq$ 1440 gal of fuel oil.	31 days
SR 3.8.1.5 Check for and remove accumulated water from each day tank.	31 days
SR 3.8.1.6 Verify the fuel oil transfer system operates to automatically transfer fuel oil from storage tank to the day tank.	92 days
<p>SR 3.8.1.7 -----NOTE----- All DG starts may be preceded by an engine prelube period.</p> <p>-----</p> <p>Verify each DG starts from standby condition and achieves:</p> <p>a. in <math>\leq</math> 10 seconds, voltage <math>\geq</math> 6480 V and frequency <math>\geq</math> 58.8 Hz; and</p> <p>b. steady state, voltage <math>\geq</math> 6480 V and <math>\leq</math> 7150 V, and frequency <math>\geq</math> 58.8 Hz and <math>\leq</math> 61.2 Hz.</p>	184 days
<p>SR 3.8.1.8 -----NOTE----- Verify requirement during MODES 3, 4, 5, 6 or with core off-loaded.</p> <p>-----</p> <p>Verify automatic and manual transfer of AC power sources from the normal offsite circuit to each alternate required offsite circuit.</p>	18 months

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.8.1.12</p> <p>-----NOTES-----</p> <ol style="list-style-type: none"> <li>1. All DG starts may be preceded by prelube period.</li> <li>2. Verify requirement during MODES 3, 4, 5, 6 or with core off-loaded.</li> </ol> <p>-----</p> <p>Verify on an actual or simulated Safety Injection (SI) actuation signal each DG auto-starts from standby condition and:</p> <ol style="list-style-type: none"> <li>a. In <math>\leq 10</math> seconds after auto-start and during tests, achieves voltage <math>\geq 6480</math> V and frequency <math>\geq 58.8</math> Hz;</li> <li>b. Achieves steady state voltage <math>\geq 6480</math> V and <math>\leq 7150</math> V and frequency <math>\geq 58.8</math> Hz and <math>\leq 61.2</math> Hz;</li> <li>c. Operates for <math>\geq 5</math> minutes;</li> </ol>	<p>18 months</p>
<p>SR 3.8.1.13</p> <p>-----NOTE-----</p> <p>Verify requirement during MODES 3, 4, 5, 6 or with core off-loaded.</p> <p>-----</p> <p>Verify each DG's automatic trips are bypassed on actual or simulated loss of voltage signal on the emergency bus concurrent with an actual or simulated SI actuation signal except:</p> <ol style="list-style-type: none"> <li>a. Engine overspeed; and</li> <li>b. Generator differential current.</li> </ol>	<p>18 months</p>

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.8.1.14</p> <p style="text-align: center;">-----NOTES-----</p> <ol style="list-style-type: none"> <li>1. Momentary transients outside the load and power factor ranges do not invalidate this test.</li> <li>2. Verify requirement during MODES 3, 4, 5, 6 or with core off-loaded.</li> </ol> <p style="text-align: center;">-----</p> <p>Verify each DG operates for <math>\geq 24</math> hours:</p> <ol style="list-style-type: none"> <li>a. For <math>\geq 2</math> hours loaded <math>\geq 6900</math> kW and <math>\leq 7700</math> kW; and</li> <li>b. For the remaining hours of the test loaded <math>\geq 6300</math> kW and <math>\leq 7000</math> kW.</li> </ol>	<p>18 months</p>
<p>SR 3.8.1.15</p> <p style="text-align: center;">-----NOTES-----</p> <ol style="list-style-type: none"> <li>1. This Surveillance shall be performed within 5 minutes of shutting down the DG after the DG has operated <math>\geq 2</math> hours loaded <math>\geq 6300</math> kW and <math>\leq 7000</math> kW. Momentary transients outside of load range do not invalidate this test.</li> <li>2. All DG starts may be preceded by an engine prelube period.</li> </ol> <p style="text-align: center;">-----</p> <p>Verify each DG starts and achieves:</p> <ol style="list-style-type: none"> <li>a. in <math>\leq 10</math> seconds, voltage <math>\geq 6480</math> V and frequency <math>\geq 58.8</math> Hz; and</li> <li>b. steady state, voltage <math>\geq 6480</math> V, and <math>\leq 7150</math> V and frequency <math>\geq 58.8</math> Hz and <math>\leq 61.2</math> Hz.</li> </ol>	<p>18 months</p>

(continued)

**SURVEILLANCE REQUIREMENTS (continued)**

SURVEILLANCE	FREQUENCY
<p>SR 3.8.1.20</p> <p>-----NOTE----- All DG starts may be preceded by an engine prelube period. -----</p> <p>Verify when started simultaneously from standby condition, each DG achieves:</p> <p>a. in <math>\leq 10</math> seconds, voltage <math>\geq 6480</math> V and frequency <math>\geq 58.8</math> Hz; and</p> <p>b. steady state, voltage <math>\geq 6480</math> V, and <math>\leq 7150</math> V and frequency <math>\geq 58.8</math> Hz and <math>\leq 61.2</math> Hz.</p>	<p>10 years</p>
<p>SR 3.8.1.21</p> <p>Calibrate BO sequencers.</p>	<p>18 months</p>
<p>SR 3.8.1.22</p> <p>-----NOTES----- 1. Verification of setpoint is not required. 2. Actuation of final devices is not included. -----</p> <p>Perform TADOT for SI and BO sequencers.</p>	<p>31 days on a STAGGERED TEST BASIS.</p>

3.8 ELECTRICAL POWER SYSTEMS

3.8.3 Diesel Fuel Oil, Lube Oil, and Starting Air

LCO 3.8.3 The stored diesel fuel oil, lube oil, and starting air subsystem shall be within limits for each required diesel generator (DG).

APPLICABILITY: When associated DG is required to be OPERABLE

ACTIONS

-----NOTE-----  
Separate Condition entry is allowed for each DG.  
-----

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more DGs with fuel level between 74,600 and 86,000 gal in MODES 1-4 or between 65,600 and 75,000 in MODES 5 & 6 in storage tank.	A.1 Restore fuel oil level to within limits.	48 hours
B. One or more DGs with lube oil inventory less than a level 1" below the low run level but greater than a level 1" above the end of the lube oil dipstick.	B.1 Restore lube oil inventory to within limits.	48 hours

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.8.4.2 Verify no visible corrosion at battery terminals and connectors.</p> <p><u>OR</u></p> <p>Verify battery connection resistance is <math>\leq 150 \times 10^{-6}</math> ohm for inter-cell connections, <math>\leq 150 \times 10^{-6}</math> ohm for inter-rack connections, <math>\leq 150 \times 10^{-6}</math> ohm for inter-tier connections, and <math>\leq 150 \times 10^{-6}</math> ohm for terminal connections.</p>	92 days
<p>SR 3.8.4.3 Verify battery cells, cell plates, and racks show no visual indication of physical damage or abnormal deterioration that could degrade battery performance.</p>	18 months
<p>SR 3.8.4.4 Remove visible terminal corrosion, verify battery cell to cell and terminal connections are clean and tight, and are coated with anti-corrosion material.</p>	18 months
<p>SR 3.8.4.5 Verify battery connection resistance is <math>\leq 150 \times 10^{-6}</math> ohm for inter-cell connections, <math>\leq 150 \times 10^{-6}</math> ohm for inter-rack connections, <math>\leq 150 \times 10^{-6}</math> ohm for inter-tier connections, and <math>\leq 150 \times 10^{-6}</math> ohm for terminal connections.</p>	18 months

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.8.4.6    Verify each battery charger supplies <math>\geq 300</math> amps at <math>\geq 130</math> V for <math>\geq 8</math> hours.</p>	<p>18 months</p>
<p>SR 3.8.4.7    -----NOTES-----</p> <ol style="list-style-type: none"> <li>1. The modified performance discharge test in SR 3.8.4.8 may be performed in lieu of the service test in SR 3.8.4.7.</li> <li>2. Verify requirement during MODES 3, 4, 5, 6 or with core off-loaded.</li> </ol> <p>-----</p> <p>Verify battery capacity is adequate to supply, and maintain in OPERABLE status, the required emergency loads for the design duty cycle when subjected to a battery service test.</p>	<p>18 months</p>

(continued)

3.8 ELECTRICAL POWER SYSTEMS

3.8.7 Inverters — Operating

LCO 3.8.7 The required Train A and Train B inverters shall be OPERABLE.

-----NOTE-----  
 Inverters may be disconnected from one DC bus for  $\leq$  24 hours to perform an equalizing charge on their associated common battery, provided:

- a. The associated AC vital bus(es) are energized; and
- b. All other AC vital buses are energized from their associated OPERABLE inverters.

-----

APPLICABILITY: MODES 1, 2, 3, and 4

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One required inverter inoperable.	A.1 -----NOTE----- Enter applicable Conditions and Required Actions of LCO 3.8.9, "Distribution Systems - Operating" with any vital bus de-energized. ----- Restore inverter to OPERABLE status.	24 hours

(continued)

3.9 REFUELING OPERATIONS

3.9.5 Residual Heat Removal (RHR) and Coolant Circulation High Water Level

LCO 3.9.5 One RHR loop shall be OPERABLE and in operation.

-----NOTE-----  
The required RHR loop may be removed from operation for  $\leq 1$  hour per 8 hour period, provided no operations are permitted that would cause reduction of the Reactor Coolant System boron concentration.  
-----

APPLICABILITY: MODE 6 with the water level  $\geq 23$  ft above the top of reactor vessel flange.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. RHR loop requirements not met.	A.1 Suspend operations involving a reduction in reactor coolant boron concentration.	Immediately
	<u>AND</u>	
	A.2 Suspend loading irradiated fuel assemblies in the core.	Immediately
	<u>AND</u>	
		(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. continued	A.3 Initiate action to satisfy RHR loop requirements.  <u>AND</u>  A.4 Close all containment penetrations providing direct access from containment atmosphere to outside atmosphere.	Immediately    4 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.9.5.1 Verify one RHR loop is in operation and circulating reactor coolant at a flow rate of $\geq 3800$ gpm.	12 hours

#### 4.0 DESIGN FEATURES (continued)

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### 4.3 Fuel Storage

#### 4.3.1 Criticality

4.3.1.1 The spent fuel storage racks are designed and shall be maintained with:

- a. Fuel assemblies having a maximum U-235 enrichment of 5.0 weight percent;
- b.  $k_{\text{eff}} \leq 0.95$  if fully flooded with unborated water, which includes an allowance for uncertainties as described in Section 4.3 of the FSAR;
- c. A nominal 9 inch center to center distance between fuel storage locations in high density storage racks;
- d. A nominal 16 inch center to center distance between fuel assemblies placed in low density fuel storage racks;
- e. New or partially spent fuel assemblies with a discharge burnup in the "2 out of 4 storage allowed" region of Figure 3.7.17-1 may be allowed restricted storage in a checkerboard pattern in high density fuel storage racks or unrestricted storage in low density fuel storage racks; and
- f. New or partially spent fuel assemblies with a discharge burnup in the "2 out of 4 storage not allowed" region of Figure 3.7.17-1 may be allowed restricted storage in an expanded checkerboard pattern (1 out of 4) in high density fuel storage racks or unrestricted storage in low density fuel storage racks.

(continued)

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5.5 Programs and Manuals (continued)

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5.5.16. Containment Leakage Rate Testing Program

- a. A program shall be established to implement the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program, dated September, 1995"
- b. The peak calculated containment internal pressure for the design basis loss of coolant accident,  $P_a$ , is 48.3 psig.
- c. The maximum allowable containment leakage rate,  $L_a$ , at  $P_a$ , shall be 0.10% of containment air weight per day.
- d. Leakage rate acceptance criteria are:
  1. Containment leakage rate acceptance criteria is  $\leq 1.0 L_a$ . During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria are  $< 0.60 L_a$  for the Type B and Type C tests and  $\leq 0.75 L_a$  for Type A tests;
  2. Air lock testing acceptance criteria are:
    - a) Overall air lock leakage rate is  $\leq 0.05 L_a$  when tested at  $\geq P_a$ .
    - b) For each door, leakage rate is  $\leq 0.01 L_a$  when pressurized to  $\geq P_a$ .
- e. The provision of SR 3.0.2 do not apply to the test frequencies specified in the Containment Leakage Rate Testing Program, with the exception of the containment ventilation isolation valves.
- f. The provisions of SR 3.0.3 are applicable to the Containment Leakage Rate Testing Program.

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(continued)

## 5.0 ADMINISTRATIVE CONTROLS

### 5.6 Reporting Requirements

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The following reports shall be submitted in accordance with 10 CFR 50.4.

#### 5.6.1 Occupational Radiation Exposure Report

-----NOTE-----  
A single submittal may be made for a multiple unit station. The submittal should combine sections common to all units at the station.  
-----

A tabulation on an annual basis of the number of station, utility, and other personnel (including contractors), for whom monitoring was performed, receiving an annual deep dose equivalent > 100 mrem and the associated collective deep dose equivalent (reported in person-rem) according to work and job functions (e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance, waste processing, and refueling). This tabulation supplements the requirements of 10 CFR 20.2206. The dose assignments to various duty functions may be estimated based on pocket ionization chamber, thermoluminescent dosimeter (TLD), electronic dosimeter, or film badge measurements. Small exposures totaling < 20 percent of the individual total dose need not be accounted for. In the aggregate, at least 80 percent of the total deep dose equivalent received from external sources should be assigned to specific major work functions. The report covering the previous calendar year shall be submitted by April 30 of each year.

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(continued)



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 66 AND 66 TO

FACILITY OPERATING LICENSE NOS. NPF-87 AND NPF-89

TEXAS UTILITIES ELECTRIC COMPANY

COMANCHE PEAK STEAM ELECTRIC STATION, UNITS 1 AND 2

DOCKET NOS. 50-445 AND 50-446

1.0 INTRODUCTION

By application dated May 4, 1999, as supplemented by letter dated June 4, 1999, Texas Utilities Electric Company (TU Electric/the licensee) requested changes to the Technical Specifications (TSs) for the Comanche Peak Steam Electric Station, Units 1 and 2. The proposed changes would revise the TSs to correct a number of editorial errors that occurred with issuance of License Amendment No. 64 dated February 26, 1999, which issued the improved TS (ITS). In addition, Surveillance Requirement (SR) 3.8.4.7 would be changed to allow the substitution of a modified performance discharge test, for a service test, for the 125 VDC batteries and SRs 3.8.1.7, 3.8.1.12, 3.8.1.15, and 3.8.1.20 would be revised to separate the voltage and frequency acceptance criteria for the diesel generator (DG) start surveillance into two sets of criteria; those criteria required to be met within 10 seconds, and those criteria required to be met following achievement of steady state conditions.

2.0 DISCUSSION AND EVALUATION

The licensee has identified the following editorial errors, which should be corrected:

- (1) ITS 3.0 - SR 3.0.3, third paragraph, would be revised to delete one occurrence of the phrase "Conditions(s) must be entered," which was inadvertently included twice.
- (2) ITS 3.1.8 - Limiting Condition for Operation (LCO) item b would be revised to delete an extraneous "≥" symbol.
- (3) ITS 3.2.4 - Required Action A.5, Note 2, would be revised to restore the word "shall," which was inadvertently deleted.
- (4) ITS 3.3.1 - SR 3.3.1.7, Note 2, would be revised to delete an extraneous comma after the word "instrumentation."

- (5) ITS 3.3.5 - Condition F would be revised to delete the word "Logic" from "Actuation Logic Relays" consistent with the terminology used in other areas of the ITS.
- (6) ITS 3.4.14 - SR 3.4.14.1 would be revised to delete the "HV" prefix for the specified residual heat removal (RHR) valves consistent with plant practice.
- (7) ITS 3.7.11 - Required Action E.1.2 would be revised to change "trains" to "train" for grammatical consistency.
- (8) ITS 3.8.3 - Condition A would be revised to change "Modes" to "MODES" in two locations consistent with use of TS definitions.
- (9) ITS 3.8.7 - LCO Note item b would be revised to restore the word "associated" in front of "OPERABLE inverters" consistent with the standard TS wording.
- (10) TS 5.5.16 - The second item "d" would be revised to be item "e."
- (11) ITS 5.6.1 - Double commas would be revised to be a single comma in two locations.
- (12) LCO 3.2.2 - The Completion Times for Required Actions A.1.2.2 and A.2 would be moved to align them with the location of their corresponding Required Actions, consistent with NUREG-1431 format.
- (13) LCO 3.4.11 - The Note for Required Actions in Conditions C and F would be revised to reference "Actions B.2 or E.2" instead of "Actions B.2 or E.3." The Condition "E" Actions had been renumbered but the changes to the note were inadvertently omitted.
- (14) SR 3.7.12.4 - SR 3.7.12.4 would be revised to add a negative sign to the "0.05 inches water gauge," which had been inadvertently deleted.
- (15) LCO 3.7.10 - The Applicability would be revised to add the word "and" after Mode 5, to be consistent with the NUREG-1431 format.
- (16) SR 3.8.4.5 - The units of "ohm" would be added to two resistance values in SR 3.8.4.5, which had been inadvertently omitted.
- (17) LCO 3.9.5 - Consistent with the title of the LCO, the page heading would be corrected to be "RHR and Coolant Circulation - High Water Level" instead of "RHR and Coolant Recirculation - High Water."
- (18) LCO 4.3.1.1 - The word "in" would be deleted from 4.3.1.1.f. because it was duplicative.

The NRC staff has reviewed the licensee's proposed editorial corrections and concludes that the errors occurred with the issuance of Amendment No. 64 to Facility Operating License Nos. NPF-87 and NPF-89, dated February 26, 1999, regarding the conversion to the ITS. The proposed editorial changes correct these inadvertent errors and therefore, are acceptable.

With regard to the TS associated with the SR for the 125 VDC batteries, at the present time SR 3.8.4.7 allows the licensee to perform a modified performance discharge test, instead of the service test, only once in any 60-month period. The requirement for SR 3.8.4.7 states:

The modified performance discharge test in SR 3.8.4.8 may be performed in lieu of the service test in SR 3.8.4.7 once per 60 months.

The licensee has proposed to delete the phrase "once per 60 months" in SR 3.8.4.7. The proposed change to SR 3.8.4.7 would allow the performance of a modified performance discharge test in lieu of a service test at any time.

With regard to battery test recommendations, IEEE-450-1995, "IEEE Recommended Practice for Maintenance, Testing, and Replacement of Vented Lead-Acid Batteries for Stationary Applications," Section 5.4, places no limitation on the use of a modified discharge test in lieu of a service test since the discharge rate is required to envelope the duty cycle of the service test. A modified performance discharge test is a test of the battery's ability to provide a high-rate, short-duration load. This test will often confirm that the battery meets the critical period of the load duty cycle, in addition to determining its percentage of rated capacity. Initial conditions for the modified performance discharge test should be identical to those specified for a service test. A battery service test is a test of battery capability, as found, to satisfy the design requirements (battery duty cycle) of the dc electrical power system. IEEE-450-1995, Section 5.4 states that "[a] modified performance discharge test can be used in lieu of a service test at any time."

The basis for acceptability of allowing the substitution of a modified performance discharge test, for a service test, is that the modified performance discharge test will envelope the service test load demand profile. The proposed Bases for ITS SR 3.8.4.7 adequately describes the modified performance discharge test. The proposed change to SR 3.8.4.7 would provide additional flexibility in allowing the performance of a modified performance discharge test in lieu of a service test at any time. Based on the above considerations, the proposed change to SR 3.8.4.7 is acceptable.

With regard to the emergency DG (EDG) SRs, these SRs address testing that requires the EDGs to start and reach a voltage and frequency within a specified band in a specified period of time (10 seconds). The EDGs actually start and accelerate on full fuel until the governor responds and reduces the fuel amount. On startup, the governor will lag the engine speed by some amount because there is a delay in building up governor oil pressure. As a consequence of this, the EDG may accelerate beyond synchronous speed before the governor responds fully. At this point, the governor may call for "0" fuel and the engine speed will decrease, and may go lower than synchronous speed. There may be several cycles like this before the EDG approaches steady state operation, and the time to reach steady state (within the specified band) may exceed 10 seconds. In such a case, based on literal interpretation of the TSs, the SR would be failed. However, there is nothing wrong with the EDG. Therefore, the licensee has proposed to change the SR requirements to only require EDGs to reach a minimum voltage

and frequency within 10 seconds, and to subsequently achieve steady state operation. The minimum voltage and frequency are those necessary for the EDG to accept load and are established by the licensee/EDG vendor. The time to reach steady state operation, while not part of the TS, is monitored and trended as a means of evaluating continued governor and voltage regulator OPERABILITY.

In summary, the TS should be changed in order to recognize a normal evolution following the start of an EDG (i.e., oscillations in voltage and frequency that may be outside the specified band for greater than 10 seconds.) As stated, this is a normal evolution and is not cause to declare the EDG inoperable. Therefore, the proposed changes to SRs 3.8.1.7, 3.8.1.12, 3.8.1.15, and 3.8.1.20 to separate the voltage and frequency acceptance criteria for the DG start surveillances into two sets of criteria; those criteria required to be met within 10 seconds, and those criteria required to be met following achievement of steady state conditions, are acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Texas State official was notified of the proposed issuance of the amendments. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued proposed findings that the amendments involve no significant hazards consideration, and there has been no public comment on such findings (64 FR 29715 dated June 2, 1999; and 64 FR 35212 dated June 30, 1999). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. H. Jaffe

Date: August 3, 1999