Mr. C. Lance Terry
TU Electric
Group Vice President, Nuclear
Attn: Regulatory Affairs Department
P. O. Box 1002
Glen Rose, TX 76043

December 30, 1997

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION, UNITS 1 AND 2 - AMENDMENT NOS. 56 AND 42 TO FACILITY OPERATING LICENSE NOS. NPF-87 AND NPF-89 (TAC NOS. M96067 AND M96068)

Dear Mr. Terry:

The Commission has issued the enclosed Amendment Nos. 56 and 42 to Facility Operating License Nos. NPF-87 and NPF-89 for the Comanche Peak Steam Electric Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated July 9, 1996 (LAR 96-003, TXXX-96393), as supplemented on December 12, 1997 (TXXX-97268).

The amendments change TS Table 3.3-3, "Engineered Safety Features Actuation System Instrumentation Trip Setpoints." The proposed changes would increase the minimum allowable value of the Unit 1 Steam Line Pressure -- Low Safety Injection and Steam Line Isolation functions. These changes are needed to ensure that the instrumentation error is properly accounted for in the TSs.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,
ORIGINAL SIGNED BY:
Timothy J. Polich, Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosures:
1. Amendment No. 56 to NPF-87
2. Amendment No. 42 to NPF-89
3. Safety Evaluation

cc w/encls: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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The amendments change TS Table 3.3-3, "Engineered Safety Features Actuation System Instrumentation Trip Setpoints." The proposed changes would increase the minimum allowable value of the Unit 1 Steam Line Pressure -- Low Safety Injection and Steam Line Isolation functions. These changes are needed to ensure that the instrumentation error is properly accounted for in the TSs.

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Docket Nos. 50-445 and 50-446

Enclosures:
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2. Amendment No. 42 to NPF-89
3. Safety Evaluation

cc w/encls: See next page

Mr. C. Lance Terry
TU Electric Company

Comanche Peak, Units 1 and 2

cc:

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
P. O. Box 2159
Glen Rose, TX 76403-2159

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011

Mrs. Juanita Ellis, President
Citizens Association for Sound Energy
1426 South Polk
Dallas, TX 75224

Mr. Roger D. Walker
TU Electric
Regulatory Affairs Manager
P. O. Box 1002
Glen Rose, TX 76043

Texas Utilities Electric Company
c/o Bethesda Licensing
3 Metro Center, Suite 810
Bethesda, MD 20814

George L. Edgar, Esq.
Morgan, Lewis & Bockius
1800 M Street, N.W.
Washington, DC 20036-5869

Honorable Dale McPherson
County Judge
P. O. Box 851
Glen Rose, TX 76043

Office of the Governor
ATTN: John Howard, Director
Environmental and Natural
Resources Policy
P. O. Box 12428
Austin, TX 78711

Arthur C. Tate, Director
Division of Compliance & Inspection
Bureau of Radiation Control
Texas Department of Health
1100 West 49th Street
Austin, TX 78756-3189

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TEXAS UTILITIES ELECTRIC COMPANY

COMANCHE PEAK STEAM ELECTRIC STATION, UNIT 1

DOCKET NO. 50-445

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 56
License No. NPF-87

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Texas Utilities Electric Company (TU Electric, the licensee) dated July 9, 1996 (TXXX-96393), as supplemented on December 12, 1997 (TXXX-97268), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-87 is hereby amended to read as follows:

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2. Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 56, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Timothy J. Polich, Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: December 30, 1997

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TEXAS UTILITIES ELECTRIC COMPANY

COMANCHE PEAK STEAM ELECTRIC STATION, UNIT 2

DOCKET NO. 50-446

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 42
License No. NPF-89

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Texas Utilities Electric Company (TU Electric, the licensee) dated July 9, 1996 (TXXX-96393), as supplemented December 12, 1997 (TXXX-97268), complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-89 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 42, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. TU Electric shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Timothy J. Polich, Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: December 30, 1997

ATTACHMENT TO LICENSE AMENDMENT NOS. 56 AND 42

FACILITY OPERATING LICENSE NOS. NPF-87 AND NPF-89

DOCKET NOS. 50-445 AND 50-446

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain marginal lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

REMOVE	INSERT
3/4 3-25	3/4 3-25
3/4 3-27	3/4 3-27

TABLE 3.3-3

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNIT	TRIP SETPOINT	ALLOWABLE VALUE
1. Safety Injection (ECCS, Reactor Trip, Feedwater Isolation, Control Room Emergency Recirculation, Emergency Diesel Generator Operation, Containment Vent Isolation, Station Service Water, Phase A Isolation, Auxiliary Feedwater-Motor Driven Pump, Turbine Trip, Component Cooling Water, Essential Ventilation Systems, and Containment Spray Pump).		
a. Manual Initiation	N.A.	N.A.
b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A
c. Containment Pressure--High 1	≤3.2 psig	≤3.8 psig
d. Pressurizer Pressure--Low	≥1820 psig	≥1803.6 psig
e. Steam Line Pressure--Low		
1) Unit 1	≥605 psig*	≥594.0 psig*
2) Unit 2	≥605 psig*	≥578.4 psig*
2. Containment Spray		
a. Manual Initiation	N.A.	N.A
b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A
c. Containment Pressure--High-3	≤18.2 psig	≤18.8 psig

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNIT	TRIP SETPOINT	ALLOWABLE VALUE
3. Containment Isolation		
a. Phase "A" Isolation		
1) Manual Initiation	N.A.	N.A
2) Automatic Actuation Logic and Actuation Relays	N.A.	N.A
3) Safety Injection	See Item 1. above for all Safety Injection Trip Setpoints and Allowable Values.	
b. Phase "B" Isolation		
1) Manual Initiation	See Item 2.a above. Phase "B" isolation is manually initiated when containment spray function is manually initiated.	
2) Automatic Actuation Logic and Actuation Relays	N.A.	N.A
3) Containment Pressure--High-3	≤18.2 psig	≤18.8 psig
c. Containment Vent Isolation		
1) Manual Initiation	See Items 3.a.1 and 2.a above. Containment Vent Isolation is manuall initiated when Phase "A" isolation function or containment spray function is manually initiated.	
2) Automatic Actuation Logic and Actuation Relays	N.A.	N.A
3) Safety Injection	See Item 1. above for all Safety Injection Trip Setpoints and Allowable Values.	

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNIT	TRIP SETPOINT	ALLOWABLE VALUE
4. Steam Line Isolation		
a. Manual Initiation	N.A.	N.A
b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A
c. Containment Pressure--High-2	≤6.2 psig	≤6.8 psig
d. Steam Line Pressure--Low		
1) Unit 1	≥605 psig*	≥594.0 psig*
2) Unit 2	≥605 psig*	≥578.4 psig*
e. Steam Line Pressure - Negative Rate--High	≤100 psi**	≤178.7 psi**

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNIT	TRIP SETPOINT	ALLOWABLE VALUE
5. Turbine Trip and Feedwater Isolation		
a. Automatic Actuation Logic and Actuation Relays	N.A.	N.A
b. Steam Generator Water Level--High-High		
1) Unit 1	≤82.4% of narrow range instrument span.	≤84.3% of narrow range instrument span
2) Unit 2	≤81.5% of narrow range instrument span.	≤83.5% of narrow range instrument span
c. Safety Injection	See Item 1. above for all Safety Injection Trip Setpoints and Allowable Values.	

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 56 AND 42 TO

FACILITY OPERATING LICENSE NOS. NPF-87 AND NPF-89

TEXAS UTILITIES ELECTRIC COMPANY

COMANCHE PEAK STEAM ELECTRIC STATION, UNITS 1 AND 2

DOCKET NOS. 50-445 AND 50-446

1.0 INTRODUCTION

By application dated July 9, 1996 (TXXX-96393), as supplemented on December 12, 1997 (TXXX-97268), Texas Utilities Electric Company (TU Electric/the licensee) requested changes to the Technical Specifications (TSs) (Appendix A to Facility Operating License Nos. NPF-87 and NPF-89) for the Comanche Peak Steam Electric Station (CPSES), Units 1 and 2. These changes apply to CPSES Unit 1 only. The TS is a common document for both units, the changes to Unit 2 are only administrative. The proposed changes would increase the minimum allowable value for the Steam Line Pressure -- Low. The supplement contains clarifying information and does not change the staff's original proposed no significant hazards determination.

The affected TSs are Table 3.3-3, "Engineered Safety Features Actuation System Instrumentation Trip Setpoints." Specifically, the change increases the minimum allowable value from 593.5 psig to 594 psig for 1.e.1, Steam Line Pressure -- Low, Unit 1 for Functional Unit 1, Safety Injection; and 4.d.1, Steam Line Pressure -- Low, Unit 1 for Functional Unit 4, Steam Line Isolation.

2.0 BACKGROUND

The Steam Line Pressure -- Low and the Pressurizer Pressure -- Low setpoints for the Reactor Trip System (RPS) and the Engineered Safety Features Actuation System (ESFAS) at CPSES are monitored using Barton model 763 and 763A Gage Pressure Electronic Transmitters. International Telephone and Telegraph Barton informed all industry users that they had identified an additional error in the transmitter's signal. This error is incurred from the difference in temperature between the calibration and operational temperatures and impacts the setpoint calculations for the trip setpoints and allowable values for the aforementioned parameters.

The requested setpoint change in the TSs is requested to properly account for the additional error incurred by this temperature difference. The changes are specific to Unit 1 and do not affect Unit 2 because Unit 2 uses Rosemount transmitters instead of Barton transmitters. Additionally, the changes are

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plant specific value changes and therefore, do not represent changes in the improved Standard Technical Specifications for Westinghouse Plants.

3.0 EVALUATION

The Steam Line Pressure -- Low allowable value is used to determine when a instrument channel has drifted beyond its expected range. The trip setpoint is the nominal value used for calibrating a channel during a surveillance. The allowable value is the least conservative value allowed for the channel to still maintain operability in accordance with the assumptions in the accident analysis.

The Steam Line Pressure -- Low trip setpoint is used in two functional units of the ESFAS; Steam Line Isolation and Safety Injection (SI). The ESFAS is initiated when two of the three Steam Line Pressure -- Low channels in any one steam line decreases below the setpoint value. The ESFAS initiation of SI subsequently leads to the actuation of the RPS. Thus, the Steam Line Pressure -- Low feeds into the control logic of both protection systems which protect and mitigate the reactor from damage during accident and transient scenarios.

For the plant licensing accident analysis, the assumed setpoint value used for the Steam Line Pressure -- Low initiation is the least conservative value possible to still mitigate the consequences of an accident. For CPSES, the assumed accident analysis setpoint value is for the initiation of the ESFAS from the Steam Line Pressure -- Low logic sequence. Since the new allowable value of 594 psig for the Steam Line Pressure -- Low setpoint is more conservative than the current allowable value of 593.5 psig and the assumed safety analysis limit value is not changed, the new allowable value is bounded by the previous analysis and does not increase the likelihood or consequences of an accident.

The setpoint methodology used for calculating the Steam Line Pressure -- Low setpoint limits was not altered from the previous methodology used to determine the channel statistical allowance. The additional error incurred from the difference between the calibration temperature and the operational temperature is of comparable origins to the error incurred from the instrument temperature sensitivity effects; therefore, the two sensor temperature effects are added in percent span and included into the total channel statistical allowance. This treatment and methodology are consistent with previously submitted analysis using the Westinghouse methodology.

In summary, the inclusion of the additional temperature error in the calculation of the allowable value for the Steam Line Pressure -- Low setpoint does not impact the safety analysis of the plant. The assumed accident analysis setpoint limit of 380 psig used in the safety analysis is not changed by the change in the allowable value. Therefore, the new allowable value is bounded by the previous safety analysis. The methodology used to incorporate the additional error in the calculation is consistent with previously approved methodology and treats the error as a temperature sensitivity error which is a consistent statistical treatment.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Texas State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The Nuclear Regulatory Commission staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (62 FR 6579). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Undine Shoop

Date: December 30, 1997