

June 15, 1996

Mr. C. Lance Terry
Group Vice President, Nuclear
TU Electric
Energy Plaza
1601 Bryan Street, 12th Floor
Dallas, TX 75201-3411

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION, UNIT 1 AND 2 - CORRECTION TO AMENDMENT NOS. 50 AND 36 RE: MODIFICATION OF ADMINISTRATIVE CONTROL TECHNICAL SPECIFICATION; AND AMENDMENT NOS. 51 AND 37 RE: CHANGE TO TECHNICAL SPECIFICATION CONTAINMENT LEAKAGE RATE TESTING PROGRAM (TAC NOS. M89423, M89424, M90205, M90206, M89423, AND M89424)

Dear Mr. Terry:

On June 12, 1996, the Commission issued Amendment Nos. 50 and 36 to Facility Operating License Nos. NPF-87 and NPF-89 for the Comanche Peak Steam Electric Station, Unit Nos. 1 and 2, respectively. The amendments consisted of changes to the Administrative Control Technical Specifications (TSs). TS page 6-1 included an inadvertently rearranged sentence being inserted during this amendment. Also, TS page 6-2 inadvertently dropped the Unit 2 amendment number.

On June 13, 1996, the Commission issued Amendment Nos. 51 and 37 to Facility Operating License Nos. NPF-87 and NPF-89 for the Comanche Peak Steam Electric Station, Unit Nos. 1 and 2, respectively. The amendments consisted of a revision to the TSs for use of the new containment leakage rate testing program. TS page 6-7 inadvertently labeled paragraph "g" as paragraph "f", that was deleted in Amendment Nos. 50 and 36. Also, on TS page 6-7 an "s" making specification plural was inadvertently dropped.

Copies of the corrected TS pages 6-1, 6-2, and 6-7 for Unit Nos. 1 and 2 are enclosed.

We regret any inconvenience this oversight may have caused. If you have any questions on this action, please call me at (301) 415-2972.

Sincerely,

Original signed by P.M. Ray
Phillip M. Ray, Acting Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosure: Changed TS pages 6-1, 6-2 and 6-7

cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 25, 1996

Mr. C. Lance Terry
Group Vice President, Nuclear
TU Electric
Energy Plaza
1601 Bryan Street, 12th Floor
Dallas, TX 75201-3411

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION, UNIT 1 AND 2 - CORRECTION TO
AMENDMENT NOS. 50 AND 36 RE: MODIFICATION OF ADMINISTRATIVE CONTROL
TECHNICAL SPECIFICATION; AND AMENDMENT NOS. 51 AND 37 RE: CHANGE TO
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Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

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Enclosure: Changed TS pages 6-1, 6-2 and 6-7

cc w/encl: See next page

Mr. C. Lance Terry
TU Electric Company

Comanche Peak, Units 1 and 2

cc:
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6.1 RESPONSIBILITY

6.1.1 The Vice President of Nuclear Operations* shall be responsible for overall operation of the units and shall delegate in writing the succession to this responsibility during his absence.

The Vice President of Nuclear Operations*, or his designee, in accordance with approved administrative procedures, shall approve prior to implementation, each proposed test or experiment and proposed changes and modifications to unit systems or equipment that affect nuclear safety.

6.1.2 The Shift Manager shall be responsible for the control room command function. A management directive to this effect, signed by the Group Vice President, Nuclear Production shall be issued annually to all station personnel. During any absence of the Shift Manager from the control room while the unit is in MODE 1, 2, 3 or 4, an individual with an active Senior Reactor Operator (SRO) license shall be designated to assume the control room command function. During any absence of the Shift Manager from the control room while the unit is in MODE 5 or 6, an individual with an active SRO license or Reactor Operator license shall be designated to assume the control room command function.

6.2 ORGANIZATION

6.2.1 ONSITE AND OFFSITE ORGANIZATION

An onsite and an offsite organization shall be established for unit operation and corporate management, respectively. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility and communication shall be established and defined from the highest management levels through intermediate levels to and including all operating organization positions. Those relationships shall be documented and updated, as appropriate, in the form of organizational charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in the equivalent forms of documentation. These requirements shall be documented in the FSAR.
- b. The Vice President of Nuclear Operations* shall be responsible for overall site safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Group Vice President, Nuclear Production shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.

*Duties may be performed by the Plant Manager if that organizational position is assigned.

COMANCHE PEAK - UNITS 1 AND 2

6-1

Unit 1 - Amendment No. 9,14,50
Unit 2 - Amendment No. 36

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ONSITE AND OFFSITE ORGANIZATION (Continued)

- d. The individuals who train the operating staff and those who carry out the radiation protection and quality assurance functions may report to the appropriate manager onsite; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

6.2.2 UNIT STAFF

The unit organization shall include the following:

- a. An Auxiliary Operator shall be assigned to each reactor containing fuel and an additional Auxiliary Operator shall be assigned if either unit is operating;
- b. At least one licensed Operator for each unit shall be in the control room when fuel is in either reactor. In addition, while either unit is in MODE 1, 2, 3, or 4, at least one licensed Senior Operator shall be in the control room;
- c. A Radiation Protection Technician* and a Chemistry Technician* shall be on site when fuel is in the reactor;
- d. A site Fire Brigade of at least five members* shall be maintained on site at all times. The Fire Brigade shall not include the Shift Manager and the two other members of the minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency;
- e. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions (e.g., licensed Senior Operators, licensed Operators, Radiation Protection Technicians, auxiliary operators, and key maintenance personnel).

The amount of overtime worked by unit staff members performing safety-related functions shall be limited in accordance with the NRC Policy Statement on working hours (Generic Letter No. 82-12);

- f. The Shift Operations Manager shall hold a Senior Reactor Operator license; and

*The Radiation Protection and the Chemistry Technicians and Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours, in order to accommodate unexpected absence, provided immediate action is taken to fill the required positions.

PROCEDURES AND PROGRAMS (Continued)

- 9) Limitations on the annual and quarterly doses to a MEMBER OF THE PUBLIC from Iodine-131, Iodine-133, tritium, and all radio-nuclides in particulate form with half-lives greater than 8 days in gaseous effluents released from each unit to areas beyond the SITE BOUNDARY conforming to Appendix I to 10 CFR 50, and
- 10) Limitations on the annual dose or dose commitment to any MEMBER OF THE PUBLIC due to releases of radioactivity and to radiation from uranium fuel cycle sources conforming to 40 CFR 190.

f. Not used.

g. Containment Leakage Rate Testing Program

A program shall be established to implement the leakage rate testing of the containment as required by 10CFR50.54(o) and 10CFR50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program, dated September 1995".

The peak calculated containment internal pressure for the design basis loss of coolant accident, P_a , is 48.3 psig.

The maximum allowable containment leakage rate, L_a at P_a , shall be 0.10% of containment air weight per day.

Leakage rate acceptance criteria are:

1. Containment leakage rate acceptance criterion is $\leq 1.0 L_a$. During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria are $\leq 0.60 L_a$ for the Type B and Type C tests and $\leq 0.75 L_a$ for Type A tests;
2. Air lock testing acceptance criteria are:
 - a) Overall air lock leakage rate is $\leq 0.05 L_a$ when tested at $\geq P_a$.
 - b) For each door, leakage rate is $\leq 0.01 L_a$ when pressurized to $\geq P_a$.

The provisions of Specification 4.0.2 do not apply to the test frequencies specified in the Containment Leakage Rate Testing Program, with the exception of the containment ventilation isolation valves, which is specified in Specifications 4.6.1.7.2 and 4.6.1.7.3.

The provisions of Specification 4.0.3 are applicable to the Containment Leakage Rate Testing Program.

COMANCHE PEAK - UNITS 1 AND 2

6-7 Unit 1 - Amendment No. ~~14,42,50,51~~
Unit 2 - Amendment No. ~~28,36,37~~